



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

February 8, 2017

LICENSEE: Entergy Nuclear Operations, Inc.

FACILITY: James A. FitzPatrick Nuclear Power Plant

SUBJECT: SUMMARY OF JANUARY 30, 2017, TELECONFERENCE WITH ENTERGY NUCLEAR OPERATIONS, INC. REGARDING VERBAL AUTHORIZATION FOR RELIEF REQUEST RR-21 FOR THE JAMES A. FITZPATRICK NUCLEAR POWER PLANT (CAC NO. MF9128)

Introduction

By application dated January 27, 2017 (Agencywide Documents Access and Management System Accession No. ML17028A011), Entergy Nuclear Operations, Inc. (Entergy, the licensee) requested approval of a proposed alternative to the requirements of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code), for the James A. FitzPatrick Nuclear Power Plant (FitzPatrick). Specifically, Relief Request RR-21 would allow FitzPatrick to restore the pressure boundary and structural integrity of a dissimilar metal weld by performing a full structural weld overlay based on ASME Code Cases N-504-4 and N-638-4, with certain exceptions. Pursuant to Title 10 of the *Code of Federal Regulations* (10 CFR) 50.55a(z)(1), the licensee requested to use the proposed alternative on the basis that it provides an acceptable level of quality and safety.

This memorandum summarizes the teleconference of January 30, 2017, between the U.S. Nuclear Regulatory Commission (NRC) staff and Entergy regarding the relief request. During this call, the NRC staff provided verbal authorization of the relief request as described below. Participants in the discussion from Entergy included Mark Hawes, David Callen, Robert Penny, Chelsea Gills, Joseph Weicks, and Keith Schoales. Participants from the NRC included Rick Ennis, David Alley, Diane Render, and John Tsao.

Background

As discussed in the licensee's application dated January 27, 2017, the licensee discovered a flaw using ultrasonic testing examination on the "A" residual heat removal low pressure coolant injection loop weld number 24-10-130. The weld is a Class 1 dissimilar metal weld located between a carbon steel valve and an austenitic stainless steel fitting (tee). The licensee would like approval of the use of ASME Code Cases N-504-4 and N-638-4, which were previously approved by the NRC, but there will be some small deviations in Entergy's application. The licensee is currently in Refueling Outage 22, and this weld overlay is required for its startup in February 2017.

Relief Request Telephone Call

The NRC staff discussed the following items during the telephone call with Entergy on January 30, 2017, with respect to the proposed relief request:

1. Technical evaluation read by David Alley, Chief of the Component Performance, Non-Destructive Examination and Testing Branch, Division of Engineering, Office of Nuclear Reactor Regulation (NRR)

By letter dated January 27, 2017, Entergy requested relief from the requirements of the ASME Code, Section XI, IWA-4000, at FitzPatrick.

Pursuant to 10 CFR 50.55a(z)(1), the licensee submitted Relief Request RR-21 for the alternate repair of the degraded dissimilar metal butt weld 24-10-130 of "A" residual heat removal low pressure coolant injection loop piping on the basis that the alternate repair provides an acceptable level of quality and safety.

The licensee proposed to install a weld overlay on the subject weld using modified ASME Code Case N-504-4, "Alternative Rules for Repair of Class 1, 2 and 3 Austenitic Stainless Steel Piping"; Code Case N-638-4, "Similar and Dissimilar Metal Welding Using Ambient Temperature Machine GTAW [Gas Tungsten Arc Welding] Temper Bead Technique"; and ASME Code, Section XI, Appendices VIII and Q. The licensee identified deviations from the provisions of the ASME Code and code cases.

The NRC staff finds that the proposed weld overlay is designed, installed, and inspected consistent with the above ASME Code, Section XI, and Code Cases. The licensee provided adequate justifications for those provisions where deviations were proposed. Therefore, the NRC staff finds that Relief Request RR-21 will provide reasonable assurance that the structural integrity of the weld overlaid subject dissimilar metal butt weld is acceptable for the life of the plant.

2. Authorization read by Rick Ennis, Acting Chief of Plant Licensing Branch I, Division of Operating Reactor Licensing (DORL), NRR

As Acting Chief of Plant Licensing Branch I, DORL, NRR, Rick Ennis agreed with the conclusions of the Component Performance, Non-Destructive Examination and Testing Branch.

The NRC staff concludes that the proposed alternative provides reasonable assurance of structural integrity of the subject dissimilar metal butt weld of the subject residual heat removal piping. As such, the NRC staff finds that the alternate repair provides an acceptable level of quality and safety. Accordingly, the NRC staff concludes that the licensee has adequately addressed all of the regulatory requirements set forth in 10 CFR 50.55a(z)(1). Therefore, as of January 30, 2017, the NRC authorizes the use of Relief Request RR-21 at the James A. FitzPatrick Nuclear Power Plant for the fourth 10-year inservice inspection interval. The weld overlay is acceptable for the remaining life of the plant until the end of the period of extended operation, which expires on October 17, 2034. For the remaining life of the plant, weld 24-10-130 will be inspected in accordance with BWRVIP-75-A: BWR [Boiling-Water Reactor] Vessel and Internals Project, Technical Basis for Revisions to Generic Letter 88-01 Inspection Schedules, paragraph 3.5.1.1, and the ASME Code, Section XI, Appendix Q, Section Q-4300.

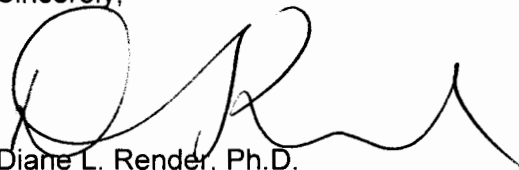
All other requirements of the ASME Code, Section XI, for which relief was not specifically requested and approved in this relief request remain applicable, including third-party review by the Authorized Nuclear Inservice Inspector.

This verbal authorization does not preclude the NRC staff from asking additional clarification questions regarding the relief request while preparing the subsequent written safety evaluation, which will be provided by separate correspondence.

The verbal relief was authorized with the concurrence of Rick Ennis, Acting Chief of Plant Licensing Branch I, and David Alley, Chief of the Component Performance, Non-Destructive Examination, and Testing Branch.

If you have any questions, please contact me at (301) 415-3629 or Diane.Render@nrc.gov.

Sincerely,

A handwritten signature in black ink, appearing to be 'DR' with a stylized flourish at the end.

Diane L. Render, Ph.D.
Project Manager
Plant Licensing Branch I
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-333

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