

**UNITED STATES NUCLEAR REGULATORY COMMISSION
PRESSURIZED WATER REACTOR GENERIC FUNDAMENTALS EXAMINATION
DECEMBER 2012--FORM A**

Please Print

Name: _____

Docket No.: _____

Facility: _____

Start Time: _____ Stop Time: _____

INSTRUCTIONS TO APPLICANT

Answer all the test items using the answer sheet provided, ensuring a single answer is marked for each test item. Each test item has equal point value. A score of at least 80 percent is required to pass this portion of the NRC operator licensing written examination. All examination materials will be collected 3 hours after the examination begins. This examination applies to a typical U.S. pressurized water reactor (PWR) nuclear power plant.

SECTION	QUESTIONS	% OF TOTAL	SCORE
COMPONENTS	1 - 22		
REACTOR THEORY	23 - 36		
THERMODYNAMICS	37 - 50		
TOTALS	50		

All work performed on this examination is my own. I have neither given nor received aid.

Applicant's Signature

RULES AND INSTRUCTIONS FOR THE NRC GENERIC FUNDAMENTALS EXAMINATION

During the administration of this examination the following rules apply:

NOTE: The term "control rod" refers to the length of neutron absorber material that can be positioned by the operator to change core reactivity.

NOTE: Numerical answers are rounded to the nearest whole number unless otherwise indicated.

1. Print your name in the blank provided on the cover sheet of the examination.
2. Fill in your individual docket number.
3. Fill in the name of your facility.
4. Fill in your start and stop times at the appropriate times.
5. Two aids are provided for your use during the examination:
 - (1) An Equations and Conversions Sheet contained within the examination copy, and
 - (2) Steam tables and Mollier Diagram provided by your proctor.
6. Place your answers on the answer sheet provided. Credit will only be given for answers properly marked on this sheet. Follow the instructions for filling out the answer sheet.
7. Scrap paper will be provided for calculations.
8. Cheating on the examination will result in the automatic forfeiture of this examination. Cheating could also result in severe penalties.
9. Restroom trips are limited. Only one examinee may leave the room at a time. In order to avoid the appearance or possibility of cheating, avoid all contact with anyone outside the examination room.
10. After you have completed the examination, sign the statement on the cover sheet indicating that the work is your own and you have neither given nor received any assistance in completing the examination. Either pencil or pen may be used.
11. Turn in your examination materials, answer sheet on top, followed by the examination copy and the examination aids, e.g., steam tables, handouts, and scrap paper.
12. After turning in your examination materials, leave the examination area as defined by the proctor. If after leaving you are found in the examination area while the examination is in progress, your examination may be forfeited.

GENERIC FUNDAMENTALS EXAMINATION
EQUATIONS AND CONVERSIONS SHEET

EQUATIONS

$$\dot{Q} = \dot{m}c_p\Delta T$$

$$A = A_0 e^{-\lambda t}$$

$$\dot{Q} = \dot{m}\Delta h$$

$$CR_{S/D} = S/(1 - K_{eff})$$

$$\dot{Q} = UA\Delta T$$

$$CR_1(1 - K_{eff_1}) = CR_2(1 - K_{eff_2})$$

$$\dot{Q} \propto \dot{m}_{Nat\ Circ}^3$$

$$1/M = CR_1/CR_x$$

$$\Delta T \propto \dot{m}_{Nat\ Circ}^2$$

$$A = \pi r^2$$

$$K_{eff} = 1/(1 - \rho)$$

$$F = PA$$

$$\rho = (K_{eff} - 1)/K_{eff}$$

$$\dot{m} = \rho A \vec{v}$$

$$SUR = 26.06/\tau$$

$$\dot{W}_{Pump} = \dot{m}\Delta P v$$

$$\tau = \frac{\bar{\beta}_{eff} - \rho}{\lambda_{eff} \rho}$$

$$P = IE$$

$$\rho = \frac{\ell^*}{\tau} + \frac{\bar{\beta}_{eff}}{1 + \lambda_{eff} \tau}$$

$$P_A = \sqrt{3}IE$$

$$P_T = \sqrt{3}IEpf$$

$$\ell^* = 1.0 \times 10^{-4} \text{ sec}$$

$$P_R = \sqrt{3}IE\sin\theta$$

$$\lambda_{eff} = 0.1 \text{ sec}^{-1} \text{ (for small positive } \rho)$$

$$\text{Thermal Efficiency} = \text{Net Work Out/Energy In}$$

$$DRW \propto \phi_{tip}^2 / \phi_{avg}^2$$

$$\frac{g(z_2 - z_1)}{g_c} + \frac{(\vec{v}_2^2 - \vec{v}_1^2)}{2g_c} + v(P_2 - P_1) + (u_2 - u_1) + (q - w) = 0$$

$$P = P_0 e^{t/\tau}$$

$$g_c = 32.2 \text{ lbm-ft/lbf-sec}^2$$

$$P = P_0 10^{SUR(t)}$$

CONVERSIONS

$$1 \text{ MW} = 3.41 \times 10^6 \text{ Btu/hr}$$

$$^{\circ}\text{C} = (5/9)(^{\circ}\text{F} - 32)$$

$$1 \text{ ft}_{\text{water}}^3 = 7.48 \text{ gal}$$

$$1 \text{ hp} = 2.54 \times 10^3 \text{ Btu/hr}$$

$$^{\circ}\text{F} = (9/5)(^{\circ}\text{C}) + 32$$

$$1 \text{ gal}_{\text{water}} = 8.35 \text{ lbm}$$

$$1 \text{ Btu} = 778 \text{ ft-lbf}$$

$$1 \text{ kg} = 2.21 \text{ lbm}$$

$$1 \text{ Curie} = 3.7 \times 10^{10} \text{ dps}$$

**USNRC GENERIC FUNDAMENTALS EXAMINATION
DECEMBER 2012 PWR--FORM A**

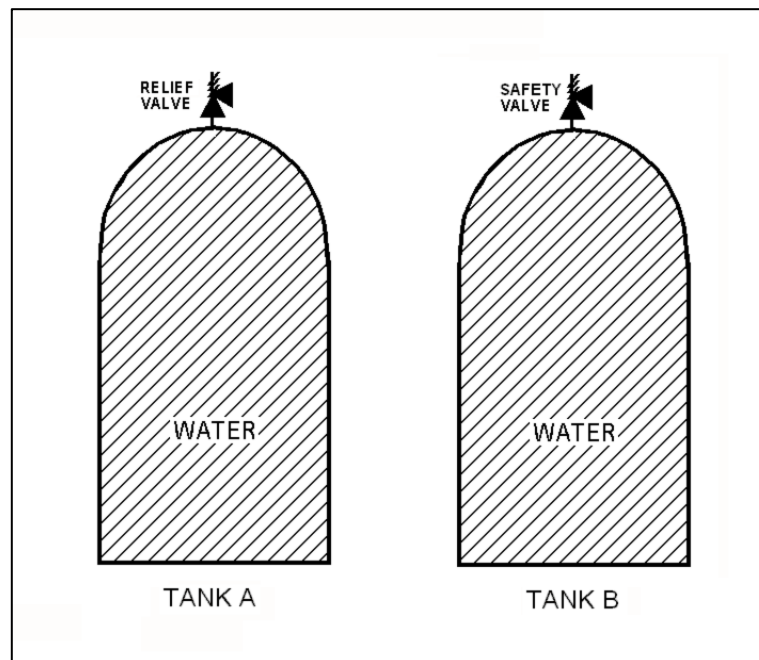
QUESTION: 1

Refer to the drawing of two identical water storage tanks (see figure below). Tank A is protected by a relief valve and Tank B is protected by a safety valve. Each valve has an opening setpoint of 205 psig and a maximum rated discharge flow rate of 8 gpm.

The tanks are being hydrostatically tested to 200 psig. Each tank is being supplied with a smooth and constant flow rate of 2 gpm from separate positive displacement pumps (PDPs). Both PDPs are inadvertently left running when tank pressures reach 200 psig.

With the PDPs running continuously, what will be the resulting status of the relief and safety valves?

- | | <u>Relief Valve Status</u> | <u>Safety Valve Status</u> |
|----|---|---|
| A. | Partially open | Partially open |
| B. | Partially open | Cycling between fully open and fully closed |
| C. | Cycling between fully open and fully closed | Partially open |
| D. | Cycling between fully open and fully closed | Cycling between fully open and fully closed |



**USNRC GENERIC FUNDAMENTALS EXAMINATION
DECEMBER 2012 PWR--FORM A**

QUESTION: 2

A stop check valve is a type of check valve that...

- A. cannot be shut remotely.
- B. can be used to prevent flow in both directions.
- C. contains both a gate valve disk and a check valve disk.
- D. can be opened manually to allow flow in both directions.

QUESTION: 3

A main steam flow rate measuring instrument uses a steam pressure input to produce main steam mass flow rate indication. Assuming steam volumetric flow rate does not change, a steam pressure decrease will cause indicated steam mass flow rate to...

- A. increase, because the density of the steam has increased.
- B. decrease, because the density of the steam has decreased.
- C. remain the same, because steam pressure does not affect the mass flow rate of steam.
- D. remain the same, because the steam pressure input compensates for changes in steam pressure.

**USNRC GENERIC FUNDAMENTALS EXAMINATION
DECEMBER 2012 PWR--FORM A**

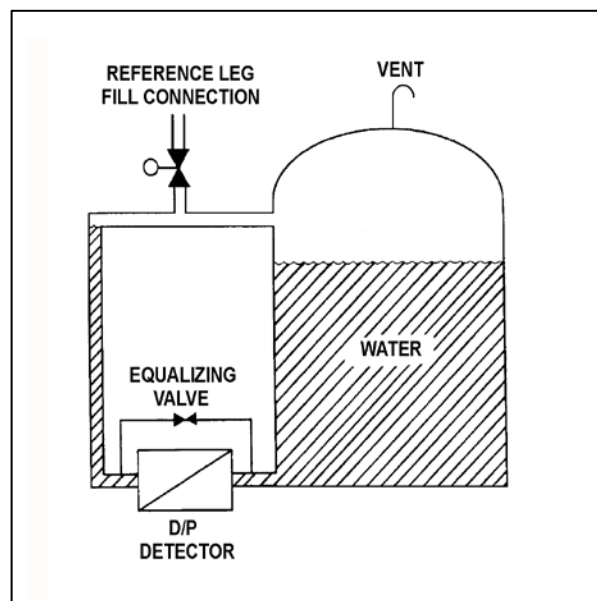
QUESTION: 4

Refer to the drawing of a vented water storage tank with a differential pressure (D/P) level detection system (see figure below). The water in the tank and reference leg is at the same temperature.

The tank level indicator was just calibrated to indicate 0 percent when the tank is empty and 100 percent when the water level reaches the upper tap. The indicator's display range is 0 percent to 120 percent. The initial water level is as indicated in the figure.

If the tank water level slowly increases and stabilizes just below the top of the tank, the level indication will increase until...

- A. the water level stabilizes, at which time the level indication will stabilize at 100 percent.
- B. the water level stabilizes, at which time the level indication will stabilize at a value greater than 100 percent.
- C. the water level reaches the upper tap, at which time the level indication will remain at 100 percent as the water level continues to increase.
- D. the water level reaches the upper tap, at which time the level indication will continue to increase as the water level continues to increase.



**USNRC GENERIC FUNDAMENTALS EXAMINATION
DECEMBER 2012 PWR--FORM A**

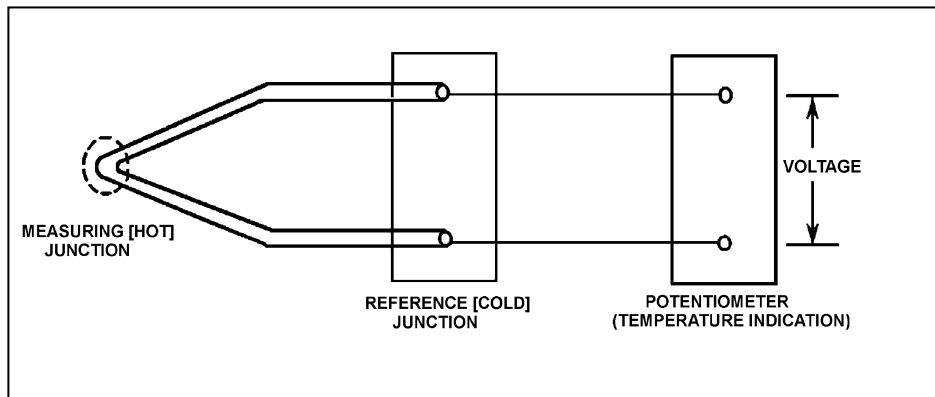
QUESTION: 5

Refer to the drawing of a simple thermocouple circuit (see figure below).

The measuring junction temperature is currently 300°F while the reference junction temperature is being held constant at 120°F . The thermocouple circuit is capable of indicating 32°F to 600°F and has just been calibrated at the current conditions.

If the measuring junction temperature decreases and stabilizes at 90°F , what temperature will be indicated?

- A. 32°F
- B. 60°F
- C. 90°F
- D. 120°F



**USNRC GENERIC FUNDAMENTALS EXAMINATION
DECEMBER 2012 PWR--FORM A**

QUESTION: 6

Which one of the following describes a characteristic of a self-reading pocket dosimeter (SRPD)?

- A. The output of an SRPD is a dose rate in mR/hr.
- B. SRPDs are primarily sensitive to beta radiation.
- C. SRPD readings must be considered inaccurate when they are dropped.
- D. SRPDs hold their charge indefinitely when removed from a radiation field.

QUESTION: 7

An air-operated isolation valve requires 3,200 pounds-force from its diaphragm actuator and 4 inches of stem travel for proper operation. The area of the actuator diaphragm is 80 square inches.

What is the approximate air pressure required for proper valve operation?

- A. 10 psig
- B. 25 psig
- C. 40 psig
- D. 55 psig

**USNRC GENERIC FUNDAMENTALS EXAMINATION
DECEMBER 2012 PWR--FORM A**

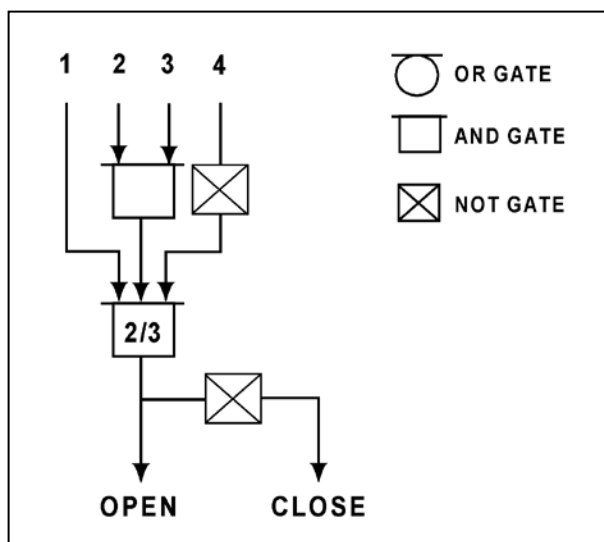
QUESTION: 8

Refer to the logic diagram for a valve controller (see figure below).

Which one of the following combinations of inputs will result in the valve receiving an OPEN signal?

INPUTS

	1	2	3	4
A.	Off	On	Off	Off
B.	Off	On	On	Off
C.	On	Off	Off	On
D.	On	Off	On	On



**USNRC GENERIC FUNDAMENTALS EXAMINATION
DECEMBER 2012 PWR--FORM A**

QUESTION: 9

A reverse-acting proportional controller will be used to maintain level in a water storage tank by positioning an air-operated makeup water flow control valve.

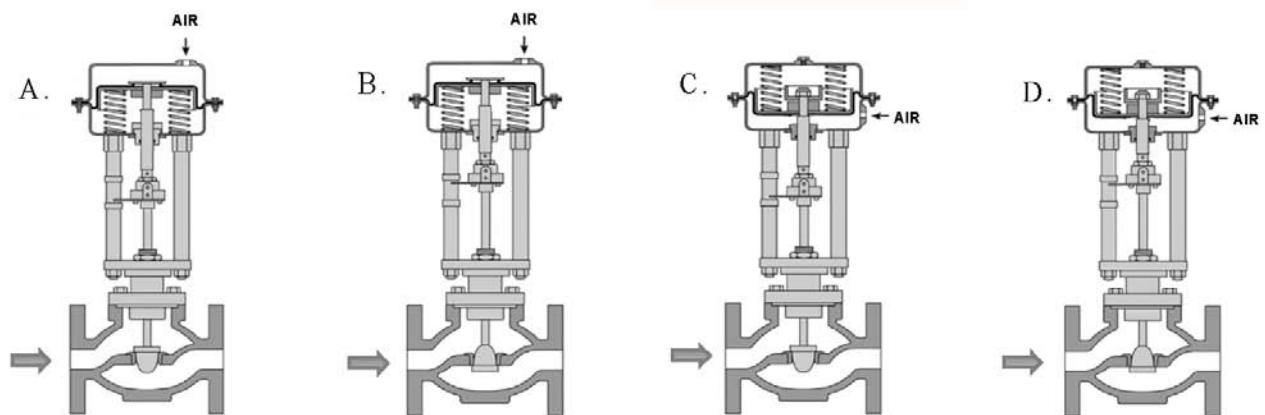
Which of the following flow control valves will be compatible with the controller in this application?

A. A and B

B. B and C

C. C and D

D. D and A



**USNRC GENERIC FUNDAMENTALS EXAMINATION
DECEMBER 2012 PWR--FORM A**

QUESTION: 10

Which one of the following is an effective method for ensuring that a centrifugal pump remains primed and does not become gas bound during operation and after shutdown?

- A. Install the pump below the level of the suction supply.
- B. Install a check valve in the discharge piping of the pump.
- C. Install an orifice plate in the discharge piping of the pump.
- D. Install a pump recirculation line from the pump discharge piping to the pump suction piping.

QUESTION: 11

An operating centrifugal pump has a net positive suction head (NPSH) requirement of 150 ft-lbf/lbm. Water at 300°F is entering the pump. Which one of the following is the lowest listed pump inlet pressure that will provide adequate NPSH for the pump?

- A. 60 psia
- B. 83 psia
- C. 108 psia
- D. 127 psia

**USNRC GENERIC FUNDAMENTALS EXAMINATION
DECEMBER 2012 PWR--FORM A**

QUESTION: 12

Refer to the drawing of an operating cooling water system (see figure below). The pump discharge valve is partially throttled to produce the following initial pump operating parameters:

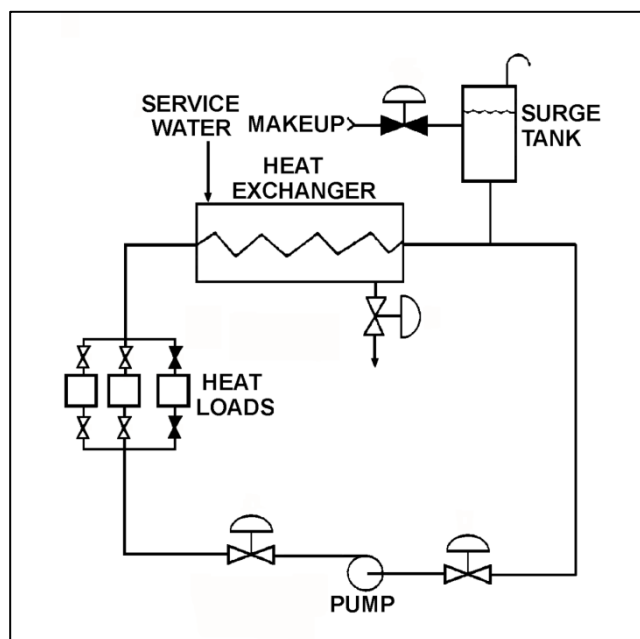
Pump discharge pressure = 45 psig
Pump suction pressure = 15 psig
Pump flow rate = 120 gpm

After a few hours of operation, the current pump operating parameters are as follows:

Pump discharge pressure = 48 psig
Pump suction pressure = 18 psig
Pump flow rate: = 120 gpm

Which one of the following could be responsible for the change in pump operating parameters?

- A. The pump speed increased with no other changes to the system.
- B. The surge tank level increased with no other changes to the system.
- C. The pump discharge valve was closed further while pump speed increased.
- D. The pump discharge valve was closed further while surge tank level increased.



**USNRC GENERIC FUNDAMENTALS EXAMINATION
DECEMBER 2012 PWR--FORM A**

QUESTION: 13

What is the purpose of the relief valve located between the pump outlet and the discharge isolation valve of many positive displacement pumps?

- A. Protect the pump and suction piping from overpressure if the discharge valve is open during system startup.
- B. Protect the pump and suction piping from overpressure if the suction valve is closed during pump operation.
- C. Protect the pump and discharge piping from overpressure if the discharge valve is closed during pump operation.
- D. Protect the pump and discharge piping from overpressure due to thermal expansion of pump contents when the pump is stopped with its suction valve closed.

QUESTION: 14

Consider two identical single-speed AC induction motors, one of which is connected to a radial-flow centrifugal pump and the other to a rotary-type positive displacement pump (PDP). Both pumps are taking suction from the bottom of a vented water storage tank.

Each pump is operating with the following initial conditions:

Flow rate = 200 gpm
Backpressure = 600 psig
Motor current = 100 amps

If the backpressure for each pump decreases to 400 psig, the centrifugal pump will have a _____ flow rate than the PDP; and the centrifugal pump will have a _____ motor current than the PDP.

- A. lower; lower
- B. lower; higher
- C. higher; lower
- D. higher; higher

**USNRC GENERIC FUNDAMENTALS EXAMINATION
DECEMBER 2012 PWR--FORM A**

QUESTION: 15

Frequent starts of large motors will result in overheating of the motor windings due to high current flow caused by...

- A. low electrical resistance of the motor windings.
- B. an electrical short circuit between the rotor and stator.
- C. high counter electromotive force at low rotor speeds.
- D. windage losses between the rotor and stator.

QUESTION: 16

The rate of heat transfer between two liquids in a single-phase heat exchanger will decrease if the...
(Assume constant specific heat capacities.)

- A. inlet temperatures of both liquids increase by 20°F.
- B. inlet temperatures of both liquids decrease by 20°F.
- C. flow rate of the hotter liquid increases by 10 percent.
- D. flow rate of the colder liquid decreases by 10 percent.

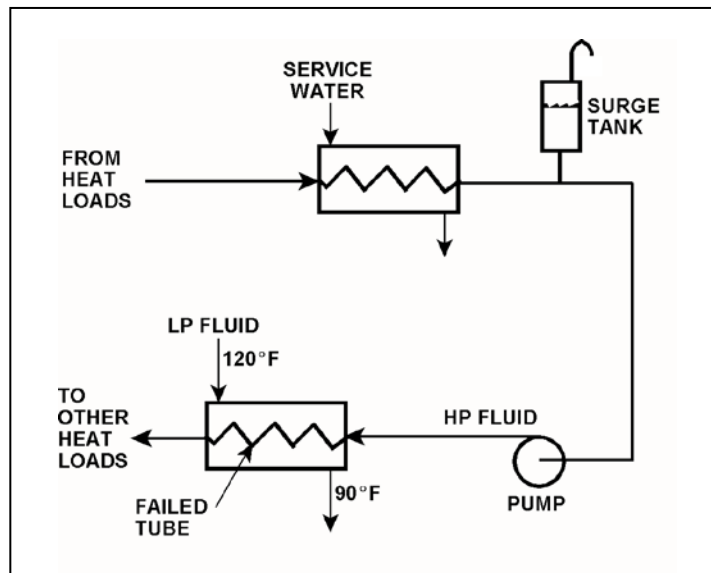
**USNRC GENERIC FUNDAMENTALS EXAMINATION
DECEMBER 2012 PWR--FORM A**

QUESTION: 17

Refer to the drawing of an operating cooling water system (see figure below).

Which one of the following effects would occur as a result of the failed tube in the heat exchanger?

- A. Level in the surge tank increases.
- B. Flow in the low pressure (LP) system reverses.
- C. Pressure in the low pressure (LP) system decreases.
- D. Low pressure (LP) fluid heat exchanger outlet temperature decreases.



**USNRC GENERIC FUNDAMENTALS EXAMINATION
DECEMBER 2012 PWR--FORM A**

QUESTION: 18

Which one of the following will cause a large pressure drop across a demineralizer that is in operation?

- A. Channeling of flow through the demineralizer.
- B. Decrease in flow rate through the demineralizer.
- C. Improper demineralizer venting after resin fill.
- D. Accumulation of suspended solids filtered by the resin beads.

QUESTION: 19

A nuclear power plant is operating at steady-state 70 percent power when the temperature of the reactor coolant letdown passing through a boron-saturated mixed bed ion exchanger increases by 20°F.

As a result, the boron concentration in the effluent of the ion exchanger will _____ because the affinity of the ion exchanger for borate ions has _____.

- A. decrease; increased
- B. decrease; decreased
- C. increase; increased
- D. increase; decreased

**USNRC GENERIC FUNDAMENTALS EXAMINATION
DECEMBER 2012 PWR--FORM A**

QUESTION: 20

Two identical 1,000 MW generators are operating in parallel supplying the same isolated electrical bus. The generator output breakers also provide identical protection for the generators. Generator A and B output indications are as follows:

<u>Generator A</u>	<u>Generator B</u>
22 KV	22 KV
60.2 Hertz	60.2 Hertz
200 MW	200 MW
25 MVAR (out)	50 MVAR (out)

A malfunction causes the voltage regulator setpoint for generator A to slowly and continuously increase. If no operator action is taken, generator B output current will...

- A. initially decrease, and then increase until the output breaker for generator A trips on overcurrent.
- B. initially decrease, and then increase until the output breaker for generator B trips on overcurrent.
- C. increase continuously until the output breaker for generator A trips on overcurrent.
- D. increase continuously until the output breaker for generator B trips on overcurrent.

**USNRC GENERIC FUNDAMENTALS EXAMINATION
DECEMBER 2012 PWR--FORM A**

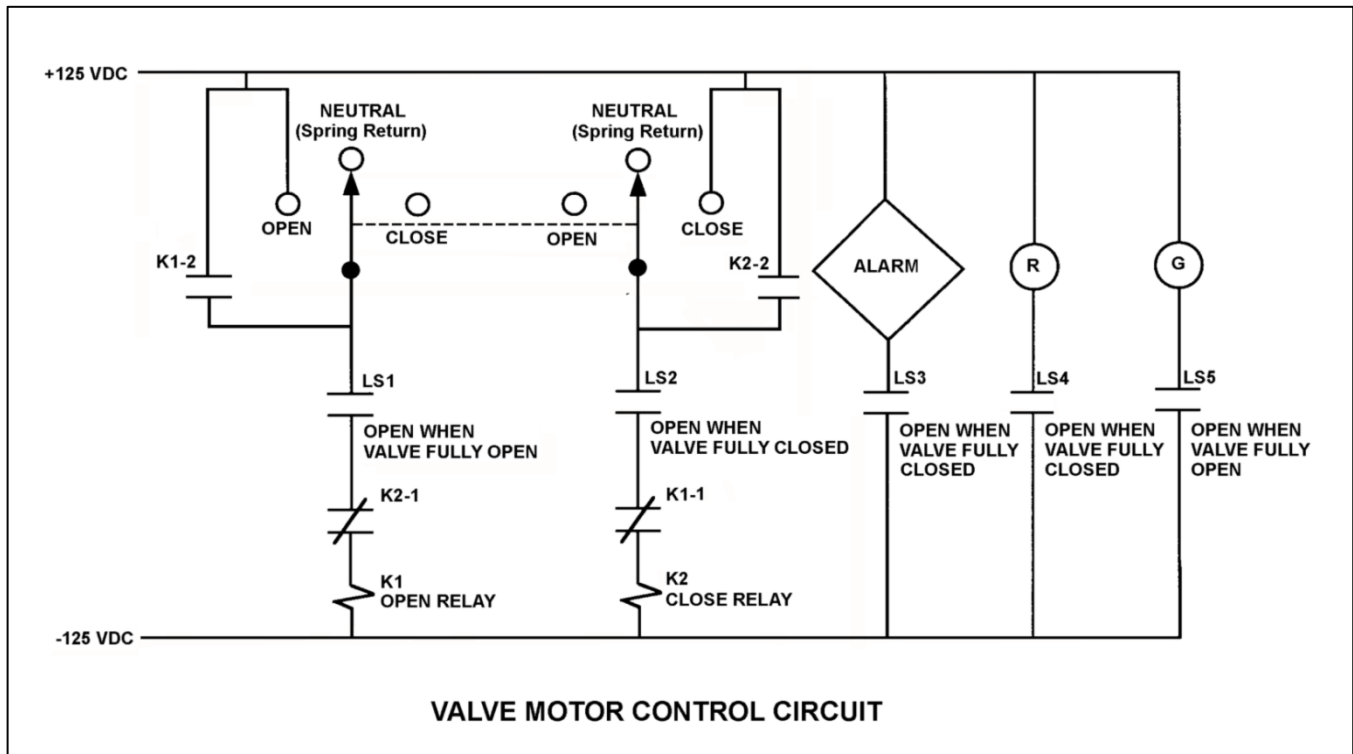
QUESTION: 21

Refer to the drawing of a valve motor control circuit (see figure below) for a valve that is currently fully open and has a 10-second stroke time.

Note: Limit switch (LS) contacts are shown open regardless of valve position, but relay contacts are shown open/closed according to the standard convention for control circuit drawings.

The operator takes the control switch to CLOSE. Two seconds later, after verifying the valve is closing, the operator releases the control switch. Which one of the following describes the valve motor control circuit alarm response after the switch is released?

- A. The alarm will continue to actuate for approximately 8 seconds.
- B. The alarm will continue to actuate until additional operator action is taken.
- C. The alarm will actuate after approximately 8 seconds.
- D. The alarm will not actuate until additional operator action is taken.



**USNRC GENERIC FUNDAMENTALS EXAMINATION
DECEMBER 2012 PWR--FORM A**

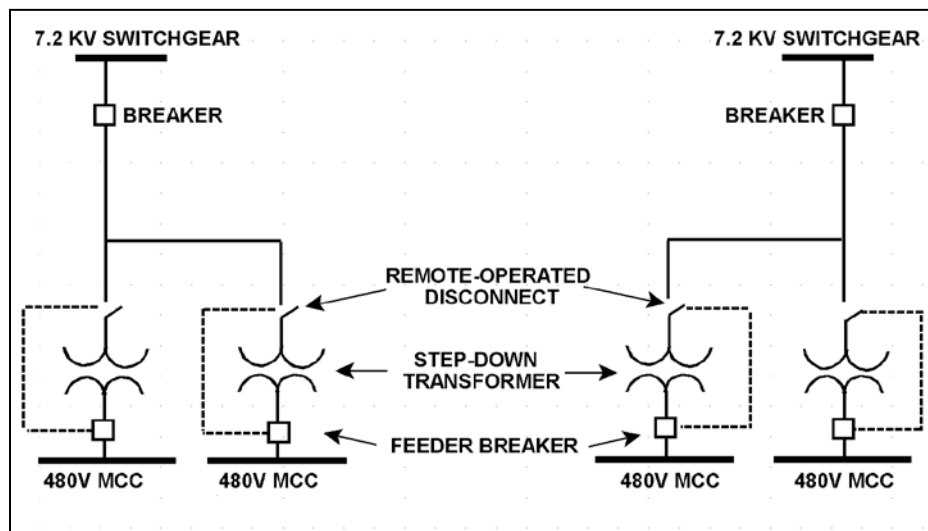
QUESTION: 22

Refer to the simplified drawing of an electrical distribution system showing 7.2 KV switchgear, step-down transformers, and 480 V motor control centers (MCCs) (see figure below).

The high voltage side of each step-down transformer has a remote-operated disconnect to allow transformer maintenance while keeping the other transformers in service. The control circuit for each disconnect is position-interlocked with the associated MCC feeder breaker.

Which one of the following describes the purpose served by the interlock?

- A. Prevent damage to the disconnect.
- B. Prevent damage to the transformer.
- C. Prevent damage to the feeder breaker.
- D. Prevent damage to the 480V MCC.



**USNRC GENERIC FUNDAMENTALS EXAMINATION
DECEMBER 2012 PWR--FORM A**

QUESTION: 23

In a comparison between a delayed neutron and a prompt neutron produced from the same fission event, the prompt neutron is more likely to... (Assume that both neutrons remain in the core.)

- A. be expelled with a lower kinetic energy.
- B. cause thermal fission of a U-235 nucleus.
- C. require a greater number of collisions to become a thermal neutron.
- D. be captured by U-238 at a resonance energy peak between 1 eV and 1000 eV.

QUESTION: 24

A 1.5 MeV neutron is about to interact with a U-238 nucleus in an operating reactor. Which one of the following describes the most likely interaction and its effect on K_{eff} ?

- A. The neutron will be scattered, thereby leaving K_{eff} unchanged.
- B. The neutron will be absorbed and the nucleus will fission, thereby decreasing K_{eff} .
- C. The neutron will be absorbed and the nucleus will fission, thereby increasing K_{eff} .
- D. The neutron will be absorbed and the nucleus will decay to Pu-239, thereby increasing K_{eff} .

**USNRC GENERIC FUNDAMENTALS EXAMINATION
DECEMBER 2012 PWR--FORM A**

QUESTION: 25

A nuclear power plant has been operating at 100 percent power for two months when a reactor trip occurs. Two months after the reactor trip, with all control rods still fully inserted, a stable count rate of 20 cps is indicated on the source range nuclear instruments.

The majority of the source range detector output is being caused by the interaction of _____ with the detector.

- A. intrinsic source neutrons
- B. fission gammas from previous power operation
- C. fission neutrons from subcritical multiplication
- D. delayed fission neutrons from previous power operation

QUESTION: 26

How does increasing the reactor coolant boron concentration affect the moderator temperature coefficient (MTC) in an overmoderated reactor?

- A. The initially negative MTC becomes more negative.
- B. The initially negative MTC becomes less negative.
- C. The initially positive MTC becomes more positive.
- D. The initially positive MTC becomes less positive.

**USNRC GENERIC FUNDAMENTALS EXAMINATION
DECEMBER 2012 PWR--FORM A**

QUESTION: 27

Given the following initial parameters:

Reactor coolant boron concentration = 600 ppm
Moderator temperature coefficient = $-0.015\% \Delta K/K/^{\circ}F$
Differential boron worth = $-0.010\% \Delta K/K/ppm$

Which one of the following is the final reactor coolant boron concentration required to decrease average reactor coolant temperature by $4^{\circ}F$. (Assume no change in control rod position or reactor/turbine power).

- A. 594 ppm
- B. 597 ppm
- C. 603 ppm
- D. 606 ppm

QUESTION: 28

Why do the control rod insertion limits generally rise as reactor power increases?

- A. Power defect increases as power increases.
- B. Control rod worth decreases as power increases.
- C. Fuel temperature coefficient decreases as power increases.
- D. Equilibrium xenon-135 negative reactivity increases as power increases.

**USNRC GENERIC FUNDAMENTALS EXAMINATION
DECEMBER 2012 PWR--FORM A**

QUESTION: 29

A reactor is operating at steady-state 75 percent power with all control rods fully withdrawn. Assuming the reactor does not trip, which one of the following compares the effects of dropping (full insertion) a center control rod to the effects of partially inserting (50 percent) the same control rod?

- A. A partially inserted rod causes a smaller change in shutdown margin.
- B. A partially inserted rod causes a greater change in shutdown margin.
- C. A partially inserted rod causes a greater change in radial power distribution.
- D. A partially inserted rod causes a greater change in axial power distribution.

QUESTION: 30

After a reactor shutdown from equilibrium xenon-135 conditions, the peak xenon-135 negative reactivity is _____ the pre-shutdown power level.

- A. independent of
- B. directly proportional to
- C. inversely proportional to
- D. dependent on, but not directly proportional to

**USNRC GENERIC FUNDAMENTALS EXAMINATION
DECEMBER 2012 PWR--FORM A**

QUESTION: 31

A nuclear power plant was initially operating at steady-state 100 percent power at the end of a fuel cycle (EOC) when the plant was shut down for refueling. After refueling, the reactor was restarted and the plant is currently operating at steady-state 100 percent power at the beginning of a fuel cycle (BOC). Assume the average energy released by each fission did not change.

Compared to the equilibrium xenon-135 concentration at 100 percent power just prior to the refueling, the current equilibrium xenon-135 concentration is...

- A. greater, because the higher fission rate at BOC produces xenon-135 at a faster rate.
- B. greater, because the lower thermal neutron flux at BOC removes xenon-135 at a slower rate.
- C. smaller, because the lower fission rate at BOC produces xenon-135 at a slower rate.
- D. smaller, because the higher thermal neutron flux at BOC removes xenon-135 at a faster rate.

QUESTION: 32

Which one of the following is not a function performed by burnable poisons in an operating reactor?

- A. Provide neutron flux shaping.
- B. Provide more uniform power density.
- C. Offset the effects of control rod burnout.
- D. Allow higher enrichment of new fuel assemblies.

**USNRC GENERIC FUNDAMENTALS EXAMINATION
DECEMBER 2012 PWR--FORM A**

QUESTION: 33

During a reactor startup, the first positive reactivity addition caused the stable source range count rate to increase from 20 cps to 30 cps. The second positive reactivity addition caused the stable count rate to increase from 30 cps to 60 cps. K_{eff} was 0.97 prior to the first reactivity addition.

Which one of the following statements accurately compares the reactivity additions?

- A. The first and second reactivity additions were approximately equal.
- B. The first reactivity addition was approximately twice as large as the second.
- C. The second reactivity addition was approximately twice as large as the first.
- D. There is not enough information given to determine the relationship of the reactivity values.

QUESTION: 34

A reactor startup is in progress. Control rod withdrawal was stopped several minutes ago to assess criticality. Which one of the following is a combination of indications that together support a declaration that the reactor has reached criticality?

- A. Startup rate is stable at 0.0 dpm; source range count rate is stable.
- B. Startup rate is stable at 0.2 dpm; source range count rate is stable.
- C. Startup rate is stable at 0.0 dpm; source range count rate is slowly increasing.
- D. Startup rate is stable at 0.2 dpm; source range count rate is slowly increasing.

**USNRC GENERIC FUNDAMENTALS EXAMINATION
DECEMBER 2012 PWR--FORM A**

QUESTION: 35

A refueling outage has just been completed in which one-third of the core was replaced with new fuel assemblies. A reactor startup has been performed to begin the sixth fuel cycle, and reactor power is being increased to 100 percent.

Which one of the following pairs of reactor fuels will provide the greatest contribution to core heat production when the reactor reaches 100 percent power?

- A. U-235 and U-238
- B. U-238 and Pu-239
- C. U-235 and Pu-239
- D. U-235 and Pu-241

QUESTION: 36

A nuclear power plant has been operating for one hour at 50 percent power following six months of operation at steady-state 100 percent power. What percentage of rated thermal power is currently being generated by fission product decay?

- A. 1 percent to 2 percent
- B. 3 percent to 5 percent
- C. 6 percent to 8 percent
- D. 9 percent to 11 percent

**USNRC GENERIC FUNDAMENTALS EXAMINATION
DECEMBER 2012 PWR--FORM A**

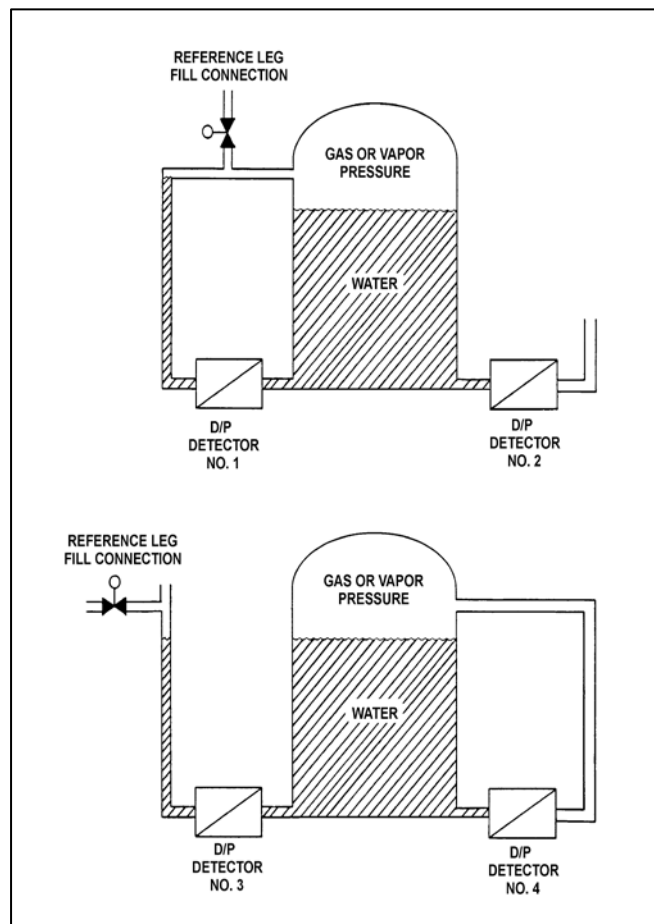
QUESTION: 37

Refer to the drawing of two water storage tanks with four differential pressure (D/P) level detectors (see figure below).

The tanks are identical with equal water levels and 20 psia gas pressure above the water. The tanks are surrounded by standard atmospheric pressure. The temperature of the water in the tanks and reference legs is 70°F.

If each detector experiences a ruptured diaphragm, which detector(s) will produce a lower level indication? (Assume that actual tank and reference leg water levels do not change.)

- A. No. 1 only
- B. No. 2 only
- C. No. 1, 2, and 3
- D. No. 2, 3, and 4



**USNRC GENERIC FUNDAMENTALS EXAMINATION
DECEMBER 2012 PWR--FORM A**

QUESTION: 38

Given the following:

- A saturated steam-water mixture with an inlet quality of 60 percent is flowing through a moisture separator.
- The moisture separator is 100 percent efficient for removing moisture.

How much moisture will be removed by the moisture separator from 50 lbm of the steam-water mixture?

- A. 10 lbm
- B. 20 lbm
- C. 30 lbm
- D. 40 lbm

QUESTION: 39

A nuclear power plant experienced a reactor trip. One hour after the trip, core cooling is being accomplished by relieving saturated steam from a steam generator (SG). Water level in the SG is being maintained by an operating feedwater pump. Average fuel temperature is stable.

Given the following current conditions:

Core decay heat rate = 33 MW
SG pressure = 1,000 psia
Feedwater temperature = 90°F

For the above conditions, approximately what feedwater flow rate is needed to maintain a constant mass of water in the SG?

- A. 100,000 lbm/hr
- B. 125,000 lbm/hr
- C. 170,000 lbm/hr
- D. 215,000 lbm/hr

**USNRC GENERIC FUNDAMENTALS EXAMINATION
DECEMBER 2012 PWR--FORM A**

QUESTION: 40

A nuclear power plant is operating at 100 percent power with steam generator pressure at 900 psia. A steam generator safety valve is leaking 100 percent saturated steam to atmosphere.

Which one of the following is the approximate temperature of the escaping steam once it reaches standard atmospheric pressure?

- A. 532°F
- B. 370°F
- C. 308°F
- D. 212°F

QUESTION: 41

The theoretical maximum efficiency of a steam cycle is given by the equation:

$$\text{Eff}_{\text{max}} = (1 - T_{\text{out}}/T_{\text{in}}) \times 100\%$$

where T_{out} is the absolute temperature for heat rejection and T_{in} is the absolute temperature for heat addition. (Fahrenheit temperature is converted to absolute temperature by adding 460°F.)

A nuclear power plant is operating with a stable steam generator pressure of 900 psia. What is the approximate theoretical maximum steam cycle efficiency this plant can achieve by establishing its main condenser vacuum at 1.0 psia?

- A. 35 percent
- B. 43 percent
- C. 65 percent
- D. 81 percent

**USNRC GENERIC FUNDAMENTALS EXAMINATION
DECEMBER 2012 PWR--FORM A**

QUESTION: 42

The primary reason for slowly opening the discharge valve of a large motor-driven centrifugal cooling water pump after starting the pump is to minimize the...

- A. net positive suction head requirements.
- B. potential for a water hammer.
- C. motor running current requirements.
- D. potential for pump cavitation.

QUESTION: 43

A four-loop PWR nuclear power plant uses four identical single-speed reactor coolant pumps (RCPs) to supply reactor coolant flow through the reactor vessel. The plant is currently shut down with one RCP in operation.

Which one of the following describes the stable reactor coolant flow rate through the reactor vessel following the start of a second RCP?

- A. Less than twice the original flow rate.
- B. Exactly twice the original flow rate.
- C. More than twice the original flow rate.
- D. Cannot be determined without additional information.

**USNRC GENERIC FUNDAMENTALS EXAMINATION
DECEMBER 2012 PWR--FORM A**

QUESTION: 44

Why is bulk boiling in the tubes of a single-phase heat exchanger undesirable?

- A. The bubble formation will break up the laminar layer in the heat exchanger tubes.
- B. The thermal conductivity of the heat exchanger tubes will decrease.
- C. The differential temperature across the tubes will decrease through the heat exchanger.
- D. The turbulence will restrict fluid flow through the heat exchanger tubes.

QUESTION: 45

A nuclear power plant is operating at 100 percent power. The reactor coolant subcooling margin will be directly reduced by...

- A. increasing reactor coolant temperature.
- B. increasing pressurizer pressure.
- C. increasing reactor coolant flow rate.
- D. increasing pressurizer level.

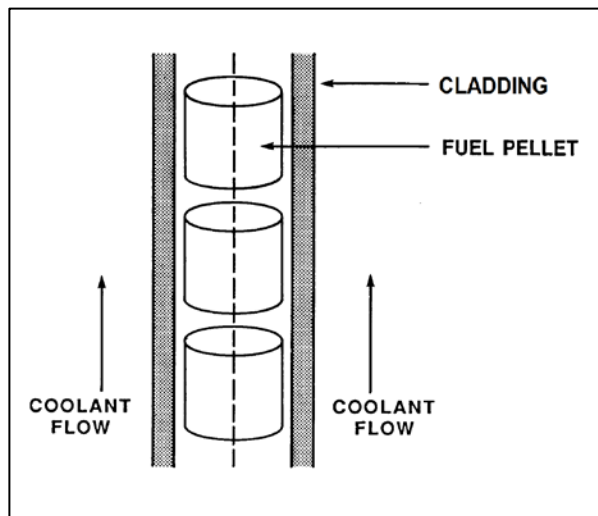
**USNRC GENERIC FUNDAMENTALS EXAMINATION
DECEMBER 2012 PWR--FORM A**

QUESTION: 46

Refer to the drawing of a fuel rod and adjacent coolant flow channel (see figure below).

With a nuclear power plant operating at steady-state 100 percent reactor power at the beginning of a fuel cycle, which one of the following has the greater temperature difference?

- A. Fuel pellet centerline-to-pellet surface
- B. Fuel pellet surface-to-cladding gap
- C. Zircaloy cladding
- D. Coolant laminar layer



**USNRC GENERIC FUNDAMENTALS EXAMINATION
DECEMBER 2012 PWR--FORM A**

QUESTION: 47

Sustained natural circulation requires that the heat source is _____ in elevation than the heat sink; and that there is a _____ difference between the heat source and the heat sink.

- A. lower; phase
- B. lower; temperature
- C. higher; phase
- D. higher; temperature

QUESTION: 48

A nuclear power plant was operating at steady-state 100 percent power when a loss of offsite power occurred, resulting in a reactor trip and a loss of forced reactor coolant circulation. Two hours later, reactor coolant system (RCS) hot leg temperature is greater than cold leg temperature and steam generator (SG) levels are stable.

Which one of the following combinations of parameter trends, observed two hours after the trip, indicates that natural circulation is not occurring? (CET = core exit thermocouples)

	<u>RCS Hot Leg Temperature</u>	<u>RCS Cold Leg Temperature</u>	<u>SG Pressures</u>	<u>RCS CET Subcooling</u>
A.	Stable	Decreasing	Decreasing	Stable
B.	Stable	Stable	Decreasing	Decreasing
C.	Decreasing	Decreasing	Decreasing	Increasing
D.	Decreasing	Stable	Stable	Increasing

**USNRC GENERIC FUNDAMENTALS EXAMINATION
DECEMBER 2012 PWR--FORM A**

QUESTION: 49

Maximum fuel cladding integrity is maintained by...

- A. always operating below 110 percent of reactor coolant system design pressure.
- B. actuation of the reactor protection system upon a reactor accident.
- C. ensuring that actual heat flux is always less than critical heat flux.
- D. ensuring operation above the critical heat flux during all operating conditions.

QUESTION: 50

Which one of the following will prevent brittle fracture failure of a reactor vessel?

- A. Manufacturing the reactor vessel from low carbon steel.
- B. Maintaining reactor vessel pressure below the maximum design limit.
- C. Operating above the nil-ductility transition temperature.
- D. Maintaining the number of reactor vessel heatup/cooldown cycles within limits.

***** FINAL ANSWER KEY *****

**DECEMBER 2012 NRC GENERIC FUNDAMENTALS EXAMINATION
PRESSURIZED WATER REACTOR - ANSWER KEY**

<u>FORM A</u>	<u>FORM B</u>	<u>ANS.</u>	<u>FORM A</u>	<u>FORM B</u>	<u>ANS.</u>
1	15	B	26	40	C
2	16	B	27	41	D
3	17	B	28	42	A
4	18	C	29	43	D
5	19	C	30	44	D
6	20	C	31	45	B
7	21	C	32	46	C
8	22	B	33	47	A
9	23	B	34	48	D
10	24	A	35	49	C
11	25	D	36	50	B
12	26	B	37	1	D
13	27	C	38	2	B
14	28	D	39	3	A
15	29	A	40	4	C
16	30	D	41	5	B
17	31	D	42	6	B
18	32	D	43	7	A
19	33	D	44	8	D
20	34	A	45	9	A
21	35	A	46	10	A
22	36	A	47	11	B
23	37	C	48	12	B
24	38	A	49	13	C
25	39	C	50	14	C