

PSEG Nuclear LLC
P.O. Box 236, Hancocks Bridge, NJ 08038-0236



ASME Code Case N-532-5

LR-N16-0187
October 11, 2016

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington DC 20555-0001

Salem Generating Station Unit 1
Renewed Facility Operating License DPR-70
NRC Docket No. 50-272

Subject: In-Service Inspection Activities – 90-Day Report

This letter submits the Owner's Activity Report for In-Service Inspection (ISI) activities conducted at the Salem Generating Station Unit 1 during the twenty-fourth refueling outage. This report is submitted in accordance with Code Case N-532-5 "Repair/Replacement Activity Documentation Requirements and In-Service Summary Report Preparation and Submission Section XI, Division 1," and within its ninety day reporting requirement.

Enclosure 1 to this letter contains the required Salem Unit 1 Owner's Activity Report.

There are no commitments contained in this letter.

Should you have any questions regarding this submittal, please contact Mr. Michael Phillips at 856-339-1873.

Sincerely,

A handwritten signature in black ink, appearing to read "F. Kenneth Grover", with the name "F. Kenneth Grover" printed in a smaller font below it.

F. Kenneth Grover
Plant Manager
Salem Generating Station

Enclosure

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cc: Mr. D. Dorman, USNRC -Administrator-Region 1
Ms. C. Sanders, USNRC -Licensing Project Manager-Salem
Mr. P. Finney, USNRC Senior Resident Inspector
Mr. P. Mulligan, NJBNE Manager IV
Mr. T. Cachaza, Salem Commitment Tracking Coordinator
Mr. L. Marabella, Corporate Commitment Tracking Coordinator
Mr. M. Washington, Chief Inspector – Occupational Safety and Health
Bureau of Boiler and Pressure Vessel Compliance

Enclosure 1
LR-N16-0187

**Salem Unit 1
Owner's Activity Report – S1RFO24
In Accordance With ASME
Section XI Code Case N-532-5**

FORM OAR-1 OWNER'S ACTIVITY REPORT

Report Number S1RFO24

Plant Salem Generating Station

Unit No. 1 Commercial service date July 1, 1977 Refueling outage no. 24
(If applicable)

Current inspection interval ISI - Fourth (4th), CISI - Second (2nd)
(1st, 2nd, 3rd, 4th, other)

Current inspection period ISI - Second (2nd), CISI - Second (2nd)
(1st, 2nd, 3rd)

Edition and Addenda of Section XI applicable to the inspection plans 2004 Edition, No Addenda

Date and revision of inspection plans ISI Revision 2, July 2015 (CISI Revision 1, July 2015)

Edition and Addenda of Section XI applicable to repair/replacement activities, if different than the inspection plans Same as ISI Plans

Code Cases used for inspection and evaluation: N-460, N-532-5 & N-706-1
(If applicable, including cases modified by Case N-532 and later revisions)

CERTIFICATE OF CONFORMANCE

I certify that (a) the statements made in this report are correct; (b) the examinations and tests meet the Inspection Plan as required by the ASME Code, Section XI; and (c) the repair/replacement activities and evaluations supporting the completion of S1RFO24
(refueling outage number)

Conform to the requirements of Section XI.

Signed Edley T. Giles Edley T Giles / ISI Program Owner Date 9/30/2016
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and employed by Hartford Steam Boiler Inspection and Insurance Company of Connecticut have inspected the items described in this Owner's Activity Report, and state that, to the best of my knowledge and belief, the Owner has performed all activities represented by this report in accordance with the requirements of Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair/replacement activities and evaluation described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Alphonso Post Commission NB 15462, A, N, I, NJ 1219
Inspector's Signature (National Board Number and Endorsement)

Date 9/30/16

FORM OAR-1 OWNER'S ACTIVITY REPORT

TABLE 1
ITEMS WITH FLAWS OR RELEVANT CONDITIONS THAT REQUIRED
EVALUATION FOR CONTINUED SERVICE

Exam Category	Item Number	Item Description	Evaluation Description
CC	L1.10	Containment Exterior Concrete Surface, Identified small pop out in concrete surface exposing reinforcement bar	ASME Section XI Subsection IWL Engineering Evaluation of exposed reinforcement and loss of concrete cover, found acceptable for continued service and repair scheduled. Evaluation documented in order 70185803.

FORM OAR-1 OWNER'S ACTIVITY REPORT

TABLE 2
ABSTRACT OF REPAIR/REPLACEMENT ACTIVITIES REQUIRED FOR CONTINUED SERVICE

Code Class	Item Description	Description of Work	Date Complete	Repair/Replacement Plan Number
3	Leak downstream of 3/4" Instrument Isolation valve 12CC197, the leak was located on 3/4" carbon steel valve to stainless steel reducing Insert fillet weld.	Leaking weld and associated fitting replaced.	9/01/2015	60114705
3	Leak upstream of 12 Chilled Water Pump Discharge 3/4" carbon steel pressure indication root valve 12CH12.	Leaking pipe and 12CH12 Valve replaced.	2/17/2015	60120640
3	Unit 1 Aux Feed Pump Room Cooler Service Water valve 1SW137 was determined to have a through wall leak in the 2" stainless steel valve body.	Replaced flanged valve 1SW137 and bolting.	11/25/2015	60126260
3	Service water leak was identified on 3/8" stainless tubing weld downstream side of 13SW41 pressure tap isolation valve.	Tubing and insert was removed from flange and replaced with new Swagelok fitting.	3/15/2016	60128003
2	Spring Can Hanger Support number RHRH 11-21 was found out of Adjustment.	Support RHRH 11-21 evaluated to be acceptable and adjusted to design specification.	7/29/2016	60129130
MC	Surface corrosion identified on containment liner plate under insulation system during License Renewal Inspections. UT of liner plate found low thickness measurement of 0.281", tnom of 0.375" with a tmin of 0.338".	Weld overlay performed on approximately 10 square feet of liner plate, performed extent of condition additional inspections, no other areas of degradation identified.	7/19/2016	60120251
1	Reactor Vessel Internals Baffle-Former bolting found degraded during Initial visual inspection.	Performed 100% Visual and Ultrasonic (UT) examinations of all accessible 832 Baffle-Former bolts, replaced 189 bolts with 3 bolts in replacement scope evaluated for 1 cycle of operation.	7/16/2016	60129129
3	Chilled Water leak developed on 3/4" carbon steel drain pipe between pump casing and drain valve 12CH11.	Replaced affected 3/4" piping and drain valve 12CH11.	9/12/2016	60130227