

ATTACHMENT TO LICENSE AMENDMENT NO. 66

TO FACILITY COMBINED LICENSE NO. NPF-92

DOCKET NO. 52-026

Replace the following pages of the Facility Combined License No. NPF-92 with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Facility Combined License No. NPF-92

REMOVE

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Appendix C to Facility Combined License No. NPF-92

REMOVE

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(7) Reporting Requirements

- (a) Within 30 days of a change to the initial test program described in FSAR Section 14, Initial Test Program, made in accordance with 10 CFR 50.59 or in accordance with 10 CFR Part 52, Appendix D, Section VIII, "Processes for Changes and Departures," SNC shall report the change to the Director of NRO, or the Director's designee, in accordance with 10 CFR 50.59(d).
- (b) SNC shall report any violation of a requirement in Section 2.D.(3), Section 2.D.(4), Section 2.D.(5), and Section 2.D.(6) of this license within 24 hours. Initial notification shall be made to the NRC Operations Center in accordance with 10 CFR 50.72, with written follow up in accordance with 10 CFR 50.73.

(8) Incorporation

The Technical Specifications, Environmental Protection Plan, and ITAAC in Appendices A, B, and C, respectively of this license, as revised through Amendment No. 66, are hereby incorporated into this license. |

(9) Technical Specifications

The technical specifications in Appendix A to this license become effective upon a Commission finding that the acceptance criteria in this license (ITAAC) are met in accordance with 10 CFR 52.103(g).

(10) Operational Program Implementation

SNC shall implement the programs or portions of programs identified below, on or before the date SNC achieves the following milestones:

- (a) Environmental Qualification Program implemented before initial fuel load;
- (b) Reactor Vessel Material Surveillance Program implemented before initial criticality;
- (c) Preservice Testing Program implemented before initial fuel load;
- (d) Containment Leakage Rate Testing Program implemented before initial fuel load;
- (e) Fire Protection Program
 - 1. The fire protection measures in accordance with Regulatory Guide (RG) 1.189 for designated storage building areas (including adjacent fire areas that could affect the storage area) implemented before initial receipt

| Table 2.2.3-4 Inspections, Tests, Analyses, and Acceptance Criteria | | | | |
|--|-----------------|---|---|--|
| No. | ITAAC No. | Design Commitment | Inspections, Tests, Analyses | Acceptance Criteria |
| 197 | 2.2.03.08c.xii | 8.c) The PXS provides RCS makeup, boration, and safety injection during design basis events. | xii) Inspections will be conducted of the CMT level sensors (PXS-11A/B/D/C, - 12A/B/C/D, - 13A/B/C/D, - 14A/B/C/D) upper level tap lines. | xii) Each upper level tap line has a downward slope of ≥ 2.4 degrees from the centerline of the connection to the CMT to the centerline of the connection to the standpipe. |
| 198 | 2.2.03.08c.xiii | 8.c) The PXS provides RCS makeup, boration, and safety injection during design basis events. | xiii) Inspections will be conducted of the surfaces in the vicinity of the containment recirculation screens. The surfaces in the vicinity of the containment recirculation screens are the surfaces located above the bottom of the recirculation screens up to and including the bottom surface of the plate discussed in Table 2.2.3-4, item 8.c.vii, out at least 8 ft, 3 in perpendicular to the front and at least 7 feet to the side of the face of the screens. | xiii) These surfaces are stainless steel. |
| 199 | 2.2.03.08c.xiv | 8.c) The PXS provides RCS makeup, boration, and safety injection during design basis events. | xiv) Inspection will be conducted of the excore (source range, intermediate range, and power range) detectors. | xiv) A report exists and concludes that the aluminum surfaces of the excore detectors are encased in a watertight stainless steel or titanium housing. |
| 200 | 2.2.03.08d | 8.d) The PXS provides pH adjustment of water flooding the containment following design basis accidents. | Inspections of the pH adjustment baskets will be conducted. | pH adjustment baskets exist, with a total calculated volume $\geq 560 \text{ ft}^3$. The pH baskets are located below plant elevation 107 ft, 2 in. |
| 201 | 2.2.03.09a.i | 9.a) The PXS provides a function to cool the outside of the reactor vessel during a severe accident. | i) A flow test and analysis for each IRWST drain line to the containment will be conducted. The test is initiated by opening isolation valves in each line. Test fixtures may be used to simulate squib valves. | i) The calculated flow resistance for each IRWST drain line between the IRWST and the containment is $\leq 4.07 \times 10^{-6} \text{ ft/gpm}^2$. |