

M. J. Yox
Regulatory Affairs Director
Vogtle 3&4
Nuclear Development

Southern Nuclear
Operating Company, Inc.
7825 River Road
Waynesboro, GA 30830

Tel 706.848.6459



Docket No.: 52-025

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ND-16-1842
10 CFR 52.99(c)(3)

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555-0001

Southern Nuclear Operating Company
Vogtle Electric Generating Plant Unit 3
Notice of Uncompleted ITAAC 225-days Prior to Initial Fuel Load
Item 2.2.03.08c.iv.01 [Index Number 183]

Ladies and Gentlemen:

Pursuant to 10 CFR 52.99(c)(3), Southern Nuclear Operating Company hereby notifies the NRC that as of September 30, 2016, Vogtle Electric Generating Plant (VEGP) Unit 3 Uncompleted Inspection, Test, Analysis, and Acceptance Criteria (ITAAC) Item 2.2.03.08c.iv.01 [Index Number 183] has not been completed greater than 225-days prior to initial fuel load. Enclosure 1 describes the plan for completing ITAAC 2.2.03.08c.iv.01 [Index Number 183]. Southern Nuclear Operating Company will at a later date provide additional notifications for ITAAC that have not been completed 225-days prior to initial fuel load.

This notification is informed by the guidance described in NEI-08-01, *Industry Guideline for the ITAAC Closure Process Under 10 CFR Part 52*, which was endorsed by the NRC in Regulatory Guide 1.215. In accordance with NEI 08-01, this notification includes ITAAC for which required inspections, tests, or analyses have not been performed or have been only partially completed. All ITAAC will be fully completed and all Section 52.99(c)(1) ITAAC Closure Notifications will be submitted to NRC to support the Commission finding that all acceptance criteria are met prior to plant operation, as required by 10 CFR 52.103(g).

This letter contains no new NRC regulatory commitments.

If there are any questions, please contact David Woods at 706-848-6903.

Respectfully submitted,


Michael J. Yox
Regulatory Affairs Director Vogtle 3&4

MJY/KMS/amm

U.S. Nuclear Regulatory Commission

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Enclosure:

1. Vogtle Electric Generating Plant (VEGP) Unit 3 Completion Plan for Uncompleted ITAAC
Item 2.2.03.08c.iv.01 [Index Number 183]

To:

Southern Nuclear Operating Company/Georgia Power Company

Mr. S. E. Kuczynski (w/o enclosures)

Mr. D. A. Bost (w/o enclosures)

Mr. M. D. Meier

Mr. M. D. Rauckhorst (w/o enclosures)

Mr. D. H. Jones (w/o enclosures)

Ms. K. D. Fili

Mr. D. L. McKinney

Mr. B. H. Whitley

Mr. D. L. Fulton

Mr. C. E. Morrow

Mr. M. J. Yox

Mr. D. Woods

Ms. A. L. Pugh

Ms. K. M. Stacy

Mr. A. S. Parton

Mr. W. A. Sparkman

Mr. J. P. Redd

Mr. D. R. Culver

Mr. F. H. Willis

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cc:

Nuclear Regulatory Commission

Ms. C. Haney (w/o enclosures)

Ms. A. Bradford (w/o enclosures)

Ms. J. L. Dixon-Herrity (w/o enclosures)

Ms. J. M. Heisserer

Mr. C. J. Even

Mr. C. P. Patel

Mr. B. M. Baval

Ms. R. C. Reyes

Ms. M. A. Sutton

Mr. M. E. Ernstes

Mr. G. J. Khouri

Mr. M. G. Kowal

Mr. J. D. Fuller

Mr. T. E. Chandler

Ms. S. E. Temple

Ms. P. Braxton

Mr. M. A. Junge

Mr. T. C. Brimfield

Mr. A. J. Lerch

Oglethorpe Power Corporation

Mr. M. W. Price
Ms. K. T. Haynes
Ms. A. Whaley

Municipal Electric Authority of Georgia

Mr. J. E. Fuller
Mr. S. M. Jackson

Dalton Utilities

Mr. D. Cope

WECTEC

Ms. K. Stoner (w/o enclosures)
Mr. C. A. Castell

Westinghouse Electric Company, LLC

Mr. R. Easterling (w/o enclosures)
Mr. J. W. Crenshaw (w/o enclosures)
Mr. L. Woodcock (w/o enclosures)
Mr. C. F. Landon
Mr. P. A. Russ
Mr. A. F. Dohse
Mr. M. Y. Shaqqo
Ms. S. DiTommaso

Other

Mr. J. E. Hesler, *Bechtel Power Corporation*
Ms. L. Matis, *Tetra Tech NUS, Inc.*
Dr. W. R. Jacobs, Jr., *Ph.D., GDS Associates, Inc.*
Mr. S. Roetger, *Georgia Public Service Commission*
Ms. S. W. Kernizan, *Georgia Public Service Commission*
Mr. K. C. Greene, *Troutman Sanders*
Mr. S. Blanton, *Balch Bingham*

ND-16-1842
Enclosure 1
Completion Plan

Southern Nuclear Operating Company

ND-16-1842

Enclosure 1

Vogtle Electric Generating Plant (VEGP) Unit 3

**Completion Plan for Uncompleted ITAAC
Item 2.2.03.08c.iv.01 [Index No. 183]**

Subject: Uncompleted ITAAC 2.2.03.08c.iv.01 [Index No. 183]

ITAAC Statement

Design Commitment

8.c) *The PXS provides RCS makeup, boration, and safety injection during design basis events.*

Inspections/Tests/Analyses

iv) *Inspections of the elevation of the following pipe lines will be conducted:*

1. IRWST injection lines; IRWST connection to DVI nozzles

Acceptance Criteria

iv) *The maximum elevation of the top inside surface of these lines is less than the elevation of:*

1. IRWST bottom inside surface

ITAAC Completion Description

Multiple ITAAC are performed to demonstrate that the Passive Core Cooling System (PXS) provides Reactor Coolant System (RCS) makeup, boration, and safety injection during design basis events. This ITAAC requires that inspections be conducted to verify that the maximum elevation of the top inside surface of the In-containment Refueling Water Storage Tank (IRWST) injection lines and IRWST connection to Direct Vessel Injection (DVI) nozzles is less than the elevation of IRWST bottom inside surface.

The inspection of the IRWST injection lines top inside surface, IRWST connection to the DVI nozzles top inside surface, and the IRWST bottom inside surface elevations is performed using survey equipment in accordance with site survey and measurement procedures. The maximum measured elevation of the IRWST injection lines top inside surface and IRWST connection to the DVI nozzles top inside surface is compared to the measured elevation of the IRWST bottom inside surface using a common reference point.

The inspection results are documented in the Principal Closure Document XXX (Reference 1) supporting the ITAAC 2.2.03.08c.iv.01 Completion Package (Reference 2) and determined that the maximum elevation of the top inside surface of the IRWST injection lines and IRWST connection to DVI nozzles is xxx feet and the elevation of IRWST bottom inside surface is yyy feet. The inspection results verify that the maximum elevation of the top inside surface of the IRWST injection lines and IRWST connection to DVI nozzles is less than the elevation of IRWST bottom inside surface.

Principal Closure Document XXX exists and is available for NRC inspection as part of the ITAAC 2.2.03.08c.iv.01 Completion Package.

List of ITAAC Findings

In accordance with plant procedures for ITAAC completion, Southern Nuclear Operating Company (SNC) performed a review of all findings pertaining to the subject ITAAC and associated corrective actions. This review found there are no relevant ITAAC findings associated with this ITAAC.

References (available for NRC inspection)

1. Principal Closure Document XXX
2. ITAAC 2.2.03.08c.iv.01 Completion Package
3. NEI 08-01, "Industry Guideline for the ITAAC Closure Process Under 10 CFR Part 52"