

## NRR-PMDAPEm Resource

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**From:** Purnell, Blake  
**Sent:** Monday, November 21, 2016 10:07 AM  
**To:** Hanson, Stephanie J.:(GenCo-Nuc)  
**Cc:** Miller, Ed; Barstow, James:(GenCo-Nuc)  
**Subject:** Exelon Fleet Request for an Alternative to Examination of Threads in Flange (CAC Nos. MF8712-MF8729)

Ms. Hanson:

By application dated October 31, 2016 (ADAMS Accession No. ML16306A270), Exelon Generation Company, LLC (the licensee) submitted a request in accordance with Paragraph 50.55a(z)(1) of Title 10 of the *Code of Federal Regulations* (10 CFR) for a proposed alternative to the requirements of 10 CFR 50.55a and the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code (ASME B&PV Code) for Braidwood Station, Units 1 and 2; Byron Station, Units 1 and 2; Calvert Cliffs Nuclear Power Plant, Units 1 and 2; Dresden Nuclear Power Station, Units 2 and 3; Limerick Generating Station, Units 1 and 2; Nine Mile Point Nuclear Station, Units 1 and 2; Peach Bottom Atomic Power Station, Units 2 and 3; Quad Cities Nuclear Power Station, Units 1 and 2; R. E. Ginna Nuclear Power Plant; and Three Mile Island Nuclear Station, Unit 1. The proposed alternative would allow the licensee to no longer perform the examination of reactor pressure vessel threads in the flange required ASME B&PV Code, Section XI, Examination Category B-G-1, Item Number B6.40.

The purpose of this email is to provide the results of the U.S. Nuclear Regulatory Commission (NRC) staff's acceptance review of this proposed alternative. The acceptance review was performed to determine if there is sufficient technical information in scope and depth to allow the NRC staff to complete its detailed technical review. The acceptance review is also intended to identify whether the application has any readily apparent information insufficiencies in its characterization of the regulatory requirements or the licensing basis of the plant.

Pursuant to 10 CFR 50.55a(z), the applicant shall demonstrate that the proposed alternatives would provide an acceptable level of quality and safety, or that compliance with the specified requirements of 10 CFR 50.55a would result in hardship or unusual difficulty without a compensating increase in the level of quality or safety.

The NRC staff has reviewed your application and concluded that it provides technical information in sufficient detail to enable the staff to complete its detailed technical review and make an independent assessment regarding the acceptability of the proposed relief request in terms of regulatory requirements and the protection of public health and safety and the environment. Given the lesser scope and depth of the acceptance review, as compared to the detailed technical review, there may be instances in which issues that impact the staff's ability to complete the detailed technical review are identified despite completion of an adequate acceptance review. You will be advised of any further information needed to support the staff's detailed technical review by separate correspondence.

Sincerely,

Blake Purnell, Project Manager  
Plant Licensing Branch III-2  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation  
U.S. Nuclear Regulatory Commission

Docket Nos. STN 50-456, STN 50-457, STN 50-454,

STN 50-455, 50-317, 50-318, 50-237,  
50-249, 50-352, 50-353, 50-220, 50-410,  
50-277, 50-278, 50-254, 50-265, 50-244,  
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(CAC Nos. MF8712-MF8729)  
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**From:** Purnell, Blake

**Created By:** Blake.Purnell@nrc.gov

**Recipients:**

"Miller, Ed" <Ed.Miller@nrc.gov>

Tracking Status: None

"Barstow, James:(GenCo-Nuc)" <James.Barstow@exeloncorp.com>

Tracking Status: None

"Hanson, Stephanie J.:(GenCo-Nuc)" <Stephanie.Hanson@exeloncorp.com>

Tracking Status: None

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