



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

December 7, 2016

Mr. Jerud Hanson  
Nuclear Energy Institute  
1201 F St., NW, Suite 1100  
Washington, DC 20004-1218

SUBJECT: LICENSE RENEWAL INTERIM STAFF GUIDANCE, LR-ISG-2016-01, "CHANGES TO AGING MANAGEMENT GUIDANCE FOR VARIOUS STEAM GENERATOR COMPONENTS"

Dear Mr. Hanson:

On June 7, 2016, the U.S. Nuclear Regulatory Commission (NRC) published in the *Federal Register* (81 FR 36612) a request for public comment on draft License Renewal Interim Staff Guidance (LR-ISG), LR-ISG-2016-01, "Changes to Aging Management Guidance for Various Steam Generator Components." In response, the Nuclear Energy Institute (NEI) submitted comments by letter dated July 7, 2016, available in the NRC's Agencywide Documents Access and Management System (ADAMS) at Accession No. ML16194A026. The NRC has reviewed and dispositioned the public comments as discussed in Appendix C of the final LR-ISG-2016-01.

The major changes to aging management guidance in this LR-ISG are: (a) addition of visual inspections of steam generator head internal areas to the Generic Aging Lessons Learned (GALL) Report, Section XI.M19, "Steam Generators," to detect signs of cracking or loss of material (e.g., detection of rust stains and gross cracking or distortion); (b) changes to the Standard Review Plan for Review of License Renewal Applications for Nuclear Power Plants (SRP-LR), Sections 3.1.2.2.11 and 3.1.3.2.11, "Cracking due to Primary Water Stress Corrosion Cracking," that describe further evaluation regarding primary water stress corrosion cracking (PWSCC) in divider plate assemblies and tube-to-tubesheet welds; and (c) changes to the aging management review items in the GALL Report and SRP-LR for managing cracking due to PWSCC in divider plate assemblies and tube-to-tubesheet welds and loss of material due to boric acid corrosion in steam generator heads (interior surfaces) and tubesheets (primary side). If a licensee demonstrates that industry generic analyses (EPRI Report 3002002850) are applicable and bounding for its unit, a plant-specific program (e.g., one-time inspection program) to ensure that cracking is not occurring in the divider plate assembly and tube-to-tubesheet welds is not necessary in accordance with the revised guidance in SRP-LR, Sections 3.1.2.2.11 and 3.1.3.2.11.

A *Federal Register* notice announcing the issuance of LR-ISG-2016-01 has been published (81 FR 88276). Also, courtesy copies of this letter are being sent to subscribers of an NRC electronic mailing list on general license renewal topics.

We will make corresponding changes to the aging management guidance for the subsequent license renewal period (i.e., up to 80 years of operation), which is documented in draft NUREG-2191, "Generic Aging Lessons Learned for Subsequent License Renewal Report" (GALL-SLR), and draft NUREG-2192, "Standard Review Plan for Review of Subsequent License Renewal for Nuclear Power Plants."

J. Hanson

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If you have any questions, please contact Dr. Seung Min of my staff by telephone at 301-415-2045 or by e-mail at [Seung.Min@nrc.gov](mailto:Seung.Min@nrc.gov).

Sincerely,

**/RA/**

Benjamin G. Beasley, Acting Deputy Director  
Division of License Renewal  
Office of Nuclear Reactor Regulation

Enclosure:  
Final LR-ISG-2016-01

cc: Reactor License Renewal Stakeholder GovDelivery

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**/RA/**

Benjamin G. Beasley, Acting Deputy Director  
Division of License Renewal  
Office of Nuclear Reactor Regulation

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Letter to J. Hanson from B. Beasley dated December 7, 2016.

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