



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**
WASHINGTON, D.C. 20555-0001

June 28, 2016

Mr. Chris Terry
Manager, Licensing & Safety Analysis
BWXT NOG, Inc.
P.O. Box 785
Lynchburg, VA 24505-0785

SUBJECT: DATES FOR BWXT RESPONSES TO REQUEST FOR SUPPLEMENTAL
INFORMATION CONCERNING RESPONSE TO GENERIC LETTER 2015-01

Dear Mr. Terry:

On April 19th and 20th, 2016, the U.S. Nuclear Regulatory Commission staff conducted a site visit to discuss the response to the generic letter. At the exit meeting it was agreed that the following request for supplemental information (RSI) is to be submitted:

- (1) Summary of maximum deflections of structural elements in the updated Master Engineers and Designers (MEAD) report.
- (2) Summary of the analysis used as a basis to determine that lateral force resisting system connections will perform with adequate strength and ductility during the design ground motion in the updated MEAD report.
- (3) Summary of the location and values of critical stresses that were evaluated in the analysis in the MEAD report.
- (4) Clarify whether or not there were deficiencies in meeting the life safety performance level. Indicate whether deficiencies were in meeting the immediate occupancy performance level or life safety performance level.
- (5) Summary of the geotechnical report used as the basis to classify the soil as Site Class C for the seismic analysis.
- (6) Summary of any walk-downs conducted as part of the ASCE 31-03 seismic evaluation.
- (7) Clarify how the loads induced by overhead cranes were incorporated in the seismic evaluation.
- (8) Confirm that the seismic analysis used a risk category of IV per ASCE 31-03. Some sample calculations specified a different risk category.
- (9) Confirm an importance factor of 1.5 per ASCE 31-03.
- (10) Analysis to evaluate the impacts from high wind events for the facility.
- (11) Updated criticality likelihood calculation in TR-0002:
 - a. Justification of the initiating event frequency for an earthquake capable of causing facility damage that can lead to a criticality. Demonstrate on a process-by-process basis that the optimum spacing and optimum moderations factors used to show compliance with Title 10 of the *Code of Federal Regulations* (10 CFR) 70.61 are appropriate.
 - b. Demonstrate compliance with the double contingency principle for natural phenomena event sequences.

- c. If applicable, provide analyses that potential failures of internal equipment in process areas do not lead to criticality or “consequences of concern” (“bounding scenarios”).
- (12) Updated information that demonstrates that natural phenomena-initiated events (or accident sequences) comply with the likelihood-consequence performance requirements of 10 CFR 70.61. Identify the information sources used to estimate the likelihood of the initiating natural phenomena event, discuss the basis for any conditional probabilities estimated for event development following the initial natural phenomena event (e.g., equipment failure, component rearrangement), and discuss the basis for categorization of event sequences (e.g., acute chemical exposure) where there are consequences.

This information is necessary to verify compliance with 10 CFR 70.62(c)(1) which requires, in part, that each licensee shall conduct and maintain an Integrated Safety Analysis that is of appropriate detail for the complexity of the process that identifies, among other things, “potential accident sequences caused by process deviations or other events internal to the facility and credible external events, including natural phenomena.” The regulations in 10 CFR 70.62 (c)(1) also require, in part, identification of consequences and the likelihood of occurrence of each potential accident sequence, and the methods used to determine the consequences and likelihoods.

On June 6, 2016 we agreed on the following RSI submittal dates:

- RSI 1, 2, 3, 10, 11 and 12 August 19, 2016 and,
- RSI 4, 5, 6, 7, 8 and 9 July 15, 2016

In accordance with 10 CFR 2.390 of the NRC’s “Agency Rules of Practice and Procedure,” a copy of this letter will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records component of NRC’s Agencywide Documents Access and Management System (ADAMS). ADAMS is accessible from the NRC Web site at: <http://www.nrc.gov/reading-rm/adams.html>.

C. Terry

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If you have any questions, feel free to contact me at (301) 415-7119, or via e-mail to Merritt.Baker@nrc.gov.

Sincerely,

/RA/

Merritt N. Baker, Senior Project Manager
Fuel Manufacturing Branch
Division of Fuel Cycle Safety, Safeguards,
and Environmental Review
Office of Nuclear Material Safety
and Safeguards

Docket No.: 70-00027
License No.: SNM-42

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NAME	MBaker	JMarcano	DMiller	RJohnson	MBaker
DATE	6/16/16	6/24/16	6/20/16	6/24/16	6/28/16

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