



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

January 30, 2017

Mr. Robert Coffey
Site Vice President
NextEra Energy Point Beach, LLC
6610 Nuclear Road
Two Rivers, WI 54241

SUBJECT: POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2 - ISSUANCE OF
AMENDMENTS FOR ELIMINATION OF TECHNICAL SPECIFICATION 3.7.14,
PRIMARY AUXILIARY BUILDING VENTILATION (CAC NOS. MF7261 AND
MF7262)

Dear Mr. Coffey:

The U.S. Nuclear Regulatory Commission has issued the enclosed Amendment Nos. 257 and 261 to Renewed Facility Operating License Nos. DPR-24 and DPR-27 for the Point Beach Nuclear Plant (Point Beach), Units 1 and 2, respectively. The amendments consist of changes to the technical specifications (TSs) in response to your application dated January 15, 2016, as supplemented by your letters dated April 27, and July 27, 2016.

The amendments eliminate TS 3.7.14, Primary Auxiliary Building Ventilation (VNPAB), for Point Beach, Units 1 and 2. The amendments delete TS 3.7.14 VNPAB in its entirety on the basis that the VNPAB is not credited for accident mitigation and, therefore, does not meet the Title 10 of the *Code of Federal Regulations*, Section 50.36, criteria for inclusion in the TS.

A copy of our related safety evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

A handwritten signature in black ink, appearing to read "Mahesh L. Chawla", is positioned above the typed name.

Mahesh L. Chawla, Project Manager
Plant Licensing Branch III
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-266 and 50-301

Enclosures:

1. Amendment No. 257 to DPR-24
2. Amendment No. 261 to DPR-27
3. Safety Evaluation



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NEXTERA ENERGY POINT BEACH, LLC

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT, UNIT 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 257
License No. DPR-24

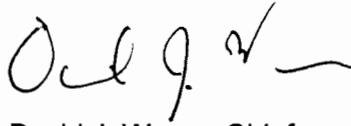
1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by NextEra Energy Point Beach, LLC (the licensee), dated January 15, 2016, as supplemented by letters dated April 27, and July 27, 2016, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 4.B of the Renewed Facility Operating License No. DPR-24 is hereby amended to read as follows:
 - B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 257, are hereby incorporated in the renewed operating license. NextEra Energy Point Beach shall operate the facility in accordance with Technical Specifications.

Enclosure 1

3. This license amendment is effective as of its date of issuance and shall be implemented within 90 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in black ink, appearing to read 'D. J. Wrona', with a stylized flourish at the end.

David J. Wrona, Chief
Plant Licensing Branch III
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the
Technical Specifications and
Renewed Facility Operating License

Date of issuance: January 30, 2017



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NEXTERA ENERGY POINT BEACH, LLC

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT, UNIT 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 261
License No. DPR-27

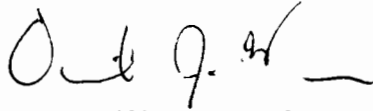
1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by NextEra Energy Point Beach, LLC (the licensee), dated January 15, 2016, as supplemented by letters dated April 27, and July 27, 2016, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 4.B of the Renewed Facility Operating License No. DPR-27 is hereby amended to read as follows:
 - B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 261, are hereby incorporated in the renewed operating license. NextEra Energy Point Beach shall operate the facility in accordance with Technical Specifications.

Enclosure 2

3. This license amendment is effective as of the date of issuance and shall be implemented within 90 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in black ink, appearing to read 'D. J. Wrona', followed by a horizontal line.

David J. Wrona, Chief
Plant Licensing Branch III
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the
Technical Specifications and
Renewed Facility Operating License

Date of issuance: January 30, 2017

ATTACHMENT TO LICENSE AMENDMENT NO. 257
TO RENEWED FACILITY OPERATING LICENSE NO. DPR-24
AND LICENSE AMENDMENT NO. 261
TO RENEWED FACILITY OPERATING LICENSE NO. DPR-27
DOCKET NOS. 50-266 AND 50-301

Replace the following pages of Renewed Facility Operating License Nos. DPR-24 and DPR-27, and Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Renewed Facility Operating License

REMOVE

-3-

INSERT

-3-

Technical Specifications

REMOVE

3.7.14-1

3.7.14-2

INSERT

None

None

- D. Pursuant to the Act and 10 CFR Parts 30, 40 and 70, NextEra Energy Point Beach to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
 - E. Pursuant to the Act and 10 CFR Parts 30 and 70, NextEra Energy Point Beach to possess such byproduct and special nuclear materials as may be produced by the operation of the facility, but not to separate such materials retained within the fuel cladding.
4. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations: 10 CFR Part 20, Section 30.34 of 10 CFR Part 30, Section 40.41 of 10 CFR Part 40, Sections 50.54 and 50.59 of 10 CFR Part 50, and Section 70.32 of 10 CFR Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified below:
- A. Maximum Power Levels

NextEra Energy Point Beach is authorized to operate the facility at reactor core power levels not in excess of 1800 megawatts thermal.
 - B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 257, are hereby incorporated in the renewed operating license. NextEra Energy Point Beach shall operate the facility in accordance with Technical Specifications.
 - C. Spent Fuel Pool Modification

The licensee is authorized to modify the spent fuel storage pool to increase its storage capacity from 351 to 1502 assemblies as described in licensee's application dated March 21, 1978, as supplemented and amended. In the event that the on-site verification check for poison material in the poison assemblies discloses any missing boron plates, the NRC shall be notified and an on-site test on every poison assembly shall be performed.

- C. Pursuant to the Act and 10 CFR Parts 30, 40 and 70, NextEra Energy Point Beach to receive, possess and use at any time any byproduct, source, and special nuclear material as sealed neutron sources for reactor startup, sealed source for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
 - D. Pursuant to the Act and 10 CFR Parts 30, 40 and 70, NextEra Energy Point Beach to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
 - E. Pursuant to the Act and 10 CFR Parts 30 and 70, NextEra Energy Point Beach to possess such byproduct and special nuclear materials as may be produced by the operation of the facility, but not to separate such materials retained within the fuel cladding.
4. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations: 10 CFR Part 20, Section 30.34 of 10 CFR Part 30, Section 40.41 of 10 CFR Part 40, Sections 50.54 and 50.59 of 10 CFR Part 50, and Section 70.32 of 10 CFR Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified below:
- A. Maximum Power Levels

NextEra Energy Point Beach is authorized to operate the facility at reactor core power levels not in excess of 1800 megawatts thermal.
 - B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 261, are hereby incorporated in the renewed operating license. NextEra Energy Point Beach shall operate the facility in accordance with Technical Specifications.
 - C. Spent Fuel Pool Modification

The licensee is authorized to modify the spent fuel storage pool to increase its storage capacity from 351 to 1502 assemblies as described in licensee's application dated March 21, 1978, as supplemented and amended. In the event that the on-site verification check for poison material in the poison assemblies discloses any missing boron plates, the NRC shall be notified and an on-site test on every poison assembly shall be performed.



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WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 257

TO RENEWED FACILITY OPERATING LICENSE NO. DPR-24

AND AMENDMENT NO. 261

TO RENEWED FACILITY OPERATING LICENSE NO. DPR-27

NEXTERA ENERGY POINT BEACH, LLC

POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2

DOCKET NOS. 50-266 AND 50-301

1.0 INTRODUCTION

By letter dated January 15, 2016 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML16015A112), as supplemented by letters dated April 27, 2016 (ADAMS Accession No. ML16118A316), and July 27, 2016 (ADAMS Accession No. ML16209A173), NextEra Energy Point Beach, LLC (NextEra, the licensee) submitted a license amendment request (LAR) to eliminate technical specification (TS) 3.7.14, Primary Auxiliary Building Ventilation (VNPAB), for Point Beach Nuclear Plant (PBNP), Units 1 and 2.

The proposed change would delete TS 3.7.14, VNPAB in its entirety on the basis that the VNPAB is not credited for accident mitigation and, therefore, does not meet Title 10 of the *Code of Federal Regulations* (10 CFR), Section 50.36, criteria for inclusion in the TS. On April 27, 2016, the licensee provided additional clarifying information for the LAR. In response to a U.S. Nuclear Regulatory Commission (NRC or Commission) staff request for additional information (RAI), in an email dated June 27, 2016 (ADAMS Accession No. ML16181A145), the licensee provided supplemental information in a letter dated July 27, 2016.

The licensee's supplemental letters dated, April 27, and July 27, 2016, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the NRC staff's original proposed no significant hazards consideration determination as published in the *Federal Register* (FR) on April 26, 2016 (81 FR 24662).

2.0 REGULATORY EVALUATION

Section 50.36 of 10 CFR, "Technical specifications," Section (c)(2)(ii), states the following:

A technical specification limiting condition for operation of a nuclear reactor must be established for each item meeting one or more of the following criteria:

- (A) *Criterion 1.* Installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary.
- (B) *Criterion 2.* A process variable, design feature, or operating restriction that is an initial condition of a design basis accident or transient analysis that either assumes the failure of or presents a challenge to the integrity of a fission product barrier.
- (C) *Criterion 3.* A structure, system, or component that is part of the primary success path and which functions or actuates to mitigate a design basis accident or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier.
- (D) *Criterion 4.* A structure, system, or component which operating experience or probabilistic risk assessment has shown to be significant to public health and safety.

The NRC's Final Policy Statement on Technical Specifications Improvements for Nuclear Power Reactors is at 58 FR 39132. In reference to the four criteria listed in 10 CFR 50.36(c)(2)(ii), this Policy Statement states: "LCOs [limiting conditions for operation] which do not meet any of [these criteria] may be proposed for removal from the Technical Specifications and relocation to licensee-controlled documents, such as the FSAR [final safety analysis report]." The Policy Statement further states:

If a licensee elects to apply these criteria, the requirements of the removed specifications will be relocated to the FSAR or other licensee-controlled documents. Licensees are to operate their facilities in conformance with the descriptions of their facilities and procedures in their FSAR. Changes to the facility or to procedures described in the FSAR are to be made in accordance with 10 CFR 50.59.

10 CFR 50.67, "Accident source term," establishes acceptance criteria for design basis accident radiological analyses. 10 CFR 50.67 states that the applicant's analysis must demonstrate with reasonable assurance that: (1) an individual located at any point on the boundary of the exclusion area for any 2-hour period following the onset of the postulated fission product release, would not receive a radiation dose in excess of 0.25 sievert (Sv) (25 roentgen equivalent man (rem)) total effective dose equivalent (TEDE); (2) an individual located at any point on the outer boundary of the low population zone, who is exposed to the radioactive cloud resulting from the postulated fission product release during the entire period of its passage,

would not receive a total radiation dose in excess 0.25 Sv (25 rem) TEDE; and (3) adequate radiation protection is provided to permit access to and occupancy of the control room (CR) under accident conditions without personnel receiving radiation exposures in excess of 0.05 Sv (5 rem) TEDE for the duration of the accident.

NRC Regulatory Guide (RG) 1.183, "Alternative Radiological Source Terms for Evaluating Design Basis Accidents at Nuclear Power Reactors" (ADAMS Accession No. ML003716792), provides guidance to licensees of operating power reactors on acceptable applications of alternative source terms (ASTs); the scope, nature, and documentation of associated analyses and evaluations; consideration of impacts on analyzed risk; and content of submittals. This guide establishes an acceptable AST and identifies the significant attributes of other ASTs that may be found acceptable by the NRC staff. This guide also identifies acceptable radiological analysis assumptions for use in conjunction with the accepted AST.

On December 8, 2008 (ADAMS Accession No. ML083450683), the licensee submitted an LAR for approval of a full scope AST for PBNP, Unit 1 and 2. In this LAR the radiological dose consequence analysis for implementation of an AST initially credited the VNPAB exhaust system to mitigate dose to CR personnel. This LAR contained several commitments for plant modifications and procedure changes necessary to support the AST. Since license commitments cannot be relied upon to make a safety determination the commitments that were necessary for a safety determination were converted to license conditions.

On April 17, 2009 (ADAMS Accession No. ML091100182), the licensee provided a supplement to the December 8, 2008, LAR for the proposed TSs for VNPAB. In this supplement, the licensee proposed a new TS as a result of an AST assumption crediting the VNPAB exhaust function. The original AST loss-of-coolant accident (LOCA) emergency core cooling system (ECCS) leakage dose analysis assumed that a portion of the activity would be released to the environment via the primary auxiliary building (PAB) vent stack. Consequently, the VNPAB exhaust met Criterion 3 for establishment of a TS per 10 CFR 50.36(c)(2)(ii), as "A structure, system, or component that is part of the primary success path and which functions or actuates to mitigate a design basis accident or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier."

In response to NRC staff RAIs, the licensee submitted a letter to the NRC dated September 3, 2010 (ADAMS Accession No. ML102460115). In this letter, the licensee described a revised CR dose analysis that did not take credit for the VNPAB system for the LOCA dose consequence analysis. The assumptions in the revised analysis were the same as those used in the AST LAR except that the CR dispersion coefficient (X/Q) value for the ECCS leakage pathway from the PAB was revised to reflect no credit for VNPAB, and flashing fractions for ECCS leakage were revised based on the time dependent sump temperature. This revised analysis showed that the CR dose criterion specified in 10 CFR 50.67 could be met without taking credit for the VNPAB.

On March 25, 2011, the NRC issued Amendment Nos. 238 and 242 to Renewed Facility Operating License Nos. DPR-24 and DPR-27, for PBNP, Units 1 and 2, regarding an auxiliary feedwater system modification (ADAMS Accession No. ML110230016). This included the NRC staff's safety evaluation (SE) of the licensee's analysis regarding the VNPAB's capability to cope with the additional heat loads created by the installation of safety-related motor-driven

auxiliary feedwater (MDAFW) pumps in the 8 foot elevation of the PAB.

On April 14, 2011, the NRC issued Amendment Nos. 240 and 244 to Renewed Facility Operating License Nos. DPR-24 and DPR-27, for PBNP, Units 1 and 2 regarding the application of an AST methodology (ADAMS Accession No. ML110240054). This included the NRC staff's SE of the licensee's radiological analysis assumptions and methods used to support the adoption of the AST. The SE documented the NRC staff's approval of the revised CR dose consequence analysis demonstrating that the CR dose criterion specified in 10 CFR 50.67 could be met without taking credit for the VNPAB.

3.0 TECHNICAL EVALUATION

VNPAB Cooling Capability

The NRC staff notes that there is a licensing basis requirement for the VNPAB system to provide a ventilation (i.e., cooling) function to the installed safety-related MDAFW pumps and their 350 horse-power motors which are located in separate rooms in the 8 foot elevation of the PAB. This requirement was credited in Section 3.3.8 of the NRC SE dated March 25, 2011, for PBNP Unit 1 Amendment No. 238, and Unit 2 Amendment No. 242. In support of this requirement, the "System Evaluation" subsection of the PBNP updated final safety analysis report (UFSAR), Chapter 9.5, "Primary Auxiliary Building Ventilation System," states in part, "Restoration of the VNPAB system within two hours of a LOOP [loss of offsite power] assures adequate cooling for PAB safety related equipment during the worst case design basis accident."

The licensee, in its letter dated July 27, 2016, provided assurance to the NRC staff that the Emergency Operating Procedures and the System Operating Instructions will continue to include the restoration of the VNPAB system in support of the above UFSAR statement. Furthermore, the licensee stated that, "The Augmented Quality VNPAB components, that support the above statement from UFSAR Section 9.5, will retain their Augmented Quality status." In support of this latter statement, the licensee indicated that, following the approval of the LAR the VNPAB system, will remain within the scope of the Maintenance Rule (10 CFR 50.65) and the License Renewal Program (10 CFR 54.37(b)).

VNPAB Filtration and Exhaust Capability

The LAR states that, "The VNPAB system is classified as non-safety related; however, components in the exhaust system required to direct radioactive releases in the auxiliary building to the vent stack are classified as Augmented Quality. The VNPAB exhaust system design provides redundancy for all active mechanical components and active and passive electrical components needed to provide PAB exhaust flow."

The PBNP, UFSAR, Section 9.5.3, states that, "No credit is given for the VNPAB exhaust system in the control room or offsite dose bounding analysis described in [UFSAR] Chapter 14.3.5, Radiological Consequences of a Loss of Coolant Accident."

As stated in the regulatory evaluation the initial AST radiological analyses took credit for the VNPAB. In response to a September 3, 2010, NRC staff RAI, the licensee provided subsequent

analyses which demonstrated that the acceptance criterion described in 10 CFR 50.67 could be met without taking credit for operation of the VNPAB. The NRC staff reviewed the documents cited in the regulatory evaluation and concludes that since the VNPAB is not credited in the current AST dose consequence analyses the operability requirements specified in TS 3.7.14 are not required to meet the acceptance criteria specified in 10 CFR 50.67. Consequently, the VNPAB no longer meets Criterion 3 for establishment of a TS per 10 CFR 50.36(c)(2)(ii) and can be eliminated from the TS in accordance with the Final Policy Statement on Technical Specifications for Nuclear Power Reactors. Consistent with the Final Policy Statement, the licensee confirmed in its July 27, 2016 supplement that the operation of the VNPAB, and the restoration of the system within 2 hours of a LOOP will continue to be directed by licensee-controlled documents, and that future changes to the facility will be made in accordance with 10 CFR 50.59.

The AST implementation as described and approved in Amendment Nos. 240 and 244 included credit for the following operator action:

Manual operator action to restore the VNPAB will occur within 30 minutes following the alignment of RHR [residual heat removal] to containment sump recirculation mode of operation. If a loss of coolant accident (LOCA) occurs coincident with a loss of off-site power (LOOP), the VNPAB will be manually restarted to ensure that the auxiliary building vent stack is the source of the release associated with the emergency core cooling system (ECCS) leakage phase of the event.

Credit for this manual action was necessary to support the initial AST dose consequence analyses that took credit for the VNPAB. Since the current AST analyses no longer credit the VNPAB the NRC staff concludes that these previously credited operator actions related to operation of the VNPAB system can be deleted.

The AST implementation also committed Point Beach to the following administrative controls:

Administrative controls will be established to ensure that CREFS [control room emergency filtration system] and the primary auxiliary building ventilation (VNPAB) system will not be in concurrent Technical Specification Action Conditions (TSACs) during planned preventive maintenance activities on components of the CREFS and VNPAB systems. These administrative controls are not applicable to planned preventive maintenance activities performed on common support system components.

Since the NRC staff has concluded that TS 3.7.14 can be eliminated, the commitment for administrative controls to ensure that the CREFS and the VNPAB will not be in concurrent TS action conditions will no longer be necessary and can therefore be eliminated.

4.0 SUMMARY

As described above, the NRC staff reviewed the documents cited in the regulatory evaluation and concludes that the VNPAB is not credited in the current AST dose consequence analyses. Therefore the operability requirements specified in TS 3.7.14 are not required and this TS can be eliminated. In addition, the associated manual actions related to the operation of the VNPAB within 30 minutes following the alignment of residual heat removal to containment sump

recirculation mode of operation, as well as the administrative controls to ensure the CREFS and the VNPAB will not be in concurrent TS action conditions, will no longer be necessary and can, therefore, be eliminated. The NRC staff further finds with reasonable assurance that PBNP, Units 1 and 2, as modified by this LAR, will continue to provide sufficient safety margins with adequate defense-in-depth to address unanticipated events and to compensate for uncertainties in accident progression and analysis assumptions and parameters. Therefore, the proposed LAR is acceptable with regard to the radiological consequences of postulated design basis accidents.

5.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Wisconsin State official was notified of the proposed issuance of the amendments. The State official had no comments.

6.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or change a surveillance requirement. The NRC staff has determined that the amendments involve no significant increase in the amounts and no significant change in the types of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding (81 FR 24662). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

7.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: John Parillo, NRR/DRA/ARCB
David Nold, NRR/DSS/SBPB

Date of issuance: January 30, 2017

R. Coffey

-2-

POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2 - ISSUANCE OF AMENDMENTS FOR ELIMINATION OF TECHNICAL SPECIFICATION 3.7.14, PRIMARY AUXILIARY BUILDING VENTILATION (CAC NOS. MF7261 AND MF7262)

DATED: January 30, 2017

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ADAMS Accession No.: ML16349A080

***via memorandum**

OFFICE	DORL/LPL3/PM	DORL/LPL3/LA	DRA/ARCB/BC*	DSS/SBPB/BC*
NAME	MChawla	SRohrer	UShoop	RDennig
DATE	12/16/2016	12/14/2016	11/08/2016	11/08/2016
OFFICE	DSS/ITSB/BC	OGC - NLO w/ comments	DORL/LPL3/BC	DORL/LPL3/PM
NAME	AKlein	JWachutka	DWrona	MChawla
DATE	12/16/2016	12/21/2016	1/30/2017	1/30/2017

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