



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**
WASHINGTON, D.C. 20555-0001

November 2, 2016

Mr. Peter P. Sena, III
President
PSEG Nuclear LLC - N09
P.O. Box 236
Hancocks Bridge, NJ 08038

**SUBJECT: HOPE CREEK GENERATING STATION – REQUEST FOR ADDITIONAL
INFORMATION REGARDING REQUEST TO DELETE TECHNICAL
SPECIFICATION ACTION STATEMENT 3.4.2.1.b ASSOCIATED WITH STUCK
OPEN SAFETY/RELIEF VALVES (CAC NO. MF7709)**

Dear Mr. Sena:

By letter dated May 11, 2016 (Agencywide Documents Access and Management System Accession No. ML16132A374), PSEG Nuclear LLC (PSEG, the licensee) submitted a license amendment request (LAR) for Hope Creek Generating Station (Hope Creek). The LAR proposes to revise Hope Creek's Technical Specification (TS) requirements by deleting TS Action Statement 3.4.2.1.b concerning stuck open safety/relief valves.

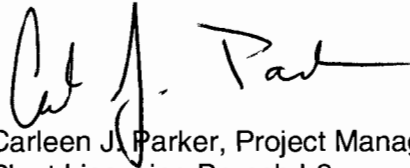
The U.S. Nuclear Regulatory Commission staff has reviewed the licensee's application and, based upon this review, determined that additional information is needed to complete our review. On September 22, 2016, a draft of these questions was sent to Mr. Paul Duke of your staff to ensure that the questions were understandable, the regulatory basis for the questions was clear, and to determine if the information was previously docketed. On October 18, 2016, a teleconference was held to clarify the questions. On October 24, 2016, Mr. Duke indicated that PSEG will submit a response within 45 days of the date of this letter.

P. Sena

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If you have any questions, please contact me at (301) 415-1603 or by e-mail at Carleen.Parker@nrc.gov.

Sincerely,

A handwritten signature in black ink, appearing to read "Carleen J. Parker". The signature is fluid and cursive, with the first name "Carleen" and last name "Parker" clearly distinguishable.

Carleen J. Parker, Project Manager
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-354

Enclosure:
Request for Additional Information

cc w/enclosure: Distribution via Listserv

REQUEST FOR ADDITIONAL INFORMATION

REQUEST TO DELETE TECHNICAL SPECIFICATION ACTION STATEMENT 3.4.2.1.b

ASSOCIATED WITH STUCK OPEN SAFETY/RELIEF VALVES

PSEG NUCLEAR LLC

HOPE CREEK GENERATING STATION

DOCKET NO. 50-354

By letter dated May 11, 2016 (Agencywide Documents Access and Management System Accession No. ML16132A374), PSEG Nuclear LLC (the licensee) submitted a license amendment request (LAR) for Hope Creek Generating Station (Hope Creek). The LAR proposes to revise Hope Creek's Technical Specification (TS) requirements by deleting TS Action Statement 3.4.2.1.b concerning stuck open safety/relief valves. The U.S. Nuclear Regulatory Commission (NRC) staff has reviewed the application and, based upon this review, determined that additional information requested below is needed to complete our review.

TS Action Statement 3.4.2.1.b currently states:

With one or more safety/relief valves stuck open, provided that suppression pool average water temperature is less than 110°F, close the stuck open safety relief valve(s); if unable to close the stuck open valve(s) within 2 minutes or if suppression pool average water temperature is 110°F or greater, place the reactor mode switch in the Shutdown position.

TS 3.6.2.1 Action b.2, also requires the reactor mode switch to be placed in the shutdown position when the suppression chamber average water temperature exceeds 110 degrees Fahrenheit (°F), while in Modes 1, 2, and 3. The licensee states that this action will remain unchanged and provides appropriate remedial action, permitted by TSs, until the limiting condition for operation can be met.

Title 10 of the *Code of Federal Regulations* (10 CFR), Section 50.36, "Technical specifications," establishes the NRC's regulatory requirements related to the content of TSs. Pursuant to 10 CFR 50.36, TSs are required to include items in the following five specific categories related to station operation: (1) safety limits, limiting safety system settings, and limiting control settings; (2) limiting conditions for operation; (3) surveillance requirements; (4) design features; and (5) administrative controls.

Request for Additional Information

In order for the staff to evaluate the licensee's proposed deletion of TS 3.4.2.1, Action b associated with the subject safety-relief valves (SRVs), the NRC staff requests the following additional information:

1. Since TS requirements are derived from a licensee's licensing basis, please
(1) provide documentation/references related to Hope Creek's current licensing

Enclosure

basis that requires inclusion of the '2 minute' SRV closure requirement in TS 3.4.2.1.b, and (2) indicate how long this requirement has been in the current TSs.

2. Please provide additional justification (such as operating experience, etc.) for the deletion of the proposed change.

In order for the staff to evaluate the licensee's statement that the requirement for the reactor mode switch to be placed in the shutdown position when the suppression chamber average water temperature exceeds 110 °F while in Modes 1, 2, and 3 will not be changed, the NRC staff requests the following additional information

3. Please clarify when TS 3.6.2.1 Action b.2 applies. Specifically, address the applicable thermal power.
4. For TS 3.6.2.1 Action b, for a stuck open SRV in Modes 1, 2, or 3, how is the requirement to place the mode switch in shutdown accomplished during the allowed time to restore temperature below the maximum average temperature of 95 °F?

P. Sena

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If you have any questions, please contact me at (301) 415-1603 or by e-mail at Carleen.Parker@nrc.gov.

Sincerely,

/RA/

Carleen J. Parker, Project Manager
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-354

Enclosure:
Request for Additional Information

cc w/enclosure: Distribution via Listserv

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***by e-mail dated**

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