



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

December 23, 2016

Mr. Bryan C. Hanson
Senior Vice President
Exelon Generation Company, LLC
President and Chief Nuclear Officer
Exelon Nuclear
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: QUAD CITIES NUCLEAR POWER STATION, UNITS 1 AND 2 – WITHDRAWAL OF REQUESTED LICENSING ACTION TO REVISE TECHNICAL SPECIFICATION 5.5.12 FOR PERMANENT EXTENSION OF TYPE A AND TYPE C LEAK RATE TEST FREQUENCIES SUBMITTED TO NRC FOR ACCEPTANCE REVIEW (CAC NOS. MF8387 AND MF8388)

Dear Mr. Hanson:

By letter dated September 19, 2016, as supplemented by letter dated December 15, 2016 (Agencywide Documents Access and Management System (ADAMS) Accession Nos. ML16263A267 and ML16350A432, respectively), Exelon Generation Company, LLC submitted a license amendment request for Quad Cities Nuclear Power Station, Units 1 and 2. The proposed amendment request would revise Technical Specification (TS) 5.5.12, "Primary Containment Leakage Rate Testing Program," to allow for the permanent extension of the Type A integrated leak rate testing and Type C leak rate testing frequencies. The purpose of this letter is to provide the results of the U.S. Nuclear Regulatory Commission (NRC) staff's acceptance review of this amendment request. The acceptance review was performed to determine if there is sufficient technical information in scope and depth to allow the NRC staff to complete its detailed technical review. The acceptance review is also intended to identify whether the application has any readily apparent information insufficiencies in its characterization of the regulatory requirements or the licensing basis of the plant.

Consistent with Section 50.90 of Title 10 of the *Code of Federal Regulations* (10 CFR), an amendment to the license (including the TSs) must fully describe the changes requested, and following as far as applicable, the form prescribed for original applications. Section 50.34 of 10 CFR addresses the content of technical information required. This section stipulates that the submittal address the design and operating characteristics, unusual or novel design features, and principal safety considerations.

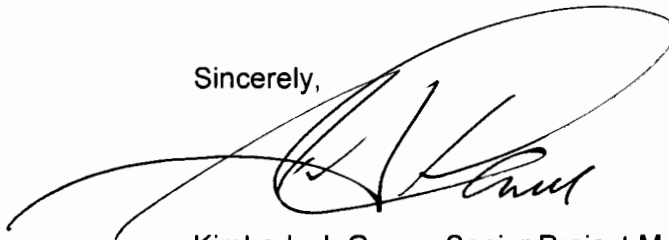
By letter dated December 22, 2016 (ADAMS Accession No. ML16357A541), you requested to withdraw the application from NRC review. The NRC acknowledges your request to withdraw the application. NRC staff activities on the review have ceased, and the associated Cost Activity Code number has been closed.

The NRC staff notes that its review to date has identified that your application did not provide the following technical information in sufficient detail to enable the staff to complete its detailed review. Therefore, if you decide to resubmit the request, it should include the following information:

- The results of a peer review of the internal events probabilistic risk assessment (PRA) that was conducted against all the supporting requirements of the PRA Standard ASME/ANS RA-Sa-2009, and Regulatory Guide 1.200, Revision 2, "An Approach for Determining the Technical Adequacy of Probabilistic Risk Assessment Results for Risk-Informed Activities" (ADAMS Accession No. ML090410014), that were affected by any PRA upgrades;
- A list of the facts and observations (F&Os) from this peer review, with details of their disposition; and
- For any open or unresolved F&Os, justification for why not meeting the corresponding Capability Category I requirements has no impact on the requested licensing action.

If you have any questions, please contact me at (301) 415-1627 or by e-mail to Kimberly.Green@nrc.gov.

Sincerely,



Kimberly J. Green, Senior Project Manager
Plant Licensing Branch III
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Russell Hanson/for

Docket Nos. 50-254 and 50-265

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The NRC staff notes that its review to date has identified that your application did not provide the following technical information in sufficient detail to enable the staff to complete its detailed review. Therefore, if you decide to resubmit the request, it should include the following information:

- The results of a peer review of the internal events probabilistic risk assessment (PRA) that was conducted against all the supporting requirements of the PRA Standard ASME/ANS RA-Sa-2009, and Regulatory Guide 1.200, Revision 2, "An Approach for Determining the Technical Adequacy of Probabilistic Risk Assessment Results for Risk-Informed Activities" (ADAMS Accession No. ML090410014), that were affected by any PRA upgrades;
- A list of the facts and observations (F&Os) from this peer review, with details of their disposition; and
- For any open or unresolved F&Os, justification for why not meeting the corresponding Capability Category I requirements has no impact on the requested licensing action.

If you have any questions, please contact me at (301) 415-1627 or by e-mail to Kimberly.Green@nrc.gov.

Sincerely,

/RA/ Russell Haskell for

Kimberly J. Green, Senior Project Manager
Plant Licensing Branch III
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-254 and 50-265

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***by e-mail dated**

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