REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

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FACIL:50-287 OCONEE #3, DUKE POWER CO. 05000287
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**REG.2,ATLANTA,OFF.OF THE DIRECTOR

SUBJECT: LER 78-019/03L-0 on 781207:both trains of PR ventilation sys were inoperable due to excess water in filters caused by leakage from 3FDW-25.

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TITLE: INCIDENT REPORTS

NOTES:

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ROLL-4

DUKE POWER COMPANY
OCONEE UNIT 3

Report Number: RO-287/78-19

Report Date: January 9, 1979

Occurrence Date: December 7, 1978

Facility: Oconee Unit 3, Seneca, South Carolina

Identification of Occurrence: Both Purge Room Ventilation System Trains Inoperable

Conditions Prior to Occurrence: 98% Full Power

Description of Occurrence:

At 0618 on December 7, 1978, it was determined by observation of flow indications, that both trains of the Reactor Building (RB) Ventilation System were inoperable. The trains registered flow indications of 0 and 150 cfm as compared to a nominal flow rate of 1000 cfm. The inoperability was apparently due to saturation of the filters with water from one or more valve leaks in the penetration room. The filters were replaced and a reactor shutdown was initiated at 1500 in order to meet the requirements of Oconee Nuclear Station Technical Specification 3.15. The operability of the A train was verified by 2300.

Apparent Cause of Occurrence:

The filters apparently failed due to moisture saturation caused by steam leakage from 3FDW-251. The valve leaked steam into the penetration room for approximately 3 days prior to being temporarily repaired during a unit outage on November 3, 1978. Both fans were run on November 1, 1978 for approximately 2.61 hours during which time the filters were apparently saturated with moisture.

Analysis of Occurrence:

For the period of time the penetration room ventilation system was inoperable, the filters could not have performed their required safety function.

In the event of a postulated loss of coolant accident, and without this system operable, however the offsite doses would be considerably less than 10CFR100.

It is considered that this incident did not adversely affect the safety of the public.

Corrective Action:

The filters were replaced and verified to be operable as required by Technical Specifications. The periodic test has been revised to assure greater reliability of the system by including verification of adequate flow (1000 cfm $\pm$ 10%). An investigation into the humidity levels which could cause such failure(s) has been initiated.

NRC FORM 366 (7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

EXHIBIT A

CONTROL BLOCK: [| | | | | |] (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

Line	Field	Value
01	LICENSEE CODE (2)	S C N E E 3
01	LICENSE NUMBER (3)	0 0 - 0 0 0 0 0 - 0 0
01	LICENSE TYPE (4)	4 1 1 1 1
01	CAT (5)	
01 CON'T	REPORT SOURCE (6)	L
01	DOCKET NUMBER (7)	0 5 0 0 0 2 8 7
01	EVENT DATE (8)	1 2 0 7 7 8
01	REPORT DATE (9)	0 1 0 9 7 8

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 At 0618 on December 7, 1978 it was observed that both trains of the PR
03 ventilation system were inoperable due to excess water in the filters. The
04 fulters were replaced and a unit shutdown was initiated at 1500. Train A
05 was verified to be operable at 2300. With the PR Vent System inoperable
06 10CFR 100 limits would not be approached even during LOCA conditions. Thus
07 it is considered that public safety was not adversely affected.
08

Line 09	Field	Value
	SYSTEM CODE (11)	M B
	CAUSE CODE (12)	X
	CAUSE SUBCODE (13)	Z
	COMPONENT CODE (14)	F I L T E R
	COMP. SUBCODE (15)	Z
	VALVE SUBCODE (16)	Z

(17) LER/RO REPORT NUMBER

EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
7 8	—	0 1 9	/	0 3	L	—	0

ACTION TAKEN (18)	FUTURE ACTION (19)	EFFECT ON PLANT (20)	SHUTDOWN METHOD (21)	HOURS (22)	ATTACHMENT SUBMITTED (23)	NPRD-4 FORM SUB. (24)	PRIME COMP. SUPPLIER (25)	COMPONENT MANUFACTURER (26)
B	G	A	A	0 0 0 8	Y	N	L	Z 9 9 9

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 The excess water in the filters was apparently caused by leakage from 3FDW-251.
11 The leak was repaired on November 3, 1978. The filters have been replaced
12 and procedures have been revised in order to assure greater reliability of
13 the system.
14

Line	Field	Value
15	FACILITY STATUS (28)	E
15	% POWER (29)	0 9 8
15	OTHER STATUS (30)	NA
15	METHOD OF DISCOVERY (31)	A
15	DISCOVERY DESCRIPTION (32)	Operator Observation
16	ACTIVITY RELEASED (33)	Z
16	CONTENT OF RELEASE (34)	Z
16	AMOUNT OF ACTIVITY (35)	NA
16	LOCATION OF RELEASE (36)	NA
17	PERSONNEL EXPOSURES NUMBER (37)	0 0 0
17	TYPE (38)	Z
17	DESCRIPTION (39)	NA
18	PERSONNEL INJURIES NUMBER (40)	0 0 0
18	DESCRIPTION (41)	NA
19	LOSS OF OR DAMAGE TO FACILITY TYPE (42)	Z
19	DESCRIPTION (43)	NA
20	PUBLICITY ISSUED (44)	N
20	DESCRIPTION (45)	NA

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