

SummerRAIsPEm Resource

From: Gleaves, Bill
Sent: Thursday, January 19, 2017 12:29 PM
To: SummerRAIsPEm Resource
Subject: V.C. Summer LAR-16-10 Acceptance Letter (WEC74)
Attachments: V.C. Summer LAR-16-10 Acceptance Letter.docx

Attached is NRC acceptance letter for LAR 16-10 (WEC74), "Shield Building Changes."

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Subject: V.C. Summer LAR-16-10 Acceptance Letter (WEC74)
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From: Gleaves, Bill

Created By: Bill.Gleaves@nrc.gov

Recipients:
"SummerRAIsPEm Resource" <SummerRAIsPEm.Resource@nrc.gov>
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Recipients Received:

Mr. Ronald A. Jones, Vice President
New Nuclear Deployment
P.O. Box 88
Jenkinsville, SC 29065

SUBJECT: ACCEPTANCE REVIEW OF SOUTH CAROLINA ELECTRIC & GAS
COMPANY'S REQUEST FOR LICENSE AMENDMENT AND EXEMPTION FOR
THE VIRGIL C. SUMMER NUCLEAR STATION UNITS 2 AND 3: SHIELD
BUILDING ROOF, aka LAR 16-10 (CAC NO. RG3022)

Dear Mr. Jones:

By letter dated November 21, 2016, (Agencywide Documents Access and Management System (ADAMS) Accession No. ML16326A394), the South Carolina Electric & Gas Company (SCE&G) on behalf of itself and the South Carolina Public Service Authority, submitted a request for a license amendment and exemption for the Virgil C. Summer Nuclear Station (VCSNS), Units 2 and 3 (Combined License (COL) Nos. NPF-93 and NPF-94). In that letter, SCE&G proposes changes to to depart from plant-specific Design Control Document (DCD) Tier 1 information with corresponding changes to the associated COL Appendix C information and Tier 2* information in the Updated Final Safety Analysis Report (UFSAR) (which includes the plant-specific DCD Tier 2 information). Specifically, the changes proposed related to the following: 1) the material from which the shield building radial roof beam are made as well as how they are fabricated, 2) the reinforcements in the roof and walls of the Passive Containment Cooling Water Storage Tank, 3) the design details of the Shield Building tension ring and air inlet structure, 4) Changes to the UFSAR to reflect 1 through 4 above as well as to remove redundant and excessive detail, and 5) changes to to note that the design of the reinforcement in the roof and the design details of the tension ring and air inlets portions of the shield building can vary from these typical design details described in UFSAR. Because this proposed change requires a departure from Tier 1 information in the Westinghouse AP1000 DCD, the licensee also requested an exemption from the requirements of the Generic DCD Tier 1 in accordance with 10 CFR 52.63(b)(1).

The purpose of this letter is to provide the results of the U.S. Nuclear Regulatory Commission (NRC) staff's acceptance review of these requests. The acceptance review was to determine if there was sufficient technical information in scope and depth to allow the NRC staff to complete its detailed technical reviews. The acceptance review was also intended to identify whether the application has any readily apparent information insufficiencies in its characterization of the regulatory requirements or the licensing basis of the plant.

Consistent with Title 10 of the *Code of Federal Regulations* (10 CFR) Section 50.90, an application for an amendment to the license must fully describe the changes requested, and follow as far as applicable, the form prescribed for original applications. 10 CFR Section 52.79 addresses content of technical information that is required for COL application Final Safety Analysis Reports. This section stipulates that the submittal address the design and operating characteristics, unusual or novel design features, and principal safety considerations.

The NRC staff has reviewed your application against the regulatory requirements and has concluded that it provides sufficient technical detail to enable the NRC staff to complete its detailed review and make an independent assessment regarding the acceptability of the proposed changes in terms of regulatory requirements and the protection of public health and

safety and the environment. Given the lesser scope and depth of the acceptance review as compared to the detailed technical review, there may arise issues that impact the NRC staff's ability to complete the detailed technical review despite an adequate acceptance review. You will be advised of any further information needed to support the NRC staff's detailed technical review by separate correspondence.

If you have any questions, please contact me at (301) 415-5848 or Bill.Gleaves@nrc.gov.

Sincerely,

/RA/

William (Billy) Gleaves, Senior Project Manager
Licensing Branch 4
Division of New Reactor Licensing
Office of New Reactors

Docket No(s): 52-027
52-028

cc: See next page