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James Nadeau Manager, Regulatory Assurance Grand Gulf Nuclear Station

Tel. (601) 437-2103

GNRO-2016/00056

November 7, 2016

U.S. Nuclear Regulatory Commission Attn: Document Control Desk Washington, DC 20555-0001

SUBJECT:

Licensee Event Report (LER) 2016-007-00, Technical Specification Shutdown

Due to Loss of Residual Heat Removal Pump

Grand Gulf Nuclear Station, Unit 1

Docket No. 50-416 License No. NPF-29

Dear Sir or Madam:

Attached is Licensee Event Report (LER) 2016-007-00, which will be supplemented upon completion of the in-progress apparent cause evaluation. This report is submitted in accordance with Title 10 Code of Federal Regulations 50.73(a)(2)(i)(A) for the for the completion of any nuclear plant shutdown required by the plant's Technical Specifications.

This letter contains no new Regulatory Commitments.

Ham Modesus

If you have any questions or require additional information, please contact James Nadeau at 601-437-2103.

Sincerely,

JJN/sas

Attachment: Licensee Event Report (LER) 2016-007-00

cc: (See Next Page)

cc: U.S. Nuclear Regulatory Commission ATTN: Mr. Jim Kim, NRR/DORL (w/2)

Mail Stop OWFN 8 B1 Rockville, MD 20852-2738

U.S. Nuclear Regulatory Commission ATTN: Mr. Kriss M. Kennedy (w/2) Regional Administrator, Region IV 1600 East Lamar Boulevard Arlington, TX 76011-4511

Mr. B. J. Smith (w/2) Director, Division of Radiological Health Mississippi State Department of Health Division of Radiological Health 3150 Lawson Street Jackson, MS 39213

NRC Senior Resident Inspector Grand Gulf Nuclear Station Port Gibson, MS 39150

Attachment to GNRO 2016/00056 Licensee Event Report (LER) 2016-007-00

NRC FORM 366

(11-2015)

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB: NO. 3150-0104

EXPIRES: 10/31/2018



LICENSEE EVENT REPORT (LER)

Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA/Privacy Section (T-5 F53), U.S. Nuclear Regulatory

PARAIS ONLINE	Sound of the second of the sec		(See reverse for required number of digits/characters for each block)					Commission, Washington, DC 20555-0001, or by internet e-mail to infocollects.resource@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.								
1. FACILITY NAME							2. DOCKET NUMBER			3. PAGE	3. PAGE					
Grand Gulf Nuclear Station, Unit 1							05000 416				10	F 3				
4. TITLE																
Technical Specification Shutdown due to Loss of Residual Heat Removal Pump																
5. E\	/ENT DA	TE	6. LER NUMBER				7. REPORT DATE			8. OTHER F			ACILITIES INVOLVED			
MONTH	DAY	YEAR	YEAR	SEQUE NUMI		REV NO.	MONTH	DAY	YEAR	FACILITY N N/A	AME		,	DOCKET NUMBER 05000 N/A		
9	8	2016		2016-00	07-00		11	7	2016	FACILITY N N/A	AME			ET NUMBER O N/A		
9. OPERATING MODE 11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check all that apply)								apply)								
1																
			20.2201(d)				20.2203(a)(3)(ii)			50.73(a)(2)(ii)(A)		50.73(a)(2)(viii)(A)				
			20.2203(a)(1)			20.2203(a)(4)			50.73(a)(2)(ii)(B)		50.73(a)(2)(viii)(B)					
			20.2203(a)(2)(i)			50.36(c)(1)(i)(A)			50.73(a)(2)(iii)		50.73(a)(2)(ix)(A)					
10. POWER LEVEL			20.2203(a)(2)(ii)			50.36(c)(1)(ii)(A)			50.73(a)(2)(iv)(A)		50.73(a)(2)(x)					
100%		٠	20.2203(a)(2)(iii)				50.36(c)(2)			50.73(a)(2)(v)(A)		73.71(a)(4)				
			20.2203(a)(2)(iv)				50.46(a)(3)(ii)			50.73(a)(2)(v)(B)			73.71(a)(5)			
						50.73(a)(2)(i)(A)			50.73(a)(2)(v)(C)		OTHER .					
			20.2203(a)(2)(vi)				50.73(a)(2)(i)(B)			50.73(a)(2)(v)(D)		Specify in Abstract below				
								or in			NRC Form 366A					
						12. LIC	ENSEE CON	ITACT F	FOR THIS	LER						
FACILITY NAME TELEPHONE NUMBER (Include Area Code) (201) 107 0100																
James Nadeau / Manager, Regulatory Assurance (601) 437-2103 13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT																
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CAUSE	S	YSTEM	СОМРО	NENT	FACTUE		TO EPIX		AUSE	SYSTEM	COMPONEN	T FACTI		TO EPIX		
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14. SUPPLEMENTAL REPORT EXPECTED								15. EXPECTED			MONTH	DAY	YEAR			
☐ YES (If yes, complete 15. EXPECTED SUBMISSION DATE) ☐ NO							SUBMISSION DATE		1	5	2017					
ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)																
On Sentember 4, 2016 at 02:58. Grand Gulf Nuclear Station entered three TS LCO Action Statements because RHR 'A'																

pump was declared inoperable.

LCO Actions entered:

- 1) 3.5.1 for one low pressure ECCS injection/spray subsystem,
- 2) 3.6.1.7 for one RHR containment spray subsystem, and
- 3) 3.6.2.3 for one RHR suppression pool cooling subsystem. All have 7 day Completion Times

A decision was made to shutdown the plant to repair the RHR 'A' pump because, based on the troubleshooting and testing plan, the pump could not be repaired and returned to service within the LCO Completion Times. At 0300 CDT on 09/08/16, GGNS initiated the transition to Mode 4.

The pump was removed from service and sent to the vendor facility for decontamination, disassembly and failure analysis. The 'A' pump was then replaced and tested satisfactorily. RHR 'A' was returned to operable.

NRC FORM (11-2015)

366A U.S. NUCLEAR REGULATORY COMMISSION

EXPIRES: 10/31/2018



LICENSEE EVENT REPORT (LER) CONTINUATION SHEET

APPROVED BY OMB: NO. 3150-0104 Estimated burden per response to comply with this mandatory collection request: 80 hours Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA, Privacy and Information Collections Branch (T-5 F53), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to Infocollects.Resource@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information

Page 2 of 3 collection.

1. FACILITY NAME	2. DOCKET	3. LER NUMBER			
Grand Gulf Nuclear Station, Unit 1	05000 416	YEAR	SEQUENTIAL NUMBER	REV. NO.	
Grand dan Macical Station, Stiff 1			2016-007-00	· ·	

NARRATIVE

PLANT CONDITIONS PRIOR TO THE EVENT

At the time of the event, Grand Gulf Nuclear Station (GGNS) Unit 1 was in Mode 1, at 100% rated thermal power. All systems, structures and components, with the exception of the RHR 'A' pump, that were necessary to mitigate, reduce the consequences of, or limit the safety implications of the event were available. No other safety significant components were out of service.

DESCRIPTION

On September 4, 2016, GGNS was performing a Residual Heat Removal (RHR) 'A' quarterly Technical Specification (TS) Surveillance Requirement (SR). At 02:58, The RHR pump failed to meet its TS SR Acceptance Criteria for flow and differential pressure (d/p) and was therefore declared Inoperable. Action Statements for TS Limiting Conditions for Operation (LCOs) 3.5.1, 3.6.1.7 and 3.6.2.3 were entered, each having Completion Times of 7 days.

LCO Action Statements entered:

- 1) 3.5.1 for one low pressure ECCS injection/spray subsystem,
- 2) 3.6.1.7 for one RHR containment spray subsystem, and
- 3) 3.6.2.3 for one RHR suppression pool cooling subsystem.

Initial troubleshooting verified that the pump was incapable of meeting the flow requirement of 7756 gpm and d/p of 131 psid simultaneously. The observed pump flow and discharge pressures were verified to be correct via a temporarily installed ultrasonic flow meter and pressure gauge. RHR system valves and lines were verified not to be clogged or leaking. The pump motor was confirmed to be operating at the proper speed.

Further troubleshooting and testing lead station management to the conclusion that RHR 'A' would not be returned to operable status within the 7 day Completion Time. A decision was made to commence an orderly shutdown. On September 8, 2016 at 0300, GGNS began the transition to Mode 4. No other systems were out of service that would have complicated an orderly shutdown to Mode 4.

REPORTABILITY

Event Notification No. 52225 was made to the U.S. Nuclear Regulatory Commission (NRC) Operations Center. This LER is being submitted pursuant to Title 10 Code of Federal Regulations 10 CFR 50.73(a)(2)(i)(A) for the completion of any nuclear plant shutdown required by the plant's Technical Specifications. Telephonic notification was made to the NRC Emergency Notification System on September 8, 2016, at 03:27, pursuant to 10 CFR 50.72(b)(2)(i) for the initiation of any nuclear plant shutdown required by the plant's Technical Specifications.

CAUSE

Direct Cause: The RHR 'A' pump was unable to provide its required flow at the required differential pressure in order to perform its safety function.

NRC FORM (11-2015)

366A U.S. NUCLEAR REGULATORY COMMISSION



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Grand Gan Madical Station, Sint 1		2016-007-00			

NARRATIVE

Apparent Cause: Subsequent investigation suggests internal pump degradation but the Apparent Cause is ongoing. A supplemental report to this LER will be provided when the Apparent Cause investigation is complete.

EXTENT OF CONDITION

Quarterly surveillance data of similar Emergency Core Cooling System (ECCS) pumps showed no evidence of degradation. Data was re-examined from the following pumps: Low Pressure Core Spray (LPCS), High Pressure Core Spray (HPCS) and RHR 'B' and 'C.' GGNS also performed a partial quarterly surveillance on the RHR 'B' which was completed satisfactorily.

CORRECTIVE ACTIONS

The RHR 'A' pump was replaced and retested satisfactorily. The pump removed from service has been sent to the vendor facility for failure analysis.

SAFETY SIGNIFICANCE

The event posed no threat to the health and safety of the general public or to nuclear safety as safetysystems performed as designed. No Technical Specification safety limits were violated. Industrial safety was not challenged, and there was no potential or actual radiological release during the event.

PREVIOUS SIMILAR EVENTS

Previous similar events will be discussed in the supplemental report upon completion of the Apparent Cause investigation.