



Public Meeting:

Issues Paper on Revisions to Transportation Safety Requirements and Harmonization with IAEA Transportation Requirements

December 5, 2016

A decorative blue graphic is located in the bottom right corner. It features a stylized atom symbol with a light blue nucleus and three white elliptical orbits. Overlaid on the orbits is the text 'Division of Spent Fuel Management' in a white serif font, arranged in a circular path. In the center of the orbits, the acronym 'NMSS' is written in a bold, white sans-serif font.

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Slide #1

12/5/16 Afternoon Agenda



- 1:00 – 1:35pm
 - Reduced external pressure design requirement for transportation packages
 - Solar insolation
- 1:35 – 1:50pm - BREAK
- 1:50 – Close
 - NRC staff-identified items
 - Adequate space for liquid expansion clarification
 - Quality assurance program clarification
 - Fissile clarifications
 - U.S. Department of Transportation items
 - Additional time for clarifying questions
 - Closing

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Issue 2

Reduced External Pressure Design Requirement for Transportation Packages

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Public Meeting

December 5, 2016

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Slide #3

Reduced External Pressure

Background



- SSG-26 (Para 645) “The containment system shall retain its radioactive contents under a reduction of ambient pressure to 60 kPa.”
- US DOT
 - July 2014 (79 FR 40590)
 - From 25 kPa (3.6 psia)
 - To 60 kPa (8.7 psia)
 - Harmonized with:
 - TS-R-1 (2009)
 - SSR-6 (2012)

The logo for the Nuclear Material Safety and Security (NMSS) Division of Spent Fuel Management. It features a stylized atomic symbol with three orbiting electrons in blue, green, and red. To the right of the symbol, the text "NMSS" is written in a large, bold, serif font. Below this, in a smaller, sans-serif font, are the words "Division of Spent Fuel Management".

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Reduced External Pressure Background



10 CFR 71.71(c)(3)

- Current
 - 25 kPa (3.6 psia)
- Proposed
 - 60 kPa (8.7 psia)

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Reduced External Pressure

Factors for Consideration



What will be the impact to package design requirements if the reduced external pressure is changed from 25 kPa (3.5 lbf/in²) to 60 kPa (8.7 psia)?

Reduced External Pressure

Proposed Actions



In 10 CFR 71.71(c)(3) - revise:

(3) Reduced external pressure. An external pressure of 60 kPa (8.7 psia).



Issue 4

Solar Insolation

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Slide #8

Solar Insolation

Topics

- Units
- Initial condition for fire test

Units for Solar Insolation

Background

- Safety Series No. 6 - 1985 Edition (as amended in 1990) changed the units for solar insolation from “*g-cal/cm² for 12 hours per day*” to “*W/m² for 12 hours per day*”
- NRC units remain g-cal/cm² for 12 hour period
- Total solar insolation applied by the IAEA regulations is 3% higher than NRC’s

Units for Solar Insolation Issue



Should the NRC change the units in Part 71 for solar insolation from g-cal/cm² to be consistent with the IAEA units of W/m²?

Units for Solar Insolation

Factors for Consideration

- Implications for the NRC certificate holders for adopting the IAEA solar insolation
 - Different thermal evaluation using the IAEA solar insolation values when seeking revalidation of the DOT Certificate of Competent Authority?
 - Other?
- Should the NRC adopt the IAEA units for solar insolation?

Units for Solar Insolation

Proposed Action

Form and location of surface	Total insolation for a 12 hour period (W/m²)
Flat surfaces transported horizontally;	
Base	None
Other Surfaces	800
Flat surfaces not transported horizontally	200
Curved surfaces	400

Fire Test Initial Conditions

Background



- IAEA, SSR-6
 - Thermal equilibrium
 - 38 °C
 - Solar insolation
- NRC, 10 CFR 71.73
 - Temperature between -29 °C and 38 °C
 - Maximum decay heat



Fire Test Initial Conditions Issue



Should NRC add solar insolation as an initial condition to the HAC fire test in 10 CFR 71.73?

Fire Test Initial Conditions

Factors for Consideration



Do NRC certificate holders perform a different thermal evaluation using the IAEA solar insolation values as an initial condition for the fire test when seeking revalidation of the DOT Certificate of Competent Authority?

Fire Test Initial Conditions

Proposed Action

Add to 71.73:

the ambient air temperature before and after the tests must remain constant at that value between -29°C (-20°F) and $+38^{\circ}\text{C}$ ($+100^{\circ}\text{F}$), with solar isolation according to the Insolation Data Table in 71.71, which is most unfavorable for the feature under consideration.

BREAK



Issue 11

Adequate Space for Liquid Expansion

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Slide #19

Space for Liquid Expansion

Background



- 10 CFR 71.87(d) requires licensees to ensure adequate ullage or other provision for liquid expansion
- DOT has similar requirement 49 CFR 173.24(h)

Space for Liquid Expansion Issue



- DOT 49 CFR 173.412(k) requirements for Type A packages:

*“Each packaging designed for liquids will—
(1) Be designed to provide for ullage to
accommodate variations in temperature of
the contents, dynamic effects and filling
dynamics”*

- NRC does not have a similar regulation for package design



Space for Liquid Expansion

Factors for Consideration



Should the NRC design requirements for all packages (10 CFR 71.43) require sufficient ullage for liquid expansion?

Space for Liquid Expansion

Proposed Actions



Add to 10 CFR 71.43:

- (i) A package must be designed, constructed, and prepared for shipment so that under the tests specified in § 71.71 ("Normal conditions of transport") § 71.73 ("Hypothetical accident conditions") neither leakage nor permanent distortion of the packaging or system containing the liquid will occur as a result of an expansion of the liquid



Issue 12

Quality Assurance Program (QAP), §71.106, Clarification

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Slide #24

QAP Clarification

Background



- §71.106 added in 2015 Part 71 rulemaking
 - Applies to QAP changes and includes associated reporting requirements
- Wording in regulation is not as clear as the language provided in the Statements of Consideration (SOCs) or associated guidance
- Questions raised by industry



QAP Clarification

Issue



- Regulation states that changes made to the QAP must be submitted to the NRC every 24-months
- SOC's state that if no changes are made, a report is required to be submitted stating no changes were made
 - Similar language is reflected in Regulatory Guide (RG) 7.10 that accompanied this rule

QAP Clarification

Issue



- Part 71 regulation intended to be similar to Part 50 (§ § 50.54(a)(3) and 50.71(e)(2))
 - Language is essentially the same between § § 50.54(a)(3) and 71.106(b) except § 50.71(e)(2) requires a submittal if no changes were made
- NRC considering clarifying § 71.106 language to clearly align with SOC's, RG 7.10 and Part 50

QAP Clarification

Factors for Consideration

- Should QAP approval holders be required to submit a biennial report even if no changes were made during the reporting period?
- Is there an alternative to stating the requirement in § 71.106?

QAP Clarification

Proposed Action



In § 71.106(b): add language to clarify that a biennial report must be submitted to the NRC even if no changes are made to the QAP during the reporting period



Issue 13

Fissile Clarification: Fissile Material General Licenses

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Slide #30

General Licenses Issue



General Licenses in 10 CFR 71.22 and 71.23 are limited to Type A quantity of material in a Type A package

- Restriction is not consistent with the mass limits for some fissile nuclides
 - 37 gram limit for ^{239}Pu in Table 71-1 corresponds to a mass more than 85 times the A_2 value (0.435 grams)
 - 240 gram limit for ^{239}Pu and ^{241}Pu in 10 CFR 71.23 is more than 21 times the A_1 value for ^{241}Pu (11 grams)

General Licenses

Factors for Consideration



Should the NRC correct the 10 CFR 71.22 and 71.23 criteria to remove the restriction that the material be limited to a Type A quantity, and state that the material must be shipped in a Type A or Type B package?

General Licenses

Proposed Actions



- Revise 10 CFR 71.22(a) and 71.23(a) to state that the material must be in a Type A or Type B package, consistent with the radiological and containment requirements of 10 CFR Part 71
- Remove the restriction in 10 CFR 71.22(c)(1) and 71.23(c)(1) that the material be limited to a Type A quantity



Issue 14

Fissile Clarification: ^{233}U Restriction in 10 CFR 71.22

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Slide #34

^{233}U Restriction Issue



Table 71-2 of the general license in 10 CFR 71.22 cannot be used if “Uranium-233 is present in the package,” according to 10 CFR 71.22(e)(5)(i)

- Initial intent was to limit ^{233}U to levels below the detection limit of existing methods
- Possible to detect ^{233}U at a much lower level than previous equipment was capable of detecting; prevents the use of this general license for some material with very low levels of ^{233}U
- Could be modified to indicate that ^{233}U must be less than 1.0 percent of the mass of ^{235}U , consistent with how ^{233}U is limited in the fissile exemption in 10 CFR 71.15(d)

^{233}U Restriction

Factors for Consideration



Should the NRC limit ^{233}U to less than 1.0 percent of the mass of ^{235}U when using table 71-2 of 10 CFR 71.22?

^{233}U Restriction

Proposed Actions



Revise 10 CFR 71.22(e)(5)(i) to replace “Uranium-233 is present in the package;” with “The mass of uranium-233 exceeds 1 percent of the mass of uranium-235.”

U.S. Department of Transportation Items

Public Comment Period



- Issues paper (ML16299A298)
- FRN (81 FR 83171): 60 day period
 - November 21, 2016 through January 20, 2017
- Electronically on Federal Rulemaking Website:
<http://www.regulations.gov>, **Docket ID NRC-2016-0179**
- Mail comments to: Cindy Bladey, Chief, Rules, Announcements, and Directives Branch (RADB), Division of Administrative Services, Office of Administration, Mail Stop: OWFN-12-H08, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001



Abbreviations

- CFR – *Code of Federal Regulations*
- CSI – Criticality Safety Index
- DOT – U.S. Department of Transportation
- DSFM – Division of Spent Fuel Management
- HAC – Hypothetical Accident Conditions
- IAEA – International Atomic Energy Agency
- IP – Industrial Package
- LSA – Low Specific Activity
- MOU – Memorandum of Understanding
- NRC – Nuclear Regulatory Commission
- ORNL – Oak Ridge National Lab
- QAP – Quality Assurance Program
- RADB – Rules, Announcements and Directives Branch
- RG – Regulatory Guide
- SCO – Surface Contaminated Object
- SOC – Statement of Consideration
- SRM – Staff Requirements Memorandum
- TI – Transport Index