



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

June 21, 2016

Mr. Scott D. Northard  
Acting Site Vice President  
Prairie Island Nuclear Generating Plant  
Northern States Power Company - Minnesota  
1717 Wakonade Drive East  
Welch, MN 55089

SUBJECT: PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNITS 1 AND 2 –  
CORRECTION LETTER FOR AMENDMENTS 185 AND 175 TO RENEWED  
FACILITY OPERATING LICENSES (CAC NOS. MD5983 AND MD5984)

Dear Mr. Davison:

On June 27, 2008, the U.S. Nuclear Regulatory Commission (NRC or the Commission) issued Amendment No. 185 to Renewed Facility Operating License No. DPR-42 and Amendment No. 175 to Renewed Facility Operating License No. DPR-60 for the Prairie Island Nuclear Generating Plant, Units 1 and 2, respectively (Agencywide Documents Access and Management System (ADAMS) Accession No. ML081650272). The amendments consisted of changes to the technical specification (TSs) regarding incorporation of Technical Specification Task Force (TSTF) industry travelers TSTF-479, -485, and -497.

The amendments revised the wording for TS 5.5.7, "Inservice Testing Program," on page 5.0-12. The amendments did not revise any wording for the TSs on page 5.0-11; however, in the course of issuing the revised TS pages, the word "year" was inadvertently removed from the last sentence in TS 5.5.6, "Reactor Coolant Pump Flywheel Inspection Program," on page 5.0-11: "...of exposed surfaces of the removed flywheels may be conducted at 20 year intervals."

This letter corrects Amendment Nos. 185 and 175 to restore TS 5.5.6 to the wording as approved in Amendment Nos. 170 and 160 when the inspection interval for TS 5.5.6, "Reactor Coolant Pump Flywheel Inspection Program," was increased from 10 years to 20 years (ADAMS Accession No. ML051510045).

The NRC staff concludes that the error was introduced during the preparation of the license amendments and is entirely editorial in nature. The proposed correction does not change any of the conclusions in the safety evaluation associated with the amendments referenced above and does not affect the associated notices to the public. Enclosed, please find the corrected replacement page.

K. Davison

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If you have any questions regarding this matter, please call me at (301) 415-3733.

Sincerely,

A handwritten signature in black ink, appearing to be 'R. Kuntz', written over a horizontal line.

Robert F. Kuntz, Senior Project Manager  
Plant Licensing Branch III-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket Nos. 50-282 and 50-306

Enclosure:

Corrected Page to Amendment No. 185 to DPR-42  
and Amendment No. 175 to DPR-60

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**ENCLOSURE**

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNITS 1 AND 2

DOCKET NOS. 50-282 AND 50-306

CORRECTED PAGE TO LICENSE AMENDMENT NOS. 185 AND 175

RENEWED FACILITY OPERATING LICENSE NO. DPR-42

RENEWED FACILITY OPERATING LICENSE NO. DPR-60

5.5 Programs and Manuals

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5.5.4 Radioactive Effluent Controls Program (continued)

- j. Limitations on the annual dose or dose commitment to any member of the public, beyond the site boundary, due to releases of radioactivity and to radiation from uranium fuel cycle sources, conforming to 40 CFR 190.

The provisions of SR 3.0.2 and SR 3.0.3 are applicable to the Radioactive Effluent Controls Program surveillance frequency.

5.5.5 Component Cyclic or Transient Limit

This program provides controls to track the USAR, Section 4.1.4, cyclic and transient occurrences to ensure that components are maintained within the design limits.

5.5.6 Reactor Coolant Pump Flywheel Inspection Program

This program shall provide for the inspection of each reactor coolant pump flywheel per the recommendations of Regulatory Position C.4.b of Regulatory Guide 1.14, Revision 1, August 1975. In lieu of Position C.4.b(1) and C.4.b(2), a qualified in-place UT examination over the volume from the inner bore of the flywheel to the circle one-half of the outer radius or a surface examination (MT or PT) of exposed surfaces of the removed flywheels may be conducted at 20 year intervals.

5.5.7 Inservice Testing Program

This program provides controls for inservice testing of ASME Code Class 1, 2, and 3 components. The program shall include the following:

K. Davison

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If you have any questions regarding this matter, please call me at (301) 415-3733.

Sincerely,

**/RA/**

Robert F. Kuntz, Senior Project Manager  
Plant Licensing Branch III-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

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**ADAMS Accession No.: ML16148A179**

OFFICE	DORL/LPL3-1/PM	DORL/LPL3-1/LA	DORL/LPL3-1/BC	DORL/LPL3-1/PM
NAME	RKuntz	SRohrer	DWrona	RKuntz
DATE	5/31/16	5/31/16	6/21/16	6/21/16

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