



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**
WASHINGTON, D.C. 20555-0001

January 27, 2017

Mr. John Dent, Jr.
Site Vice President
Entergy Nuclear Operations, Inc.
Pilgrim Nuclear Power Station
600 Rocky Hill Road
Plymouth, MA 02360-5508

**SUBJECT: PILGRIM NUCLEAR POWER STATION – CORRECTION TO TECHNICAL
SPECIFICATION PAGE 5.0-11 (CAC NO. ME3253)**

Dear Mr. Dent:

By letter dated January 10, 2017 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML16363A66), the U.S. Nuclear Regulatory Commission (NRC) issued a correction to address typographical errors in Technical Specification (TS) page 5.0-11 regarding periodic measurement frequency of the control room envelope in TS Section 5.5.8.d. The periodic measurement frequency of the control room envelope (CRE) pressure was to be 24 months instead of 18 months. The measurement frequency of 24 months was approved in Amendment No. 231 (ADAMS Accession No. ML081570366), dated November 20, 2008.

As background, by letter dated January 26, 2011 (ADAMS Accession No. ML110050298), the NRC issued Amendment No. 234 to Facility Operating License No. DPR-35 for the Pilgrim Nuclear Power Station. This amendment consisted of changes to the TSs in response to Entergy Nuclear Operations, Inc.'s application dated January 24, 2010, as supplemented by letters dated September 7, 2010, and November 4, 2010. The amendment revised TSs Sections 1.0, 3.6, 3.6.A, and 5.5 to include reference to the pressure and temperature limits report.

Subsequent to the issuance of the January 10, 2017, correction letter, the NRC staff identified typographical errors in the revised TS page 5.0-11 (for Amendment No. 234), which were inadvertently introduced when staff revised this TS page. In Section 5.5.8.d, the last line should read "the results shall be trended and used as part of the 24 month assessment of the CRE boundary." In Sections 5.5.9.b and 5.5.9.c, the word "NRG" should read "NRC."

J. Dent

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Enclosed is the revised TS page 5.0-11 (for Amendment No. 234). We apologize for any inconvenience.

If you have any questions, please contact me at 301-415-2934 or Booma.Venkataraman@nrc.gov.

Sincerely,

A handwritten signature in black ink, appearing to read "V. Boome", with a horizontal line drawn through the middle of the signature.

Booma Venkataraman, Project Manager
Plant Licensing Branch I
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-293

Enclosure:
Revised TS page 5.0-11 (for
Amendment No. 234)

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ENCLOSURE

Revised TS page 5.0-11 (for Amendment No. 234)

"Demonstrating Control Room Envelope Integrity at Nuclear Power Reactors," Revision 0, May 2003, and (ii) assessing CRE habitability at the Frequencies specified in Sections C.1 and C.2 of Regulatory Guide 1.197, Revision 0.

- d. Measurement, at designated locations, of the CRE pressure relative to all external areas adjacent to the CRE boundary during the pressurization mode of operation by one subsystem of the MCREC System, operating at the flow rate required by the VFTP, at a Frequency of 24 months on a STAGGERED TEST BASIS. The results shall be trended and used as part of the 24 month assessment of the CRE boundary.
- e. The quantitative limits on unfiltered air leakage into the CRE. These limits shall be stated in a manner to allow direct comparison to the unfiltered air leakage measured by the testing described in paragraph c. The unfiltered air leakage limit for radiological challenges is the leakage flow rate assumed in the licensing basis analyses of DBA consequences. Unfiltered air leakage limits for hazardous chemicals must ensure that exposure of CRE occupants to these hazards will be within the assumptions in the licensing basis.
- f. Each Surveillance Requirement shall be performed within the specified SURVEILLANCE INTERVAL with a maximum allowable extension not to exceed 25 percent of the specified SURVEILLANCE INTERVAL. The SURVEILLANCE INTERVAL requirement is applicable to the Frequencies for assessing CRE habitability, determining CRE unfiltered leakage, and measuring CRE pressure and assessing the CRE boundary as required by paragraphs c and d, respectively.

5.5.9 Reactor Coolant System (RCS) Pressure and Temperature Limits Report (PTLR)

- a. RCS pressure and temperature limits for heatup, cool-down, low temperature operation criticality and hydrostatic testing as well as heatup and cool-down rates shall be established and documented in the PTLR for the following:
 - i) Limiting conditions for Operation Section 3.6.A.2
- b. The analytical methods used to determine the RCS pressure and temperature limits shall be those previously reviewed and approved by the NRC, specifically those described in the following document:
 - i) SIR-05-044-A "Pressure-Temperature Limits Report Methodology for Boiling Water Reactors", April 2007
- c. The PTLR shall be provided to the NRC upon issuance for each reactor vessel fluence period and for any reason or supplement thereto.

(continued)

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If you have any questions, please contact me at 301-415-2934 or
Booma.Venkataraman@nrc.gov.

Sincerely,

/RA/

Booma Venkataraman, Project Manager
Plant Licensing Branch I
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