

M. J. Yox  
Regulatory Affairs Director  
Vogtle 3&4  
Nuclear Development

Southern Nuclear  
Operating Company, Inc.  
7825 River Road  
Waynesboro, GA 30830  
  
Tel 706.848.6459



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ND-16-2175  
10 CFR 52.99(c)(3)

U.S. Nuclear Regulatory Commission  
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Southern Nuclear Operating Company  
Vogtle Electric Generating Plant Unit 3  
Notice of Uncompleted ITAAC 225-days Prior to Initial Fuel Load  
Item 2.3.07.05.i [Index Number 396]

Ladies and Gentlemen:

Pursuant to 10 CFR 52.99(c)(3), Southern Nuclear Operating Company hereby notifies the NRC that as of October 14, 2016, Vogtle Electric Generating Plant (VEGP) Unit 3 Uncompleted Inspection, Test, Analysis, and Acceptance Criteria (ITAAC) Item 2.3.07.05.i [Index Number 396] has not been completed greater than 225-days prior to initial fuel load. The Enclosure describes the plan for completing ITAAC 2.3.07.05.i [Index Number 396]. Southern Nuclear Operating Company will at a later date provide additional notifications for ITAAC that have not been completed 225-days prior to initial fuel load.

This notification is informed by the guidance described in NEI-08-01, *Industry Guideline for the ITAAC Closure Process Under 10 CFR Part 52*, which was endorsed by the NRC in Regulatory Guide 1.215. In accordance with NEI 08-01, this notification includes ITAAC for which required inspections, tests, or analyses have not been performed or have been only partially completed. All ITAAC will be fully completed and all Section 52.99(c)(1) ITAAC Closure Notifications will be submitted to NRC to support the Commission finding that all acceptance criteria are met prior to plant operation, as required by 10 CFR 52.103(g).

This letter contains no new NRC regulatory commitments.

If there are any questions, please contact David Woods at 706-848-6903.

Respectfully submitted,

  
Michael J. Yox  
Regulatory Affairs Director Vogtle 3&4

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Enclosure: Vogtle Electric Generating Plant (VEGP) Unit 3  
Completion Plan for Uncompleted ITAAC 2.3.07.05.i [Index Number 396]

MJY/kms/amm

**To:**

**Southern Nuclear Operating Company/Georgia Power Company**

Mr. S. E. Kuczynski (w/o enclosures)  
Mr. D. A. Bost (w/o enclosures)  
Mr. M. D. Meier  
Mr. M. D. Rauckhorst (w/o enclosures)  
Mr. D. H. Jones (w/o enclosures)  
Ms. K. D. Fili  
Mr. D. L. McKinney  
Mr. B. H. Whitley  
Mr. D. L. Fulton  
Mr. C. E. Morrow  
Mr. M. J. Yox  
Mr. D. Woods  
Ms. A. L. Pugh  
Ms. K. M. Stacy  
Mr. A. S. Parton  
Mr. W. A. Sparkman  
Mr. J. P. Redd  
Mr. D. R. Culver  
Mr. F. H. Willis  
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**cc:**

**Nuclear Regulatory Commission**

Ms. C. Haney (w/o enclosures)  
Ms. A. Bradford (w/o enclosures)  
Ms. J. L. Dixon-Herrity (w/o enclosures)  
Ms. J. M. Heisserer  
Mr. C. J. Even  
Mr. C. P. Patel  
Mr. B. M. Baval  
Ms. R. C. Reyes  
Ms. M. A. Sutton  
Mr. M. E. Ernstes  
Mr. G. J. Khouri  
Mr. J. D. Fuller  
Mr. T. E. Chandler  
Ms. S. E. Temple  
Ms. P. Braxton  
Mr. T. C. Brimfield  
Mr. A. J. Lerch

**Oglethorpe Power Corporation**

Mr. M. W. Price  
Ms. K. T. Haynes  
Ms. A. Whaley

**Municipal Electric Authority of Georgia**

Mr. J. E. Fuller

Mr. S. M. Jackson

**Dalton Utilities**

Mr. D. Cope

**WECTEC**

Mr. C. A. Castell

**Westinghouse Electric Company, LLC**

Mr. R. Easterling (w/o enclosures)

Mr. J. W. Crenshaw (w/o enclosures)

Mr. L. Woodcock (w/o enclosures)

Mr. C. F. Landon

Mr. A. F. Dohse

Mr. M. Y. Shaqqo

Ms. S. DiTommaso

**Other**

Mr. J. E. Hesler, *Bechtel Power Corporation*

Ms. L. Matis, *Tetra Tech NUS, Inc.*

Dr. W. R. Jacobs, Jr., *Ph.D., GDS Associates, Inc.*

Mr. S. Roetger, *Georgia Public Service Commission*

Ms. S. W. Kernizan, *Georgia Public Service Commission*

Mr. K. C. Greene, *Troutman Sanders*

Mr. S. Blanton, *Balch Bingham*

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**Southern Nuclear Operating Company  
ND-16-2175  
Enclosure**

**Vogtle Electric Generating Plant (VEGP) Unit 3  
Completion Plan for Uncompleted ITAAC 2.3.07.05.i [Index Number 396]**

**Subject: Uncompleted ITAAC 2.3.07.05.i [Index No. 396]**

## **ITAAC Statement**

### **Design Commitment**

5. *The seismic Category I components identified in Table 2.3.7-1 can withstand seismic design basis loads without loss of safety functions.*

### **Inspections/Tests/Analyses**

- i) *Inspection will be performed to verify that the seismic Category I components identified in Table 2.3.7-1 are located on the Nuclear Island.*

### **Acceptance Criteria**

- i) *The seismic Category I components identified in Table 2.3.7-1 are located on the Nuclear Island.*

## **ITAAC Completion Description**

Multiple ITAAC are performed to demonstrate that the seismic Category I components identified in VEGP Unit 3 Combined License (COL) Appendix C Table 2.3.7-1 (Attachment A) can withstand seismic design basis loads without loss of safety functions. The subject ITAAC requires an inspection to verify that the seismic Category I components identified in Attachment A are located on the Nuclear Island, which is a Seismic Category I structure.

To assure that seismic Category I components can withstand seismic design basis loads without loss of safety functions, all of the components in VEGP Unit 3 COL Appendix C Table 2.3.7-1 are designed to be located on the seismic Category I Nuclear Island. In accordance with Inspection Procedure XYZ (Reference 1), an inspection is conducted on the Spent Fuel Pool Cooling System (SFS) to confirm the satisfactory installation of the seismically qualified components. The inspection includes verification of the equipment make/model/serial number and verification of the as-built equipment location (Building, Elevation, Room). The inspection of the component locations is documented in Inspection Reports (Reference 2).

Attachment A identifies the Inspection Reports which verify that the installed locations of the Seismic Category I components identified in VEGP Unit 3 COL Appendix C Table 2.3.7-1 are located on the Nuclear Island. The Inspection Reports are available for NRC inspection as part of the ITAAC Completion Package (Reference 3).

## **List of ITAAC Findings**

In accordance with plant procedures for ITAAC completion, Southern Nuclear Operating Company (SNC) performed a review of all findings pertaining to the subject ITAAC and

associated corrective actions. This review found there are no relevant ITAAC findings associated with this ITAAC.

**References (available for NRC inspection)**

1. Inspection Procedure XYZ
2. Inspection Reports as identified in Attachment A
3. ITAAC 2.3.07.05.i Completion Package
4. NEI 08-01, "Industry Guideline for the ITAAC Closure Process Under 10 CFR Part 52"

**Attachment A: Excerpt from COL Appendix C Table 2.3.7-1**

**ITAAC COMPLIANCE MATRIX FOR SEISMIC CATEGORY I COMPONENTS  
(SPENT FUEL POOL COOLING SYSTEM)**

<b>Equipment Name</b>	<b>Tag No.</b>	<b>Seismic Category I</b>	<b>Inspection Report</b>
Spent Fuel Pool Level Sensor	SFS-019A	Yes	XXX
Spent Fuel Pool Level Sensor	SFS-019B	Yes	XXX
Spent Fuel Pool Level Sensor	SFS-019C	Yes	XXX
Refueling Cavity Drain to SGS Compartment Isolation Valve	SFS-PL-V031	Yes	XXX
Refueling Cavity to SFS Pump Suction Isolation Valve	SFS-PL-V032	Yes	XXX
Refueling Cavity Drain to Containment Sump Isolation Valve	SFS-PL-V033	Yes	XXX
IRWST to SFS Pump Suction Line Isolation Valve	SFS-PL-V039	Yes	XXX
Fuel Transfer Canal to SFS Pump Suction Iso. Valve	SFS-PL-V040	Yes	XXX
Cask Loading Pit to SFS Pump Suction Isolation Valve	SFS-PL-V041	Yes	XXX
Cask Loading Pit to SFS Pump Suction Isolation Valve	SFS-PL-V042	Yes	XXX
SFS Pump Discharge Line to Cask Loading Pit Isolation Valve	SFS-PL-V045	Yes	XXX
Cask Loading Pit to WLS Isolation Valve	SFS-PL-V049	Yes	XXX
Spent Fuel Pool to Cask Washdown Pit Isolation Valve	SFS-PL-V066	Yes	XXX
Cask Washdown Pit Drain Isolation Valve	SFS-PL-V068	Yes	XXX
Refueling Cavity Drain Line Check Valve	SFS-PL-V071	Yes	XXX
Refueling Cavity Drain Line Check Valve	SFS-PL-V072	Yes	XXX
SFS Containment Floodup Isolation Valve	SFS-PL-V075	Yes	XXX