



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

October 18, 2016

Vice President, Operations  
Entergy Operations, Inc.  
Grand Gulf Nuclear Station  
P.O. Box 756  
Port Gibson, MS 39150

SUBJECT: GRAND GULF NUCLEAR STATION, UNIT 1 - CORRECTION TO AMENDMENT  
NO. 197 RE: CHANGE TECHNICAL SPECIFICATION SURVEILLANCE  
REQUIREMENT FREQUENCIES FROM 18 MONTHS TO 24 MONTHS  
(CAC NO. ME9764)

Dear Sir or Madam:

By letter dated December 26, 2013 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML13343A109), the U.S. Nuclear Regulatory Commission (NRC) issued Amendment No. 197 to Facility Operating License No. NPF-29 for the Grand Gulf Nuclear Station, Unit 1 (GGNS). This amendment consisted of changes to the Technical Specifications (TSs) in response to Entergy Operations, Inc.'s application dated October 2, 2012 (ADAMS Accession No. ML122770130), as supplemented by letters dated April 26, June 4, August 15, September 24, September 26, October 14, November 12, December 5, and December 11, 2013 (ADAMS Accession Nos. ML13119A102, ML13162A201, ML13232A057, ML13267A218, ML13270A056, ML13288A179, ML13323A548, ML13340A773, ML13346A283, respectively).

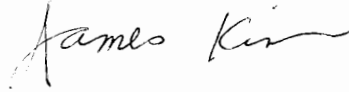
The amendment revised the TSs for GGNS to support operation with 24-month fuel cycles. Specifically, the change revised the frequency of certain TS Surveillance Requirements (SRs) from "18 months" to "24 months," in accordance with the guidance of NRC Generic Letter (GL) 91-04, "Changes in Technical Specification Surveillance Intervals to Accommodate a 24-Month Fuel Cycle,"

Subsequent to issuance of the amendment, typographical errors were discovered on TS pages 3.3-56 and 3.3-72 of the amendment. On TS page 3.3-56, Function 3c., "RCIC Steam Supply Line Pressure - Low," the Allowable Value should be  $\geq 57$  psig instead of  $\geq 53$  psig. The Allowable Value of  $\geq 57$  psig was approved in Amendment No. 194 (ADAMS Accession No. ML13157A029); however, the changed TS page 3.3-56 was inadvertently not replaced in the authority file. This error was carried forward in Amendment No. 197. On TS page 3.3-72, the symbol " $\nabla$ " in Allowable Values for SRs 3.3.6.5.3a and 3.3.6.5.3b should be replaced with the symbol " $\pm$ ." These corrections are editorial in nature and do not affect the previously issued safety evaluation of the amendments.

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Enclosed are the revised TS pages 3.3-56 and 3.3-72. If you have any questions regarding this matter, please contact me at 301-415-4125 or by e-mail at [James.Kim@nrc.gov](mailto:James.Kim@nrc.gov).

Sincerely,

A handwritten signature in cursive script that reads "James Kim".

James Kim, Project Manager  
Plant Licensing IV-2 and Decommissioning  
Transition Branch  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-416

Enclosure:  
Corrected TS pages 3.3-56 and 3.3-72  
for Amendment No. 197

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Primary Containment and Drywell Isolation Instrumentation  
3.3.6.1

Table 3.3.6.1-1 (page 3 of 5)  
Primary Containment and Drywell Isolation Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS PER TRIP SYSTEM	CONDITIONS REFERENCED FROM REQUIRED ACTION C.1	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
3. Reactor Core Isolation Cooling (RCIC) System Isolation					
a. RCIC Steam Line Flow - High	1,2,3	1	F	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.3 SR 3.3.6.1.7 SR 3.3.6.1.8	≤ 64 inches water
b. RCIC Steam Line Flow Time Delay	1,2,3	1	F	SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.8	≥ 3 seconds and ≤ 7 seconds
c. RCIC Steam Supply Line Pressure - Low	1,2 <sup>(d)</sup> ,3 <sup>(d)</sup>	1	F	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.3 SR 3.3.6.1.7 SR 3.3.6.1.8	≥ 57 psig
d. RCIC Turbine Exhaust Diaphragm Pressure - High	1,2,3	2	F	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.3 SR 3.3.6.1.7 SR 3.3.6.1.8	≤ 20 psig
e. RCIC Equipment Room Ambient Temperature - High	1,2,3	1	F	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.5 SR 3.3.6.1.8	≤ 191°F
f. Main Steam Line Tunnel Ambient Temperature - High	1,2,3	1	F	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.5 SR 3.3.6.1.8	≤ 191°F
g. Main Steam Line Tunnel Temperature Timer	1,2,3	1	F	SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.8	≤ 30 minutes
h. RHR Equipment Room Ambient Temperature - High	1,2,3	1 per room	F	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.5 SR 3.3.6.1.8	≤ 171°F
i. RCIC/RHR Steam Line Flow - High	1,2,3	1	F	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.3 SR 3.3.6.1.7 SR 3.3.6.1.8	≤ 43 inches water
(continued)					

<sup>(d)</sup> Not required to be OPERABLE in MODE 2 or 3 with reactor steam dome pressure less than 150 psig during reactor startup.

# SURVEILLANCE REQUIREMENTS

-----NOTE-----  
When a channel is placed in an inoperable status solely for performance of required Surveillances, entry into associated Conditions and Required Actions may be delayed for up to 6 hours, provided the associated Function maintains LLS or relief initiation capability, as applicable.  
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SURVEILLANCE	FREQUENCY
SR 3.3.6.5.1 Perform CHANNEL FUNCTIONAL TEST.	92 days
SR 3.3.6.5.2 Calibrate the trip unit.	92 days
SR 3.3.6.5.3 Perform CHANNEL CALIBRATION. The Allowable Values shall be:  a. Relief Function  Low: 1103 ± 15 psig Medium: 1113 ± 15 psig High: 1123 ± 15 psig  b. LLS Function  Low open: 1033 ± 15 psig close: 926 ± 15 psig Medium open: 1073 ± 15 psig close: 936 ± 15 psig High open: 1113 ± 15 psig close: 946 ± 15 psig	24 months
SR 3.3.6.5.4 Perform LOGIC SYSTEM FUNCTIONAL TEST.	24 months

- 2 -

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Sincerely,

/RA/

James Kim, Project Manager  
Plant Licensing IV-2 and Decommissioning  
Transition Branch  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-416

Enclosure:  
Corrected TS pages 3.3-56 and 3.3-72  
for Amendment No. 197

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