

50-220

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TO:

Mr. S. Nowicki

FROM:

Niagara Mohawk Power Corp.
Syracuse, New York
W. R. D'Angelo

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5/6/77

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ACKNOWLEDGED

PLANT NAME:

Nine Mile Point Unit No. 1

RJL

ENCLOSURE

Amdt. to OL/change to Appendix A tech
specs..notorized 5/6/77..concerns request
for change regarding the lowering of the
reactor vessel water level.....

(6-P)

DO NOT REMOVE

SAFETY

FOR ACTION/INFORMATION

ENVIRO

ASSIGNED AD:

BRANCH CHIEF:

PROJECT MANAGER:

LIC. ASST.:

ASSIGNED AD:

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PROJECT MANAGER:

LIC. ASST.:

INTERNAL DISTRIBUTION

REG. FILE

NRC-PDR

I & E (2)

OELD

GOSSICK & STAFF

MIPC

CASE

HANAUER

HARLESS

SYSTEMS SAFETY

HEINEMAN

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ENGINEERING

MACAFAY

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PLANT SYSTEMS

TEDESCO

BENAROYA

LAINAS

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KIRKWOOD

OPERATING REACTORS

STELLO

SITE SAFETY &

ENVIRO ANALYSIS

DENTON & MULLER

ENVIRO TECH:

ERNST

BALLARD

YOUNGBLOOD

SITE TECH.

GAMMILL

STAPP

HULMAN

SITE ANALYSIS

VOLLMER

BUNCH

J. COLLINS

KREGER

EXTERNAL DISTRIBUTION

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LPDR: Oswego, NY

TIC:

NSIC:

ASLB:

ACRS 16 CYS HOLDING/SENT AS CAT B

NAT. LAB:

REG V.IE

LA PDR

CONSULTANTS:

BROOKHAVEN NAT. LAB.

ULRIKSON (ORNL)

771330025



UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

To: Stan Nowicki
7878
From: W.R. D'Amico
Niagara Mohawk

REGULATORY DOCKET FILE COPY

In the Matter of

NIAGARA MOHAWK POWER CORPORATION
(Nine Mile Point Nuclear Station
Unit No. 1)

Docket No. 50-220

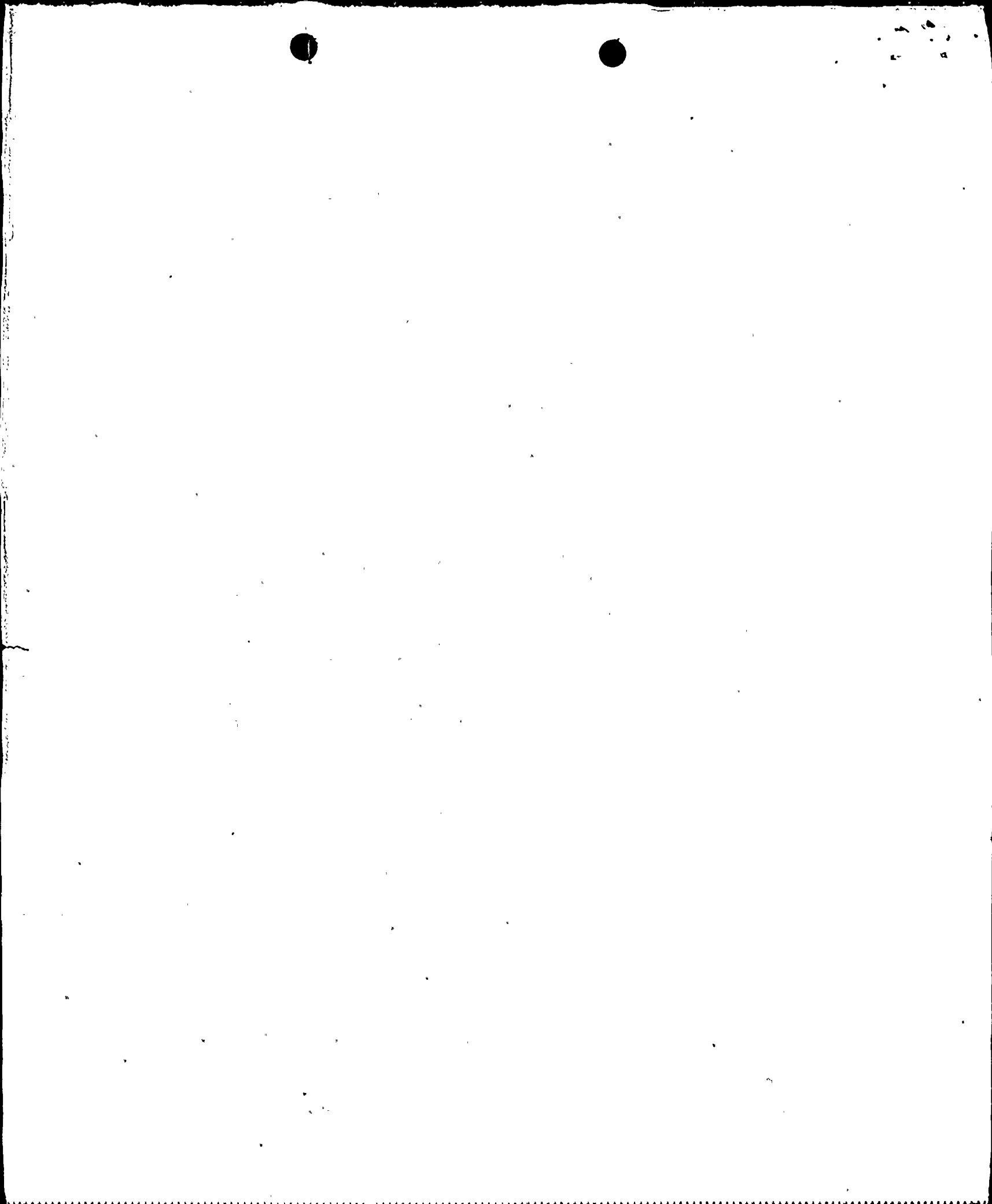
APPLICATION FOR AMENDMENT
TO
OPERATING LICENSE



Pursuant to Section 50.90 of the regulations of the Nuclear Regulatory Commission, Niagara Mohawk Power Corporation, holder of Facility Operating License No. DFR-63, hereby requests that Sections 2.1.1 and 3.1.4 of the Technical Specifications and Bases set forth in Appendix A to that License be amended. This proposed change has been concurred with by the Site Operations Review Committee and Safety Review and Audit Board.

The proposed Technical Specification change is set forth in Attachment A to this application. Supporting Information, which demonstrates that the proposed change does not involve a significant hazards consideration, is set forth in Attachment B. The proposed change would not authorize any change in the types or any increase in the amounts of effluents or any change in the authorized power level of the facility.

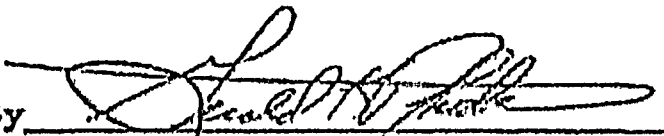
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- 2 -

WHEREFORE, Applicant respectfully requests that
Appendix A to Facility Operating License No. DPR-63 be
amended in the form attached hereto as Attachment A.

NIAGARA MOHAWK POWER CORPORATION

By 
Gerald K. Rhode
Vice President-Engineering

Subscribed and sworn to before
me this 4th day of May, 1977.


NOTARY PUBLIC

PHYLLIS D. VOYTKO
Notary Public in the State of New York
Qualified in Chen, Co. No. 32-942883
My Commission Expires March 30, 1978

TELECOM-BR-DFO5
1977 MAY 6 PM 4 26
U.S. NUCLEAR REGULATORY
COMMISSION

ATTACHMENT A

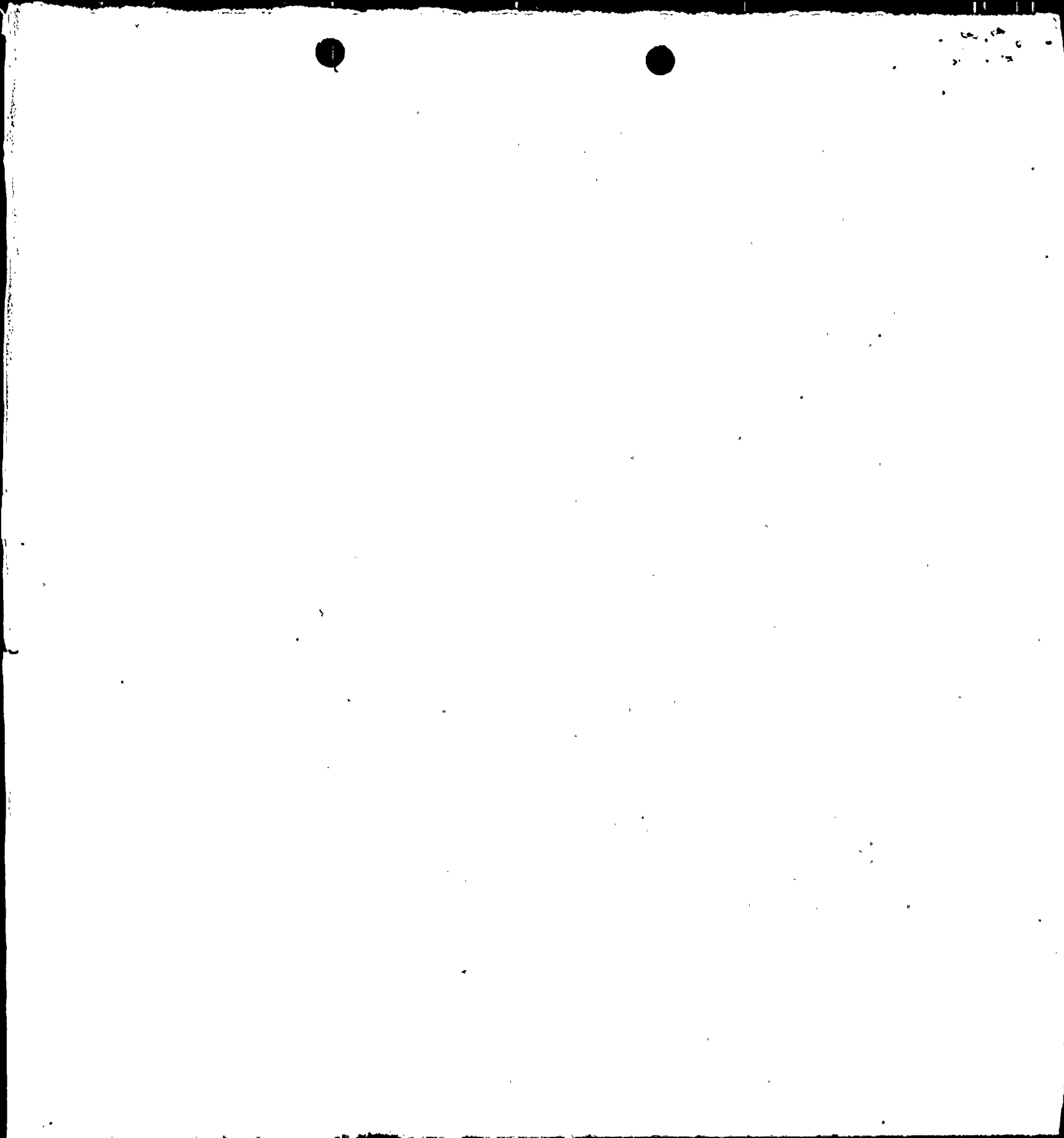
NIAGARA MOHAWK POWER CORPORATION

License No. DPR-63

Docket No. 50-220

Proposed Changes to Technical Specifications

Attached are revisions to Pages 6 and 53a of
Appendix A to DPR-63.



SAFETY LIMIT

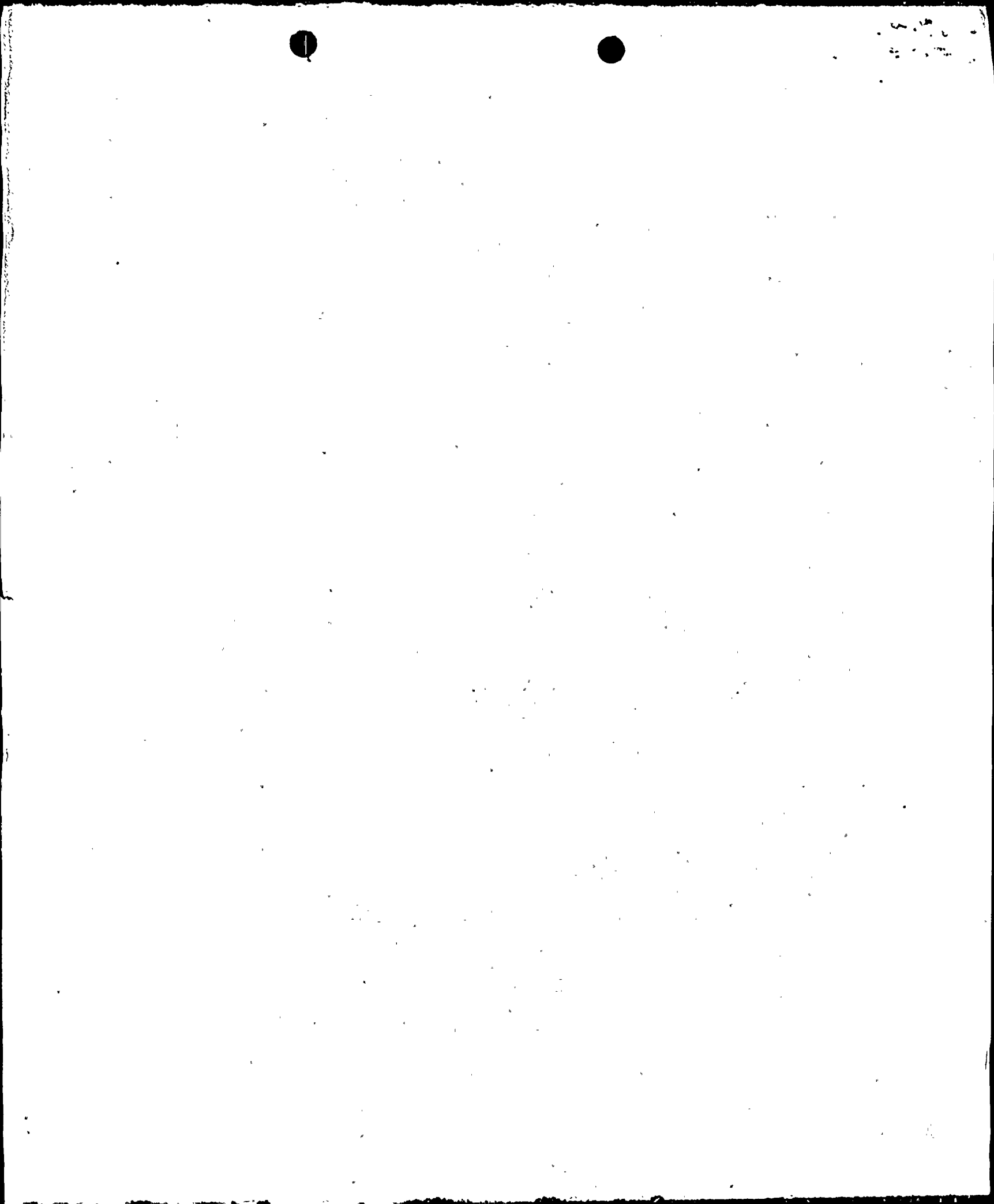
- c. The neutron flux shall not exceed its scram setting for longer than 1.5 seconds as indicated by the process computer. When the process computer is out of service, a safety limit violation shall be assumed if the neutron flux exceeds the scram setting and control rod scram does not occur.

To ensure that the Safety Limit established in Specifications 2.1.1a and 2.1.1b is not exceeded, each required scram shall be initiated by its expected scram signal. The Safety Limit shall be assumed to be exceeded when scram is accomplished by a means other than the expected scram signal.

- d. Whenever the reactor is in the shutdown condition with irradiated fuel in the reactor vessel, the water level shall not be more than 7 feet 11 inches (127.1 inches indicator scale) below minimum normal water level (Elevation 302'9"), except as specified in "e" below.
- e. For the purpose of performing major maintenance (not to exceed 12 weeks in duration) on the reactor vessel, the reactor water level may be lowered 9' below the minimum normal water level (Elevation 302'9"). Whenever the reactor water level is to be lowered below the low-low-low level set point redundant instrumentation will be provided to monitor the reactor water level.

LIMITING SAFETY SYSTEM SETTING

- d. The reactor water low level scram trip setting shall be no lower than -12 inches (53 inches indicator scale) relative to the minimum normal water level (302'9").
- e. The reactor water low-low level setting for core spray initiation shall be no less than -5 feet (5 inches indicator scale) relative to the minimum normal water level (Elevation 302'9").
- f. The flow biased APRM rod block trip settings shall be less than or equal to that shown in Figure 2.1.1.



LIMITING CONDITION FOR OPERATION

SURVEILLANCE REQUIREMENT

1. For the purpose of performing major maintenance (not to exceed 12 weeks in duration) on the reactor vessel, the reactor water level may be lowered to 9' below the minimum normal water level (elevation 302'9"). Whenever the reactor water level is to be lowered below the low-low-low level set point redundant instrumentation will be provided to monitor the reactor water level and written procedures will be developed and followed whenever the reactor water level is lowered below the low-low level set point. The procedures will define the valves that will be used to lower the vessel water level. All other valves that have the potential of lowering the vessel water level will be identified by valve number in the procedures and these valves will be red tagged to preclude their operation during the major maintenance with the water level below the low-low level set point.

During the period of major maintenance requiring lowering the water level to more than 7 feet 11 inches below minimum normal water level (127.1 inches indicator scale), either both Core Spray Systems must be operable or, if one Core Spray System is inoperable because of the maintenance, all of the redundant components of the other Core Spray System must be operable.

Attachment B

NIAGARA MOHAWK POWER CORPORATION

License No. DPR-63

Docket No. 50-220

Supporting Information

Our submittal of May 9, 1977 requested a change which would allow lowering of the reactor vessel water level below the low-low-low level set point. This was required so that maintenance could be performed on the feedwater spargers and nozzles.

The maintenance work on the spargers and nozzles has been slower than anticipated. Also, additional work has been found to be necessary on the control rod drive hydraulic return line. Therefore, a change in the duration from 6 to 12 weeks is necessary.

Since the core decay heat generation is constantly decreasing, the consequences of a LOCA while in the shutdown condition will continue to be less severe.

