



Oyster Creek
Route 9 South
P.O. Box 388
Forked River, NJ 08731

ASME Code, Section XI
10CFR50.55a

RA-17-014

February 10, 2017

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555

Oyster Creek Nuclear Generating Station
Renewed Facility Operating License No. DPR-16
Docket No. 50-219

Subject: Oyster Creek Nuclear Generating Station (OCNGS) 1R26 Refueling Outage Inservice Inspection (ISI) Form OAR-1 Owner's Activity Report

The purpose of this letter is to provide the Oyster Creek Nuclear Generating Station Owner's Activity Report (OAR-1) for Inservice Inspections as required by the American Society of Mechanical Engineers (ASME) Code, Section XI, 2007 Edition with 2008 Addenda ASME Code Case N-532-4 Repair/Replacement Activity Documentation Requirements and Inservice Summary Report Preparation and Submission – Section XI, Division 1.

Attachment 1 is the Form OAR-1 Owner's Activity Report covering ISI activities associated with the 1R26 Refueling Outage which started on September 19, 2016, and was the first outage of the second period of the current ISI inspection interval.

OCNGS was unable to complete the Form OAR-1 within 90 calendar days of the completion of the refueling outage as required by Code Case N-532-5. This non-compliance was entered into the site corrective action program under Issue Report Number 3961771 and communicated to the NRC per letter RA-16-112 dated January 11, 2017 (Accession Number ML17018A073).

If you should have any questions, please contact the Oyster Creek ISI point of contacts, Valmicky Samlal at 609-971-4112 or Tedd Nickerson at 609-971-2197.

Sincerely,

A handwritten signature in black ink, appearing to read "Garey L. Stathes".

Garey L. Stathes
Vice President
Oyster Creek Nuclear Generating Station

A047
NRR

U. S. Nuclear Regulatory Commission

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Enclosures:

Attachment 1 – Form OAR-1 Owner's Activity Report – 1R26 Outage

cc: USNRC, Regional Administrator, Region I
USNRC, Senior Project Manager, NRR
USNRC, Senior Resident Inspector
Gary Gustofson, Hartford Steam Boiler I&I Co.

FORM OAR-1 OWNER'S ACTIVITY REPORT

Report Number: OC1R26

Plant: Oyster Creek Nuclear Generating Station, US Route 9 South, Forked River, NJ 08731

Unit No: 1 Commercial service date: December 23, 1969 Refueling Outage No: 26
(if applicable)

Current Inspection Interval: ISI = 5th / CISI = 2nd
(1st, 2nd, 3rd, 4th, other)

Current inspection period: ISI = 2nd / CISI = 3rd
(1st, 2nd, 3rd)

Edition and Addenda of Section XI applicable to the inspection plans: 2007 Edition with 2008 Addenda for ISI and 2001 Edition with 2003 Addenda for CISI

Date and revision of inspection plans: 09/02/2015, Revision 2 for ISI and 07/26/2011, Revision 1 for CISI

Edition and Addenda of Section XI applicable to repair/replacement activities, if different than the inspection plans

N/A

Code Cases used for inspection and evaluation: N-432-1, N-513-3, N-516-3, N-526, N-532-5, N-537, N-561-2, N-562-2, N-583, N-586-1, N-597-2, N-600, N-606-1, N-609, N-613-1, N-629, N-638-4, N-639, N-641, N-647, N-648-1, N-649, N-651, N-658, N-661-2, N-663, N-665, N-666, N-683, N-696, N-700, N-705, N-712, N-730, N-733, N-735, N-747, N-762, N-765, N-769, N-773, N-786, and N-789

(if applicable, including cases modified by N-532 and later revisions)

CERTIFICATE OF CONFORMANCE

I certify that (a) the statements made in this report are correct; (b) the examinations and tests meet the Inspection Plan as required by the ASME Code, Section XI; and (c) the repair/replacement activities and evaluations supporting the completion of OC1R26 conform to the requirements of Section XI.

(refueling outage number)

Signed Gene Navratil, ISI Program Owner Gene Navratil Date 2/8/17
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and employed by The Hartford Steam Boiler Inspection and Insurance Company have inspected the items described in this Owner's Activity Report, and state that, to the best of my knowledge and belief, the Owner has performed all activities represented by this report in accordance with the requirements of Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair/replacement activities and evaluation described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personnel injury or property damage or a loss of any kind arising from or connected with this inspection.

Gary Gustofson Gary Gustofson Commission 13855 ANIC NJ948

Inspector's Signature

(National Board Number and Endorsement)

Date 2/8/17

Table 1
Items with Flaws or Relevant Conditions That Required Evaluation for Continued Service

Examination Category and Item Number	Item Description	Evaluation Description
F-A, F1.20C	Containment Spray piping support 241-0080	Support evaluated with one anchor bolt not credited for structural support per Engineering Change 617885.
B-P, B15.10	Mechanical joint leakage identified during Class 1 leakage test	Mechanical joint leakage evaluated as acceptable per IR 3968718.

Table 2
Abstract of Repair/Replacement Activities Required for Continued Service

Code Class	Item Description	Description of Work	Date Completed	Repair/Replacement Plan Number
3	P-3-3A	Replace ESW pump.	02/07/2017	R2131578
1	P-37-4	Replace recirculation pump seal.	02/02/2017	C2036150
3	Y-11-2	Replace expansion joint.	01/29/2017	C2036974
1	V-1-164	Replace main steam relief valve due to failed lift test. Valve lifted at -3.3% of set pressure (-4 psig out of tolerance) and would have fulfilled its function to relieve reactor pressure if required.	01/10/2017	R2181961
1	Control Rod Drive 14-47	Replace control rod drive due to flange joint leak.	02/06/2017	C2036154