

M. J. Yox  
Regulatory Affairs Director  
Vogtle 3&4  
Nuclear Development

Southern Nuclear  
Operating Company, Inc.  
7825 River Road  
Waynesboro, GA 30830

Tel 706.848.6459



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U.S. Nuclear Regulatory Commission  
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Southern Nuclear Operating Company  
Vogtle Electric Generating Plant Unit 3  
Notice of Uncompleted ITAAC 225-days Prior to Initial Fuel Load  
Item 2.3.06.02b [Index Number 356]

Ladies and Gentlemen:

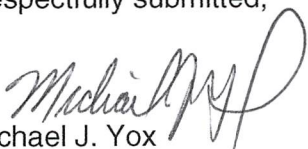
Pursuant to 10 CFR 52.99(c)(3), Southern Nuclear Operating Company hereby notifies the NRC that as of October 14, 2016, Vogtle Electric Generating Plant (VEGP) Unit 3 Uncompleted Inspection, Test, Analysis, and Acceptance Criteria (ITAAC) Item 2.3.06.02b [Index Number 356] has not been completed greater than 225-days prior to initial fuel load. The Enclosure describes the plan for completing ITAAC 2.3.06.02b [Index Number 356]. Southern Nuclear Operating Company will at a later date provide additional notifications for ITAAC that have not been completed 225-days prior to initial fuel load.

This notification is informed by the guidance described in NEI-08-01, *Industry Guideline for the ITAAC Closure Process Under 10 CFR Part 52*, which was endorsed by the NRC in Regulatory Guide 1.215. In accordance with NEI 08-01, this notification includes ITAAC for which required inspections, tests, or analyses have not been performed or have been only partially completed. All ITAAC will be fully completed and all Section 52.99(c)(1) ITAAC Closure Notifications will be submitted to NRC to support the Commission finding that all acceptance criteria are met prior to plant operation, as required by 10 CFR 52.103(g).

This letter contains no new NRC regulatory commitments.

If there are any questions, please contact David Woods at 706-848-6903.

Respectfully submitted,

  
Michael J. Yox  
Regulatory Affairs Director Vogtle 3&4

U.S. Nuclear Regulatory Commission

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Enclosure: Vogtle Electric Generating Plant (VEGP) Unit 3  
Completion Plan for Uncompleted ITAAC 2.3.06.02b [Index Number 356]

MJY/kms/amm

**To:**

**Southern Nuclear Operating Company/Georgia Power Company**

Mr. S. E. Kuczynski (w/o enclosures)

Mr. D. A. Bost (w/o enclosures)

Mr. M. D. Meier

Mr. M. D. Rauckhorst (w/o enclosures)

Mr. D. H. Jones (w/o enclosures)

Ms. K. D. Fili

Mr. D. L. McKinney

Mr. B. H. Whitley

Mr. D. L. Fulton

Mr. C. E. Morrow

Mr. M. J. Yox

Mr. D. Woods

Ms. A. L. Pugh

Ms. K. M. Stacy

Mr. A. S. Parton

Mr. W. A. Sparkman

Mr. J. P. Redd

Mr. D. R. Culver

Mr. F. H. Willis

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cc:

**Nuclear Regulatory Commission**

Ms. C. Haney (w/o enclosures)

Ms. A. Bradford (w/o enclosures)

Ms. J. L. Dixon-Herrity (w/o enclosures)

Ms. J. M. Heisserer

Mr. C. J. Even

Mr. C. P. Patel

Mr. B. M. Bovol

Ms. R. C. Reyes

Ms. M. A. Sutton

Mr. M. E. Ernestes

Mr. G. J. Khouri

Mr. J. D. Fuller

Mr. T. E. Chandler

Ms. S. E. Temple

Ms. P. Braxton

Mr. T. C. Brimfield

Mr. A. J. Lerch

**Oglethorpe Power Corporation**

Mr. M. W. Price

Ms. K. T. Haynes

Ms. A. Whaley

**Municipal Electric Authority of Georgia**

Mr. J. E. Fuller  
Mr. S. M. Jackson

**Dalton Utilities**

Mr. D. Cope

**WECTEC**

Mr. C. A. Castell

**Westinghouse Electric Company, LLC**

Mr. R. Easterling (w/o enclosures)  
Mr. J. W. Crenshaw (w/o enclosures)  
Mr. L. Woodcock (w/o enclosures)  
Mr. C. F. Landon  
Mr. A. F. Dohse  
Mr. M. Y. Shaqqo  
Ms. S. DiTommaso

**Other**

Mr. J. E. Hesler, *Bechtel Power Corporation*  
Ms. L. Matis, *Tetra Tech NUS, Inc.*  
Dr. W. R. Jacobs, Jr., *Ph.D., GDS Associates, Inc.*  
Mr. S. Roetger, *Georgia Public Service Commission*  
Ms. S. W. Kernizan, *Georgia Public Service Commission*  
Mr. K. C. Greene, *Troutman Sanders*  
Mr. S. Blanton, *Balch Bingham*

**Southern Nuclear Operating Company  
ND-16-2154  
Enclosure**

**Vogtle Electric Generating Plant (VEGP) Unit 3  
Completion Plan for Uncompleted ITAAC 2.3.06.02b [Index Number 356]**

**Subject: Uncompleted ITAAC 2.3.06.02b [Index No. 356]**

## **ITAAC Statement**

### **Design Commitment**

*2.b) The piping identified in Table 2.3.6-2 as ASME Code Section III is designed and constructed in accordance with ASME Code Section III requirements.*

### **Inspections/Tests/Analyses**

*Inspection will be conducted of the as-built piping as documented in the ASME design reports.*

### **Acceptance Criteria**

*The ASME Code Section III design reports exist for the as-built piping identified in Table 2.3.6-2 as ASME Code Section III.*

## **ITAAC Completion Description**

An inspection is performed in accordance with the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code (BPVC) Section III (Reference 1) to demonstrate that the as-built piping identified in VEGP Unit 3 Combined License (COL) Appendix C Table 2.3.6-2 as ASME Code Section III (Attachment A) is designed and constructed in accordance with ASME Code Section III requirements. This ITAAC is complete once the as-built ASME Code Section III design reports for the piping in Attachment A are completed, the piping is stamped with a Code Symbol N-Stamp, and the Authorized Nuclear Inspector (ANI) has signed the N-5 Code Data Report(s) (Reference 2) listed in Attachment A indicating the as-built piping is installed in accordance with ASME Code Section III requirements.

The as-built piping listed in Attachment A is subjected to a design report reconciliation process which verifies that the as-built piping complies with all design specification and Code provisions. Design reconciliation of the as-built piping validates that construction completion, including field changes and any nonconforming condition dispositions, is consistent with and bounded by the approved design. All applicable fabrication, installation and testing records, as well as those for the related Quality Assurance verification and inspection activities, which confirm adequate construction in compliance with ASME Code Section III and the design provisions, are referenced in the N-5 Code Data Report(s) and their sub-tier references.

The completed ASME Code Section III design reports for the as-built piping identified in Attachment A exist and document that the piping conforms to approved design details. The design reports support completion of the N-5 Code Data Report(s) signed by the Authorized Nuclear Inspector.

The completion of the piping Code Symbol N-Stamp stamping, and the N-5 Code Data Report(s) which incorporate the design reports for the piping listed in Attachment A, confirm that the as-built piping is designed and constructed in accordance with ASME Code Section III requirements. The N-5 Code Data Report(s) and associated ASME Code Section III design reports are available for NRC inspection as part of the ITAAC Completion Package (Reference 3).

### **List of ITAAC Findings**

In accordance with plant procedures for ITAAC completion, Southern Nuclear Operating Company (SNC) performed a review of all findings pertaining to the subject ITAAC and associated corrective actions. This review found there are no relevant ITAAC findings associated with this ITAAC.

### **References (available for NRC inspection)**

1. American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code (BPVC) Section III requirements as described in VEGP 3&4 Updated Final Safety Analysis Report, Section 5.2.1, Compliance with Codes and Code Cases
2. ASME Section III N-5 Code Data Report(s) XXX identified in Attachment A
3. ITAAC 2.3.06.02b Completion Package
4. NEI 08-01, "Industry Guideline for the ITAAC Closure Process Under 10 CFR Part 52"

**Attachment A: Excerpt from COL Appendix C Table 2.3.6-2**

Line Name	Line Number	ASME Code Section III	N-5 Code Data Report
RNS Suction Lines, from the RCS Hot Leg Connection to the RCS Side of Valves RNS PL-V001A [sic] and RNS-PL-V001B	RNS-L001 RNS-L002A RNS-L002B	Yes	XXX
RNS Suction Lines, from the RCS Pressure Boundary Valves, RNS-PL-V001A and RNS-PL-V001B, to the RNS pumps	RNS-L004A RNS-L004B RNS-L005 RNS-L006 RNS-L007A RNS-L007B RNS-L009A RNS-L009B	Yes	XXX
RNS Suction Line from CVS	RNS-L061	Yes	XXX
RNS Suction Line from IRWST	RNS-L029	Yes	XXX
RNS Suction Line LTOP Relief	RNS-L040	Yes	XXX
RNS Discharge Lines, from the RNS Pumps to the RNS Heat Exchangers RNS-ME-01A and RNS-ME-01B	RNS-L011A RNS-L011B	Yes	XXX
RNS Discharge Lines, from RNS Heat Exchanger RNS-ME-01A to Containment Isolation Valve RNS-PL-V011	RNS-L012A RNS-L014	Yes	XXX
RNS Discharge Line, from RNS Heat Exchanger RNS-ME-01B to Common Discharge Header RNS-L014	RNS-L012B	Yes	XXX
RNS Discharge Lines, Containment Isolation Valve RNS-PL-V011 to Containment Isolation Valve RNS-PL-V013	RNS-L016	Yes	XXX
RNS Suction Line from Cask Loading Pit	RNS-L065	Yes	XXX
RNS Discharge Lines, from Containment Isolation Valve RNS-PL-V013 to RCS Pressure Boundary Isolation Valves RNS-PL-V015A and RNS-PL-V015B	RNS-L017 RNS-L018A RNS-L018B	Yes	XXX



Line Name	Line Number	ASME Code Section III	N-5 Code Data Report
RNS Discharge Lines, from Direct Vessel Injection (DVI) Line RNS-BBC-L018A to Passive Core Cooling System (PXS) IRWST Return Isolation Valve RNS-PL-V024	RNS-L020	Yes	XXX
RNS Discharge Lines, from RCS Pressure Boundary Isolation Valves RNS-PL-V015A and RNS-PL-V015B to Reactor Vessel DVI Nozzles	RNS-L019A RNS-L019B	Yes	XXX
	PXS-L019A PXS-L019B	Yes	XXX
RNS Heat Exchanger Bypass	RNS-L008A RNS-L008B	Yes	XXX
RNS Suction from Spent Fuel Pool	RNS-L052	Yes	XXX
RNS Pump Miniflow Return	RNS-L030A RNS-L030B	Yes	XXX
RNS Discharge to Spent Fuel Pool	RNS-L051	Yes	XXX
RNS Discharge to CVS Purification	RNS-L021	Yes	XXX