

607514

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

FILE NUMBER

TO:

Mr. Benard C. Rusche

FROM:

Niagara Mohawk Power Corp.
Syracuse, New York
R. R. Schneider

DATE OF DOCUMENT

3/2/77

DATE RECEIVED

3/9/77

☒ LETTER☐ NOTORIZED

PROP

INPUT FORM

NUMBER OF COPIES RECEIVED

☐ ORIGINAL
☐ COPY☒ UNCLASSIFIED

One signed

DESCRIPTION

Ltr. furnishing information concerning
Containment Integrated Leak Rate Test..

(2-P)

PLANT NAME:

Nine Mile Point Unit No. 1

RJL

ENCLOSURE

ACKNOWLEDGED
DO NOT REMOVE

SAFETY

FOR ACTION/INFORMATION

ENVIRO

ASSIGNED AD:

BRANCH CHIEF:

PROJECT MANAGER:

LIC. ASST.:

ASSIGNED AD:

BRANCH CHIEF:

PROJECT MANAGER:

LIC. ASST.:

INTERNAL DISTRIBUTION

REG. FILE

SYSTEMS SAFETY

PLANT SYSTEMS

SITE SAFETY

NRC PDR

HEINEMAN

TEDESCO

ENVIRO ANALYSIS

I & E (2)

SCHROEDER

BENAROYA

DENTON & MILLER

OELD

LAINAS

GOSSICK & STAFF

ENGINEERING

IPPOLITO

ENVIRO TECH.

MIPC

MACARRY

KIRKWOOD

ERNST

CASE

BOSNAK

BALLARD

HANAUER

SIUWEIL

OPERATING REACTORS

YOUNGBLOOD

HARLESS

PAWLICKI

STELLO

SITE TECH.

PROJECT MANAGEMENT

REACTOR SAFETY

OPERATING TECH.

GANNILL

BOYD

ROSS

EISENHUT

STAPP

P. COLLINS

NOVAK

SHAO

HULMAN

HOUSTON

ROSZTCCZY

BAER

SITE ANALYSIS

PETERSON

CHECK

BUTLER

VOLLMER

KELTZ

GRIMES

BUNCH

HELTEMES

AT & I

J. COLLINS

SKOVHOLT

SALTZMAN

KREGER

RUTBERG

EXTERNAL DISTRIBUTION

CONTROL NUMBER

LPDR: Oswego, NY

NAT. LAB:

BROOKHAVEN NAT. LAB.

TIC:

REG V. II

ULRIKSON (ORNL)

NSIC:

LA PDR

ASLB:

CONSULTANTS:

ACRS 16 CYS HOLDING/SENT

As CAT B

770750270

ACKNOWLEDGED
DO NOT REMOVE

NIAGARA MOHAWK POWER CORPORATION

NIAGARA  MOHAWK

300 ERIE BOULEVARD, WEST
SYRACUSE, N. Y. 13202

Regulatory Docket File

March 2, 1977

Mr. Bernard C. Rusche
Director, Nuclear Reactor Regulation
United States Nuclear Regulatory Commission
Washington, D.C. 20555



RE: Docket No. 50-220
License No. DPR-63
Containment Integrated Leak Rate Test

Dear Mr. Rusche:

The fourth successful Containment Integrated Leak Rate Test for Nine Mile Point Unit #1 was completed in November 1975. In accordance with Paragraph III.A.6.(a) of Appendix J 10 CFR 50, the following schedule for succeeding Type A tests is submitted for approval:

Second half of 1978
First half of 1983
First half of 1986
First half of 1989
Second half of 1993
Second half of 1996

While establishing conditions for the Primary Containment Integrated Leak Rate Test (Type A Test) of November 1975, it was evident that vessel pressure loss rate was in excess of that which would have been required to assure an acceptable test. Leakage was through the tubes of the containment spray heat exchangers. Repairs were made and an acceptable CILRT was completed. This incident was recorded in the CILRT report submitted in February 1976.

Refueling outages are scheduled for approximately eighteen (18) month intervals starting with September 1975, and consequentially, the ten year Inservice Inspections are scheduled for the first half of 1986 and the second half of 1996. The test schedule presented meets the requirements of Paragraph III D (a) and (b) of Appendix J, 10 CFR 50, for the Type A tests as closely as possible for an eighteen month refueling outage interval.

2549

770750270

Regulatory District File

An accelerated test schedule following the 1975 CILRT is not considered necessary since the source of leakage in the preliminary 1975 test was located and corrected. Local leak tests in accordance with Technical Specifications will be completed during the outage starting March 1977, and in addition, a local leak test of the containment spray heat exchangers will be conducted. These tests and the continuous leak rate monitoring required by Technical Specifications, will verify continuous containment integrity.

A separate detailed summary report of the alledged 1975 pretest was not made in accordance with V.B.3 of Appendix J, 10 CFR 50, since as noted in the original test report, this did not actually constitute a full test during which meaningful quantative data was generated. The loss of containment test pressure was not through a barrier normally subject to local leak rate tests or which would constitute a leakage path during postulated accident conditions. During the interval while repairs were made to the containment spray heat exchangers, no other repairs were made to containment components and approximate containment test pressure was maintained.

Very truly yours,



R.R. Schneider
Vice President -
Electric Production

MAS/mtm

cc: Mr. J.P. O'Reilly
US Nuclear Regulatory Commission
King of Prussia, PA. 19406

CERTIFIED MAIL-RETURN RECEIPT REQUEST

