

November 9, 2016

Mr. Kristopher Cummings  
Senior Project Manager, Used Fuel Programs  
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1201 F Street, NW, Suite 1100  
Washington, D.C. 20004

SUBJECT: DRAFT SAFETY EVALUATION FOR NUCLEAR ENERGY INSTITUTE TOPICAL  
REPORT NEI 16-03 - GUIDANCE FOR MONITORING OF FIXED NEUTRON  
ABSORBERS IN SPENT FUEL POOLS (CAC NO. MF8122)

Dear Mr. Cummings:

By letter dated May 10, 2016 (Agencywide Documents Access Management System (ADAMS) Accession No. ML16147A078), as supplemented by letter dated August 30, 2016 (ADAMS Accession No. ML16265A248), the Nuclear Energy Institute (NEI), on behalf of the nuclear industry, submitted NEI 16-03, "Guidance for Monitoring of Fixed Neutron Absorbers in Spent Fuel Pools," Revision 0. The NEI submittal provides guidance for monitoring programs for fixed neutron absorbers in spent fuel pools as a means to demonstrate compliance with the applicable requirements in Title 10 of the *Code of Federal Regulations* Part 50.68, "Criticality Accident Requirements," with respect to the neutron absorbing materials.

Twenty working days are provided for you to comment on any factual errors or clarity concerns contained in the safety evaluation (SE). The final SE will be issued after making any necessary changes. The NRC staff's disposition of your comments on the draft SE will be discussed in the final SE. To facilitate the NRC staff's review of your comments, please provide a marked-up copy of the draft SE showing proposed changes and provide a summary table of the proposed changes.

If you have any questions, please contact Brian Benney at 301-415-2767.

Sincerely,

/RA/

Kevin Hsueh, Chief  
Licensing Processes Branch  
Division of Policy and Rulemaking  
Office of Nuclear Reactor Regulation

Project No. 689

Enclosure:  
Draft SE

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**ADAMS Accession No.: Draft SE ML16280A369;**

**\*concurred via e-mail**

**NRR-1043**

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DATE	11/2/16	10/27/16	11/2/16	11/9/16

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1 DRAFT SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

2  
3 NUCLEAR ENERGY INSTITUTE TOPICAL REPORT 16-03

4  
5 "GUIDANCE FOR MONITORING OF FIXED NEUTRON ABSORBERS

6  
7 IN SPENT FUEL POOLS"

8  
9 PROJECT NO. 689

10  
11 1.0 INTRODUCTION

12  
13 By letter dated May 10, 2016 (Agencywide Documents Access Management System (ADAMS)  
14 Accession No. ML16147A078), as supplemented by letter dated August 30, 2016 (ADAMS  
15 Accession No. ML16265A248), the Nuclear Energy Institute (NEI), on behalf of the nuclear  
16 industry, submitted NEI 16-03, "Guidance for Monitoring of Fixed Neutron Absorbers in Spent  
17 Fuel Pools," Revision 0. The purpose of the document is to provide guidance for licensees to  
18 develop an acceptable fixed neutron absorber monitoring program in spent fuel pools (SFPs) as  
19 a means to demonstrate compliance with applicable regulations in Section 50.68 of Title 10 of  
20 the *Code of Federal Regulations* (10 CFR), "Criticality Accident Requirements," 10 CFR Part 50,  
21 Appendix A, General Design Criterion (GDC) 61, "Fuel Storage and Handling and Radioactivity  
22 Control," and 10 CFR Part 50, Appendix A, GDC 62 "Prevention of Criticality in Fuel Storage  
23 and Handling," with respect to neutron absorbing materials (NAMs).

24  
25 2.0 REGULATORY EVALUATION

26  
27 The credited neutron absorber monitor (NAM) installed in SFP storage racks ensures that the  
28 effective multiplication factor ( $k_{\text{eff}}$ ) does not exceed the values and assumptions used in the  
29 criticality analysis of record (AOR) and other licensing basis documents. The AOR is the basis,  
30 in part, for demonstrating compliance with plant technical specifications and with applicable  
31 NRC regulations. Degradation or deformation of the credited NAM may reduce safety margin  
32 and potentially challenge the subcriticality requirement. NAMs utilized in SFP racks exposed to  
33 treated water or treated borated water may be susceptible to reduction of neutron absorbing  
34 capacity, changes in dimension that increase  $k_{\text{eff}}$ , and loss of material. A monitoring program is  
35 implemented to ensure that degradation of the NAM used in SFPs, which could compromise the  
36 ability of the NAM to perform its safety function as assumed in the AOR, will be detected. The  
37 U.S. Nuclear Regulatory Commission's (NRC's) regulatory requirements and corresponding  
38 staff review criteria and guidance for NAM monitoring programs are contained in the following  
39 documents:

- 40  
41 • 10 CFR 50.68(b)(4), "Criticality accident requirements," states that if the licensee does  
42 not credit soluble boron in the SFP criticality AOR, the  $k_{\text{eff}}$  of the SFP  
43 storage racks must not exceed 0.95 at a 95 percent probability, 95 percent confidence  
44 level. If the licensee does take credit for soluble boron, the  $k_{\text{eff}}$  of the SFP storage racks  
45 must not exceed 0.95 at a 95 percent probability, 95 percent confidence level, if flooded  
46 with borated water, and if flooded with unborated water, the  $k_{\text{eff}}$  must remain below 1.0 at  
47 a 95 percent probability, 95 percent confidence level. The  $k_{\text{eff}}$  is defined as the effective  
48 neutron multiplication factor.

Enclosure

- 1 • GDC 61, "Fuel storage and handling and radioactivity control," states that "The fuel  
2 storage and handling, radioactive waste, and other systems which may contain  
3 radioactivity shall be designed to assure adequate safety under normal and postulated  
4 accident conditions. These systems shall be designed (1) with a capability to permit  
5 appropriate periodic inspection and testing of components important to safety..."  
6
- 7 • GDC 62, "Prevention of Criticality in Fuel Storage and Handling," states that "Criticality in  
8 the fuel storage and handling system shall be prevented by physical systems or  
9 processes, preferably by use of geometrically safe configurations."  
10
- 11 • NUREG-0800, "Standard Review Plan [(SRP)]," Section 9.1.1, Revision 3, "Criticality  
12 Safety of Fresh and Spent Fuel Storage and Handling" (ADAMS Accession  
13 No. ML070570006) provides guidance regarding the acceptance criteria and review  
14 procedures to ensure that the proposed changes satisfy the requirements in  
15 10 CFR 50.68.  
16
- 17 • NUREG-0800, "Standard Review Plan," Section 9.1.2, Revision 4, "New and Spent Fuel  
18 Storage" (ADAMS Accession No. ML07055057) (ML103490041) provides guidance  
19 regarding the acceptance criteria and review procedures to ensure that the proposed  
20 changes satisfy the requirements in 10 CFR 50.68.  
21
- 22 • NUREG-1801, "Generic Aging Lessons Learned (GALL) Report," Revision 2 (ADAMS  
23 Accession No. ML103490041) provides guidance on what constitutes an acceptable  
24 monitoring program for NAM credited for criticality control in the SFP.  
25

### 26 3.0 TECHNICAL EVALUATION

27  
28 Guidance for developing a NAM monitoring program for the SFP is provided in NEI 16-03. The  
29 purpose of a NAM monitoring program is to verify that the NAM installed in SFPs continues to  
30 perform its safety function (i.e., criticality control) as assumed in the AOR. The guidance  
31 provided in NEI 16-03 for a NAM monitoring program, relies on periodic inspection, testing,  
32 monitoring, and analysis of the NAM to ensure that the required subcriticality margin is  
33 maintained in accordance with 10 CFR 50.68 requirements. To accomplish this purpose, the  
34 guidance document states that a monitoring program must be capable of identifying  
35 unanticipated changes in the absorber material and determining whether anticipated changes  
36 can be verified. The guidance recommends a combination of coupon testing, in-situ  
37 measurement, and SFP water chemistry monitoring as a means to monitor potential changes in  
38 characteristics of the NAM. The NRC staff reviewed the proposed guidance for what constitutes  
39 an acceptable monitoring program and its ability to ensure that potential degradation of SFP  
40 NAM will be detected, monitored, and mitigated. The staff determined that an appropriate  
41 combination of the three methods listed above (coupon testing, in-situ measurement, and SFP  
42 water chemistry monitoring) can comprise an effective NAM monitoring program. During the  
43 course of the NRC staff's review, there were several topics identified in the guidance that  
44 required clarification. A Category 2 public meeting was held with NEI on August 10, 2016, to  
45 seek clarification on these topics. The NRC staff and NEI representatives discussed these  
46 topics and NEI subsequently submitted a revision to NEI 16-03. A meeting summary is included  
47 as a reference (ADAMS Accession No. ML16209A375) in this safety evaluation (SE) that

describes the topics that were discussed at the public meeting, as well as the changes that were made to the guidance document as a result of the discussion.

### 3.1 Coupon Testing Program

#### 3.1.1 Description of NEI 16-03

The guidance document states that the use of a coupon testing program is the preferred method for a neutron absorber monitoring program. This program consists of small sections (coupons) of the same NAM installed in the SFP, which are attached to a structure (coupon tree) in the SFP. The coupon tree is placed near freshly discharged fuel assemblies in an attempt to accelerate potential degradation mechanisms. The document provides the following criteria for an acceptable coupon program:

- The number of coupons needs to be adequate to allow for sampling at interval for the intended life of the absorbers.
- The sampling intervals are based on the expected rate of material change.
- Performance of coupon testing
  - Basic testing: visual observations, dimensional measurements, and weight
  - Full testing: density measurements, Boron-10 ( $^{10}\text{B}$ ) areal density (AD) measurements, microscopic analysis, and characterization of changes, in addition to the basic testing parameters

The guidance document states that the coupons will be located in the SFP “such that their exposure to parameters controlling change mechanisms is conservative or similar to the in-service neutron absorbers.” For neutron attenuation testing, NEI 16-03 provides acceptance criteria for the NAM depending on if there is, or is not, an anticipated loss of  $^{10}\text{B}$  AD. The acceptable result for NAMs with expected  $^{10}\text{B}$  AD loss is the  $^{10}\text{B}$  AD of the test coupon is greater than the  $^{10}\text{B}$  AD assumed in the licensee’s SFP criticality AOR. For NAM without an expected loss of  $^{10}\text{B}$  AD, the acceptable result is the  $^{10}\text{B}$  AD of the test coupon is equal to the original  $^{10}\text{B}$  AD of the coupon (within measurement uncertainty). Furthermore, the guidance states that the acceptable initial sampling interval for testing of new material (i.e., with a limited, or no, operating history) is 5 years, with subsequent intervals up to 10 years. For those materials that have well documented operating experience, do not have a history of degradation or degradation mechanisms, and information on stability of the material condition is well developed, the document states initial and subsequent test intervals up to 10 years are acceptable. The document states that for materials with known degradation mechanisms, or a history of known degradation (e.g., Boraflex, Carborundum, Tetrabor, etc.), the acceptable interval for neutron attenuation testing is at least once every five years. In addition, NEI 16-03 includes neutron attenuation testing in the full testing approach for any NAMs used, as a component of a satisfactory NAM monitoring program.

#### 3.1.2 NRC Staff Evaluation

The NRC staff has evaluated the guidance for the basic and full portions of a coupon testing program. The basic portion of the testing includes methods to monitor the physical condition of the NAM so that signs of potential degradation may be observed. The full portion of the testing

1 includes neutron attenuation testing for all NAMs that are credited in the SFP criticality analysis  
2 that will allow the licensee to detect a potential loss in  $^{10}\text{B}$  AD. The staff finds the coupon testing  
3 program to be acceptable because it includes measurements of  $^{10}\text{B}$  AD and of geometric  
4 changes in the material that can impact the ability of the NAM to perform its function as  
5 assumed in the licensee's SFP criticality AOR.  
6

7 The NRC staff also determined the acceptance criteria for the coupon testing program provided  
8 in NEI 16-03 is acceptable. The acceptance criteria provide reasonable assurance that the  
9 assumptions in the licensee's SFP criticality AOR will be maintained, because the acceptance  
10 criteria show that the material is either not losing  $^{10}\text{B}$  AD (for materials not expected to lose  
11  $^{10}\text{B}$  AD), or the  $^{10}\text{B}$  AD is still above the  $^{10}\text{B}$  AD assumed in the licensee's SFP criticality AOR  
12 (for NAM anticipated to lose  $^{10}\text{B}$  AD). The NRC staff also finds that it is not acceptable for the  
13 measurement uncertainty to result in a  $^{10}\text{B}$  AD value that is lower than the assumed value in the  
14 SFP criticality AOR. In addition, the NRC staff recognizes that if a coupon being tested  
15 approaches the  $^{10}\text{B}$  AD limit as stated in the licensee's SFP criticality AOR, the licensee would  
16 likely need to perform further evaluations and/or take additional corrective actions to provide  
17 reasonable assurance that the in-service NAM will not exceed the stated  $^{10}\text{B}$  AD limit, given the  
18 active degradation of the NAM. Guidance on additional corrective actions that may be  
19 necessary is given in Section 2.3, "Evaluating Neutron Absorber Test Results," of NEI 16-03,  
20 and this guidance is evaluated in Section 3.3 of this SE.  
21

### 22 3.2 In-Situ Measurement Program

#### 23 3.2.1 Description of NEI 16-03

24 The NEI guidance document states that in-situ measurement is another method that can be  
25 used to confirm  $^{10}\text{B}$  AD of NAM. It further states that this method can be used to supplement  
26 coupon monitoring to extend the coupon testing interval, permit greater reliance on basic  
27 testing, or in lieu of coupon testing for plants that may no longer have coupons in the SFP. It  
28 also states that in-situ measurement can be used instead of coupon testing if coupons do not  
29 exist.  
30  
31  
32

33 The guidance states that all in-situ measurement campaigns are to be performed at an  
34 acceptable interval and on an adequate number of panels. The guidance gives two options for  
35 determining what constitutes an adequate number of panels. The first option uses the  
36 methodology of NUREG-6698 (ADAMS Accession No. ML050250061) to measure a minimum  
37 of 59 panels to provide 95/95 confidence limits. The second option selects the panels with the  
38 greatest exposure (top 5%) to parameters that influence degradation (e.g., neutron fluence,  
39 temperature, time). The amount of panels will be no less than one percent of the total panels in  
40 the SFP, although more panels can be tested from other areas of the SFP to gain a more  
41 representative sampling. The guidance also states sources of uncertainty in the in-situ  
42 measurement will be identified and quantified.  
43

44 The sampling interval will be based upon the NAM credited in the SFP. New materials with  
45 minimal operating experience will have an initial test interval that does not exceed 5 years, with  
46 subsequent intervals up to 10 years (with appropriate operating experience). For materials with  
47 known histories of degradation and known degradation mechanisms, test intervals do not  
48 exceed 5 years. For other materials that do not have known histories of degradation or known

1 degradation mechanisms test intervals will not exceed 10 years. The guidance also states that  
2 if used in conjunction with a coupon monitoring program, the in-situ sampling interval can be  
3 longer.  
4

5 The NEI document also provides acceptance criteria for in-situ measurements. It states that for  
6 NAMs that do not have potential degradation mechanisms for loss of  $^{10}\text{B}$  AD, results of the  
7 in-situ measurements are acceptable if the nominal measured  $^{10}\text{B}$  AD is greater than or equal  
8 to the value assumed in the licensee's criticality AOR (within measurement uncertainties). For  
9 materials that have potential degradation mechanisms that result in loss of  $^{10}\text{B}$  AD, results are  
10 considered acceptable if the nominal measured  $^{10}\text{B}$  AD minus measurement uncertainty is  
11 greater than the  $^{10}\text{B}$  AD in the licensee's criticality AOR.  
12

### 13 3.2.2 NRC Staff Evaluation 14

15 The NRC staff has reviewed the guidance for performing in-situ measurement testing and finds  
16 it to be acceptable, because it allows for detection of degradation mechanisms, potential loss of  
17 neutron absorption capacity (e.g. loss of  $^{10}\text{B}$ ), and ensure the NAM will continue to perform its  
18 safety function. The NRC staff reviewed the methodology recommended for determining the  
19 number of panels that may be selected for in-situ inspection and finds it to be acceptable  
20 because it is based in part on guidance provided in NUREG-6698, "Guide for Validation of  
21 Nuclear Criticality Safety Calculational Methodology," and on selecting panels that have  
22 experienced the greatest exposure to the SFP environment. The NRC staff also finds that  
23 depending on the population of NAM panels in the SFP, a licensee may need to measure more  
24 than the minimum of 59 panels in order to produce 95/95 confidence limits. The method used  
25 for selecting the panels for in-situ testing is used to obtain data that is bounding or  
26 representative of the entire NAM in the SFP.  
27

28 In addition, the NRC staff has determined that the proposed testing intervals (10 year intervals  
29 for materials with no known history of degradation/degradation mechanisms, and 5 year  
30 intervals for materials with a known history of degradation/degradation mechanisms or for new  
31 materials (i.e., no operating history)) are acceptable and consistent with NRC guidance in the  
32 GALL report, Revision 2. The neutron attenuation testing must be performed on the intervals as  
33 described in the document, regardless of how the licensee uses the in-situ monitoring program  
34 (e.g., in conjunction with coupons, without coupon program, or other reasons as described in  
35 NEI 16-03). The statement in the guidance that the in-situ sampling interval can be longer if  
36 used in conjunction with a coupon program does not obviate the need to perform neutron  
37 attenuation testing on the 5 or 10 year intervals.  
38

39 In addition, the NRC staff finds it to be acceptable to identify and evaluate sources of  
40 uncertainty in order to assess the reliability of the instruments and methodology used to the  
41 collect the data. Sources of uncertainty can greatly impact results and confidence in the data  
42 collected, especially as it relates to the subcriticality margin.  
43

### 3.3 Evaluating Neutron Absorber Test Results

#### 3.3.1 Description of NEI 16-03

The guidance document states that the test results from neutron absorber monitoring may fall within the following categories:

- 1) Confirmation that no material changes are occurring
- 2) Confirmation that anticipated changes are occurring, and/or
- 3) Identification that unanticipated changes are occurring.

Furthermore, the guidance document states that the testing results will be compared to the AOR input (i.e., <sup>10</sup>B AD assumed in criticality AOR). If no changes, or if anticipated changes are occurring, then the guidance assumes that the material continues to be adequately represented in the AOR.

The guidance document also describes the additional actions that may be necessary when unanticipated changes in the NAM are identified. It states that there are certain technical evaluations that may be necessary in addition to any required regulatory or licensing processes. The technical evaluations include a determination if these changes may result in a loss of <sup>10</sup>B AD. Any potential impacts of a loss of <sup>10</sup>B AD on the SFP criticality AOR will be evaluated and addressed through licensee procedures. In addition, the results of monitoring and testing are to be evaluated and trended, regardless of potential impact on the SFP criticality AOR. If the unanticipated changes do not appear to result in the loss of <sup>10</sup>B AD, the changes will still be evaluated for impacts on the SFP criticality AOR. The effects on the SFP criticality AOR due to potential dimensional changes of the NAM, or other material in the SFP, are evaluated and addressed in accordance with licensee procedures.

#### 3.3.2 NRC Staff Evaluation

The NRC staff has reviewed the actions described in the guidance for when potential degradation is detected in the neutron absorbing material as potential degradation of the NAM may impact <sup>10</sup>B AD assumptions in the SFP criticality AOR. The NRC staff finds the actions described in the guidance acceptable because they will be able to identify anticipated, and unanticipated changes in order to provide information that will allow a licensee to determine whether or not the neutron absorbing material is performing its safety function (i.e., whether or not there is a loss of <sup>10</sup>B AD).

The NRC staff has also determined that it is necessary to evaluate and trend the results of <sup>10</sup>B AD measurements from neutron attenuation testing in the NAM as described in NEI 16-03. The NRC staff finds the methods, and requirement, to trend data acceptable because it will provide information regarding the potential degradation mechanism(s) and rate for the NAM in the SFP. This information will also help to provide reasonable assurance that the <sup>10</sup>B AD of the NAM will not decrease below the value assumed in the SFP criticality AOR between the specified test intervals for neutron attenuation testing. In addition, this data can identify previously un-evaluated degradation mechanisms that may have an impact on the SFP criticality AOR.



1 The actions described above ensure, in part, that the ability of the NAM to perform its safety  
2 function as assumed in the AOR, is maintained.

### 3 4 3.4 Technical Evaluation Conclusion

5  
6 The NRC staff has determined that in order for a NAM monitoring program to be acceptable,  
7 the licensee must perform neutron attenuation testing at the intervals stated in NEI 16-03. The  
8 NRC staff finds the interval for inspection and testing acceptable because the frequency is  
9 determined based on the neutron absorbing material credited, and operating experience of that  
10 material. Depending on the material used, the interval for neutron attenuation testing will not  
11 exceed 5 years (for materials with a history of known degradation or a known degradation  
12 mechanism, and new materials), or 10 years (for other materials that do not have a history of  
13 degradation, or a known degradation mechanism). Periodic neutron attenuation testing, and the  
14 intervals described in NEI 16-03, are consistent with staff guidance (i.e., the GALL Report,  
15 Revision 2). Licensees must request site-specific NRC review and approval to extend the  
16 interval of any neutron attenuation testing past the 5 year and 10 year intervals, as described in  
17 NEI 16-03.

18  
19 Based on its review of NEI 16-03, the NRC staff has determined that a NAM monitoring program  
20 meeting the provisions in NEI 16-03 will allow a licensee to reasonably ensure that the ability of  
21 the NAM to perform its safety function, as assumed in the AOR, is maintained, thus  
22 demonstrating compliance with the subcriticality requirements of 10 CFR 50.68.

### 23 24 4.0 CONCLUSION

25  
26 The NRC staff has reviewed NEI 16-03, and the proposed methods for developing a NAM  
27 monitoring program. A NAM monitoring program implementing the proposed guidance provides  
28 reasonable assurance that such program will be able to detect degradation of neutron absorbing  
29 material, and provides assurance that the ability of the NAM to perform its safety function as  
30 assumed in the AOR, is maintained. The NRC staff finds that the requirements of  
31 10 CFR 50.68(b)(4), GDC 61, and GDC 62, as well as the guidance provided in SRP 9.1.1,  
32 SRP 9.1.2, and the GALL, Revision 2, with respect to NAMs and the NAM monitoring program,  
33 are satisfied. Therefore, the NRC staff finds the proposed guidance in NEI 16-03 acceptable for  
34 developing a NAM monitoring program.

### 35 36 5.0 REFERENCES

- 37  
38 1. Letter from Kristopher W. Cummings to Timothy J. McGinty, "Submittal of NEI 16-03,  
39 Guidance for Monitoring of Fixed Neutron Absorbers in Spent Fuel Pools, Draft A, dated  
40 May 2016," May 10, 2016 (ADAMS Accession No. ML16147A078).  
41  
42 2. Letter from Kristopher W. Cummings to Brian J. Benney, "Submittal of NEI 16-03,  
43 Guidance for Monitoring of Fixed Neutron Absorbers in Spent Fuel Pools, Revision 0,  
44 dated August 2016," August 30, 2016 (ADAMS Accession No. ML16265A248).  
45  
46 3. Federal Regulation, U.S. Code of Federal Regulations, "Criticality accident  
47 requirements," Title 10 of the *Code of Federal Regulations* Part 50, Section 50.68(b)(4).  
48

- 1 4. U.S. Code of Federal Regulations, "Domestic Licensing of Production and Utilization  
2 Facilities – Proposed General Design Criteria for Nuclear Power Plants," Part 50,  
3 Appendix A (10 CFR Part 50, Appendix A).  
4
- 5 5. U.S. Nuclear Regulatory Commission, "Standard Review Plan, Section 9.1.1, Criticality  
6 Safety of Fresh and Spent Fuel Storage and Handling," NUREG-0800, Revision 3,  
7 March 2007 (ADAMS Accession No. ML070570006)  
8
- 9 6. U.S. Nuclear Regulatory Commission, "Standard Review Plan, Section 9.1.1, New and  
10 Spent Fuel Storage," NUREG-0800, Revision 4, March 2007 (ADAMS Accession  
11 No. ML070550057)  
12
- 13 7. U.S. Nuclear Regulatory Commission, "Generic Aging Lessons Learned (GALL) Report,"  
14 NUREG-1801, Revision 2, December 2010 (ADAMS Accession No. ML103490041)  
15
- 16 8. U.S. Nuclear Regulatory Commission Category 2 Public Meeting, 'Summary of  
17 August 10, 2016, Meeting with the Nuclear Energy Institute to Discuss NEI 16-03,  
18 "Guidance for Monitoring of Fixed Neutron Absorbers in Spent Fuel Pools,"  
19 September 27, 2016 (ADAMS Accession No. ML16209A375)  
20
- 21 9. U.S. Nuclear Regulatory Commission, "Guide for Validation of Nuclear Criticality Safe  
22 Calculation Methodology," NUREG/CR-6698, January 2001 (ADAMS Accession  
23 No. ML050250061)  
24

25 Principle Contributor: Alex Chereskin, NRR

26  
27 Date:  
28