

# REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR:7904160171 DOC.DATE: 79/04/09 NOTARIZED: NO  
 FACIL:50-287 OCONEE NUCLEAR STATION, UNIT 3, DUKE POWER CO.  
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 REGION 2, ATLANTA, OFFICE OF THE DIRECTOR

DOCKET #  
 05000287

SUBJECT: LER 79-007/03L-0 ON 790310:CONTAINMENT ISOLATION VALVE  
 3FDW-104 ON SHELL DRAIN LINE FAILED TO CLOSE.CAUSED BY VALVE  
 OPERATOR TORQUE SWITCH WHICH WAS SET TOO LOW.SETTING  
 INCREASED & VALVE RETESTED & DECLARED OPERABLE.

DISTRIBUTION CODE: A002S COPIES RECEIVED:LTR L ENCL L SIZE: 1+3  
 TITLE: INCIDENT REPORTS

NOTES: M CUNNINGHAM - ALL AMOIS TO FSAR + CHANGES TO TECH SPECS.

ACTION:	RECIPIENT	COPIES		RECIPIENT	COPIES	
	ID CODE/NAME	LTR	ENCL		ID CODE/NAME	LTR
ACTION:	05 BC ORB 4	4	4			
INTERNAL:	01 REG FILE	1	1	02 NRC PDR	1	1
	09 I&E	2	2	11 MPA	3	3
	14 TA/EDO	1	1	15 NOVAK/KNIEL	1	1
	16 EEB	1	1	17 AD FOR ENGR	1	1
	18 PLANT SYS BR	1	1	19 I&C SYS BR	1	1
	20 AD PLANT SYS	1	1	21 AD SYS/PROJ	1	1
	22 REAC SAFT BR	1	1	23 ENGR BR	1	1
	24 KREGER	1	1	25 PWR SYS BR	1	1
	26 AD/SITE ANAL	1	1	27 OPERA LIC BR	1	1
	28 ACCIDENT ANALYS	1	1	29 AUX SYS BR	1	1
	E JORDAN/IE	1	1			
EXTERNAL:	03 LPDR	1	1	04 NSIC	1	1
	29 ACRS	16	16			

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DUKE POWER COMPANY  
OCONEE UNIT 3

Report Number: RO-287/79-7

Report Date: April 9, 1979

Occurrence Date: March 10, 1979

Facility: Oconee Unit 3, Seneca, South Carolina

Identification of Occurrence: Containment Isolation Valve Inoperable in the Open Position

Conditions Prior to Occurrence: Unit Heatup: 2150 PSIG  
532°F

Description of Occurrence:

At 2232 on March 10, 1979, Unit 3 was returning to power when valve 3FDW-104, the motor operated containment isolation valve which is located outside the Reactor Building (RB) on the steam generator 3B shell drain line, was opened in order to allow blowdown of that steam generator. The valve opened to an intermediate position. Subsequently, when an attempt was made to reclose the valve, it failed to close. Oconee Nuclear Station Technical Specification 3.6.1 requires that containment integrity be maintained whenever reactor coolant system temperature and pressure exceed 200°F and 300 PSIG, respectively, but continued operation was permitted according to Technical Specification 3.6.3.b(3), which requires that the affected penetration be isolated within four hours by closing a manual valve. At 0048 on March 11, 1979, valve 3FDW-104 was isolated by closing valve 3FDW-335, the manual isolation valve located inside the RB on the steam generator 3B shell drain line. These valves are normally closed during power operation, and are only opened to drain steam generator 3B while the unit is shut down. Therefore, although the failure of valve 3FDW-104 to close constituted operation of the unit in a degraded mode allowed by a limiting condition for operation, this incident was of no significance with respect to safe operation of the unit.

Apparent Cause of Occurrence:

The valve appears to have failed to close because the valve operator torque switch was set too low. When the torque switch setting was increased from 2.5 to 3.0, the valve operated properly.

Analysis of Occurrence:

Valve 3FDW-104 is the motor operated containment isolation valve located outside the RB on the steam generator 3B shell drain line. The failure of this valve to close did not violate containment integrity, since the redundant manual valve inside the RB was operable and was closed well within the time required by Technical Specifications. Therefore, safe operation of the unit was not affected and the health and safety of the public were not endangered.

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Corrective Action:

Isolation of the steam generator 3B shell drain line was accomplished by closing the redundant manual isolation valve approximately two hours after valve 3FDW-104 was declared inoperable. Subsequent investigation of the valve resulted in the valve operator torque switch setting being increased from 2.5 to 3.0. The valve was retested and found to be operating properly. When the valve is tested during the next performance test, it will be observed closely for any binding or other problems, and should any problem be noted at that time, appropriate action will be taken.

**LICENSEE EVENT REPORT**

EXHIBIT A

CONTROL BLOCK: 

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(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32

0 1 S C N E E 3 2 0 0 - 0 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5

LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT 5

CONT

[illegible]

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

On March 10, 1979 containment isolation valve 3FDW-104 on the steam generator 3B shell drain line failed to close subsequent to steam generator blowdown during unit startup. Redundant manual valve 3FDW-335 was closed, accomplishing containment isolation well within Technical Specification limits. Therefore, although the failure of valve 3FDW-104 constituted operation in a degraded mode allowed by a limiting condition for operation, safe operation of the unit was not affected, and public health and safety were not endangered.

SYSTEM CODE H I 11		CAUSE CODE E 12		CAUSE SUBCODE B 13		COMP. SUBCODE L 15		VALVE SUBCODE O 16	
LEAK/REPORT NUMBER 7 8		EVENT YEAR 7 9		SEQUENTIAL REPORT NO. 0 0 7		OCCURRENCE CODE 0 3		REPORT TYPE L	
ACTION TAKEN E 18		FUTURE ACTION X 19		EFFECT ON PLANT Z 20		SHUTDOWN METHOD Z 21		HOURS 0 0 0 0 0 0	
ATTACHMENT SUBMITTED Y 23		NPRO-4 FORM SUB. Y 24		PRIME COMP. SUPPLIER L 25		REVISION NO. 0		COMPONENT MANUFACTURER L 26	

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 The valve appears to have failed to close because the valve operator torque

11 switch was set too low. Containment isolation was achieved by closing the

12 redundant manual isolation valve. The torque switch setting for valve 3FDW-104

13 was increased from 2.5 to 3.0, and the valve was retested and declared oper-

14 able. The valve will be examined again at the next performance test.

FACILITY STATUS		3 POWER		OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION				
1	5	C	28	0	0	0	29	NA	A	31	Operator Observation	32
ACTIVITY CONTENT		RELEASED OF RELEASE		AMOUNT OF ACTIVITY		LOCATION OF RELEASE						
1	6	Z	33	Z	34	NA	35					
PERSONNEL EXPOSURES		NUMBER		TYPE		DESCRIPTION						
1	7	0	0	0	37	Z	38	NA				
PERSONNEL INJURIES		NUMBER		DESCRIPTION								
1	8	0	0	0	40			NA				
LOSS OF OR DAMAGE TO FACILITY		TYPE		DESCRIPTION								
1	9	Z	42			NA						
PUBLICITY		ISSUED		DESCRIPTION								
2	0	N	44			NA						
								NRC USE ONLY				

NAME OF PREPARER S. R. Lewis

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