



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**
WASHINGTON, D.C. 20555-0001

October 18, 2016

Mr. Mano Nazar
President and Chief Nuclear Officer
Nuclear Division
Florida Power & Light Co.
Mail Stop NT3/JW
15430 Endeavor Drive
Jupiter, FL 33478

SUBJECT: ST. LUCIE PLANT, UNIT NOS. 1 AND 2 – CORRECTION TO AMENDMENT NOS. 236 AND 186 REGARDING USE OF NEW COMPUTER CODE TO MODEL THE CONTAINMENT VACUUM ANALYSES (CAC NOS. MF6980 AND MF6981)

Dear Mr. Nazar:

On October 5, 2016, the U.S. Nuclear Regulatory Commission (NRC, the Commission) issued Amendment Nos. 236 and 186 to the St. Lucie Plant, Units Nos. 1 and 2, Renewed Facility Operating License Nos. DPR-67 and NPF-16, respectively (Agencywide Documents Access and Management System (ADAMS) Accession No. ML16166A424). The amendments updated the St. Lucie Plant, Units Nos. 1 and 2, Technical Specifications (TSs) to reflect the use of "Generation of Thermal Hydraulic Information for Containments (GOTHIC Version 7.2b(QA))" containment vacuum analyses computer code. The amendments also updated the St. Lucie Plant, Units Nos. 1 and 2, TSs to credit the design-basis ability of the containment vessel to withstand a higher external pressure differential of 1.04 pounds per square inch (psi) for Unit No. 1 and 1.05 psi for Unit No. 2, and updated TS 3.6.1.4 for both units to revise the allowable containment operating pressure range.

Specifically, Unit No. 1 TS page 3/4 6-12 and Unit No. 2 TS page 3/4 6-11 were updated by Amendment Nos. 236 and 186. These two TS pages were recently updated when Amendment Nos. 234 and 184 were issued on August 30, 2016 (ADAMS Accession No. ML16210A374). However, in issuing Amendment Nos. 236 and 186, the NRC staff inadvertently included TS pages that did not reflect changes for Amendment Nos. 234 and 184.

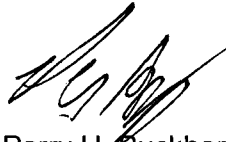
Enclosed are the corrected Unit No. 1 page 3/4 6-12 and Unit No. 2 page 3/4 6-11 to replace the corresponding pages issued by Amendment Nos. 236 and 186 for the St. Lucie Plant. This correction does not change any of the conclusions in the safety evaluation associated with the amendments.

M. Nazar

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If you have any questions regarding this matter, please contact me at (301) 415-1383 or Perry.Buckberg@nc.gov.

Sincerely,

A handwritten signature in black ink, appearing to read 'Perry H. Buckberg', written over a horizontal line.

Perry H. Buckberg, Senior Project Manager
Plant Licensing Branch II-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-335 and 50-389

Enclosures:

1. Corrected Unit No. 1 TS page 3/4 6-12
2. Corrected Unit No. 2 TS page 3/4 6-11

cc w/enclosures: Distribution via Listserv

ENCLOSURE 1

Corrected Unit No. 1 TS page 3/4 6-12

CONTAINMENT SYSTEMS

INTERNAL PRESSURE

LIMITING CONDITION FOR OPERATION

3.6.1.4 Primary containment internal pressure shall be maintained between -0.490 and +0.5 psig.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With the containment internal pressure outside of the limits above, restore the internal pressure to within the limits within 1 hour or be in at least HOT STANDBY within the next 6 hours and in HOT SHUTDOWN within the following 6 hours. LCO 3.0.4.a is not applicable when entering HOT SHUTDOWN.

SURVEILLANCE REQUIREMENTS

4.6.1.4 The primary containment internal pressure shall be determined to be within the limits in accordance with the Surveillance Frequency Control Program.

ENCLOSURE 2

Corrected Unit No. 2 TS page 3/4 6-11

CONTAINMENT SYSTEMS

INTERNAL PRESSURE

LIMITING CONDITION FOR OPERATION

3.6.1.4 Primary containment internal pressure shall be maintained between -0.420 and +0.400 psig.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With the containment internal pressure outside of the limits above, restore the internal pressure to within the limits within 1 hour or be in at least HOT STANDBY within the next 6 hours and in HOT SHUTDOWN within the following 6 hours. LCO 3.0.4.a is not applicable when entering HOT SHUTDOWN.

SURVEILLANCE REQUIREMENTS

4.6.1.4 The primary containment internal pressure shall be determined to be within the limits in accordance with the Surveillance Frequency Control Program.

M. Nazar

- 2 -

If you have any questions regarding this matter, please contact me at (301) 415-1383 or Perry.Buckberg@nc.gov.

Sincerely,

/RA/

Perry H. Buckberg, Senior Project Manager
Plant Licensing Branch II-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

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