



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**
WASHINGTON, D.C. 20555-0001

May 26, 2016

Vice President, Operations
Entergy Operations, Inc.
Grand Gulf Nuclear Station
P.O. Box 756
Port Gibson, MS 39150

**SUBJECT: GRAND GULF NUCLEAR STATION, UNIT 1 - CORRECTION TO AMENDMENT
NO. 207, RE: REQUEST FOR CHANGING FIVE TECHNICAL
SPECIFICATIONS ALLOWABLE VALUES (CAC NO. MF4693)**

Dear Sir or Madam:

By letter dated August 31, 2015 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML15195A355), the U.S. Nuclear Regulatory Commission issued Amendment No. 207 to Facility Operating License No. NPF-29 for the Grand Gulf Nuclear Station, Unit 1 (GGNS). This amendment consisted of changes to the Technical Specifications (TS) in response to Entergy Operations, Inc.'s application dated August 1, 2014 (ADAMS Accession No. ML 14216A383), as supplemented by letters dated March 3, 2015, and June 30, 2015 (ADAMS Accession Nos. ML15063A279 and ML15181A152, respectively).

The amendment revised the following five non-conservative Technical Specification Allowable Values (AVs) in the GGNS TSs:

- Automatic Depressurization System Initiation Timer (TS Table 3.3.5.1-1)
- System A and B Containment Spray Timers (TS Table 3.3.6.3-1)
- Division 1 and 2 Degraded 4.16 kiloVolt (kV) Bus Voltage (TS Table 3.3.8.1-1)
- Division 3 Degraded 4.16 kV Bus Voltage (TS Table 3.3.8.1-1)
- Division 3 Degraded 4.16 kV Bus Voltage Time Delay-LOCA (Loss of Coolant Accident) (TS Table 3.3.8.1-1)

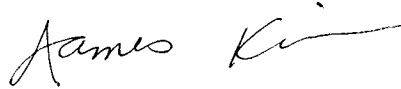
Subsequent to issuance of the amendment, typographical errors were discovered on TS pages 3.3-42, 3.3-43, 3.3-66, and 3.3-79 of the amendment. The footnote (d), under the "Applicable Modes or Other Specified Conditions" title, was not superscripted on TS pages 3.3-42 and 3.3-43, and the letter "c" needs to be replaced with "—" in several places under the "Function" title. On TS pages 3.3-66 and 3.3-79, the letter "c" needs to be replaced with "—" in several places under the "FUNCTION" title. These corrections are editorial in nature and do not affect the previously issued safety evaluation of the amendment.

In addition, TS page 3.3-79, regarding Division 3—4.16 kV Emergency Bus Undervoltage, Degraded Voltage – Time Delay, LOCA (TS Table 3.3.8.1-1 (Item 2.e)), new AV of 3.68 seconds was not revised on the Table 3.3.8.1-1 (Item 2.e). In the safety evaluation of the amendment, the NRC staff evaluated and confirmed a proposed AV of 3.68 seconds for the Division 3 Degraded 4.16 kV Bus Voltage time delay.

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Enclosed are the revised TS pages 3.3-42, 3.3-43, 3.3-66, and 3.3-79. If you have any questions regarding this matter, please contact me at 301-415-4125 or by e-mail at James.Kim@nrc.gov.

Sincerely,

A handwritten signature in black ink, appearing to read "James Kim", with a stylized flourish at the end.

James Kim, Project Manager
Plant Licensing IV-2 and Decommissioning
Transition Branch
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-416

Enclosure:

Corrected TS pages 3.3-42, 3.3-43, 3.3-66
3.3-79 for Amendment No. 207

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Table 3.3.5.1-1 (page 4 of 5)
Emergency Core Cooling System Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS PER FUNCTION	CONDITIONS REFERENCED FROM REQUIRED ACTION A.1	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
4. Automatic Depressurization System (ADS) Trip System A					
a. Reactor Vessel Water Level - Low Low Low, Level 1	1,2(d),3(d)	2	F	SR 3.3.5.1.1 SR 3.3.5.1.2 SR 3.3.5.1.3 SR 3.3.5.1.5 SR 3.3.5.1.6	≥ -152.5 inches
b. Drywell Pressure - High	1,2(d),3(d)	2	F	SR 3.3.5.1.1 SR 3.3.5.1.2 SR 3.3.5.1.3 SR 3.3.5.1.5 SR 3.3.5.1.6	≤ 1.44 psig
c. ADS Initiation Timer	1,2(d),3(d)	1	G	SR 3.3.5.1.2 SR 3.3.5.1.4 SR 3.3.5.1.6	≤ 115 seconds
d. Reactor Vessel Water Level - Low, Level 3 (Confirmatory)	1,2(d),3(d)	1	F	SR 3.3.5.1.1 SR 3.3.5.1.2 SR 3.3.5.1.3 SR 3.3.5.1.5 SR 3.3.5.1.6	≥ 10.8 inches
e. LPCS Pump Discharge Pressure - High	1,2(d),3(d)	2	G	SR 3.3.5.1.1 SR 3.3.5.1.2 SR 3.3.5.1.3 SR 3.3.5.1.5 SR 3.3.5.1.6	≥ 125 psig and ≤ 165 psig
f. LPCI Pump A Discharge Pressure - High	1,2(d),3(d)	2	G	SR 3.3.5.1.1 SR 3.3.5.1.2 SR 3.3.5.1.3 SR 3.3.5.1.5 SR 3.3.5.1.6	≥ 115 psig and ≤ 135 psig
g. ADS Bypass Timer (High Drywell Pressure)	1,2(d),3(d)	2	G	SR 3.3.5.1.2 SR 3.3.5.1.4 SR 3.3.5.1.6	≤ 9.4 minutes
h. Manual Initiation	1,2(d),3(d)	2/system	G	SR 3.3.5.1.6	NA

(continued)

(d) With reactor steam dome pressure > 150 psig.

Table 3.3.5.1-1 (page 5 of 5)
Emergency Core Cooling System Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS PER FUNCTION	CONDITIONS REFERENCED FROM REQUIRED ACTION A.1	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
5. ADS Trip System B					
a. Reactor Vessel Water Level - Low Low Low, Level 1	1,2(d),3(d)	2	F	SR 3.3.5.1.1 SR 3.3.5.1.2 SR 3.3.5.1.3 SR 3.3.5.1.5 SR 3.3.5.1.6	≥ -152.5 inches
b. Drywell Pressure - High	1,2(d),3(d)	2	F	SR 3.3.5.1.1 SR 3.3.5.1.2 SR 3.3.5.1.3 SR 3.3.5.1.5 SR 3.3.5.1.6	≤ 1.44 psig
c. ADS Initiation Timer	1,2(d),3(d)	1	G	SR 3.3.5.1.2 SR 3.3.5.1.4 SR 3.3.5.1.6	≤ 115 seconds
d. Reactor Vessel Water Level - Low, Level 3 (Confirmatory)	1,2(d),3(d)	1	F	SR 3.3.5.1.1 SR 3.3.5.1.2 SR 3.3.5.1.3 SR 3.3.5.1.5 SR 3.3.5.1.6	≥ 10.8 inches
e. LPCI Pumps B & C Discharge Pressure - High	1,2(d),3(d)	2 per pump	G	SR 3.3.5.1.1 SR 3.3.5.1.2 SR 3.3.5.1.3 SR 3.3.5.1.5 SR 3.3.5.1.6	≥ 115 psig and ≤ 135 psig
f. ADS Bypass Timer (High Drywell Pressure)	1,2(d),3(d)	2	G	SR 3.3.5.1.2 SR 3.3.5.1.4 SR 3.3.5.1.6	≤ 9.4 minutes
g. Manual Initiation	1,2(d),3(d)	2/system	G	SR 3.3.5.1.6	NA

(d) With reactor steam dome pressure > 150 psig.

RHR Containment Spray System Instrumentation
3.3.6.3

Table 3.3.6.3-1 (page 1 of 1)
RHR Containment Spray System Instrumentation

FUNCTION	REQUIRED CHANNELS PER TRIP SYSTEM	CONDITIONS REFERENCED FROM REQUIRED ACTION A.1	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
1. Drywell Pressure—High	2	B	SR 3.3.6.3.1 SR 3.3.6.3.2 SR 3.3.6.3.3 SR 3.3.6.3.5 SR 3.3.6.3.6	≤ 1.44 psig
2. Containment Pressure—High	1	C	SR 3.3.6.3.1 SR 3.3.6.3.2 SR 3.3.6.3.3 SR 3.3.6.3.5 SR 3.3.6.3.6	≤ 8.34 psig
3. Reactor Vessel Water Level—Low Low Low, Level 1	2	B	SR 3.3.6.3.1 SR 3.3.6.3.2 SR 3.3.6.3.3 SR 3.3.6.3.5 SR 3.3.6.3.6	≥ -152.5 inches
4. System A and System B Timers	1	C	SR 3.3.6.3.2 SR 3.3.6.3.4 SR 3.3.6.3.6	≥ 10.6 minutes and ≤ 11.1 minutes

Table 3.3.8.1-1 (page 1 of 1)
Loss of Power Instrumentation

FUNCTION	REQUIRED CHANNELS		SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
	PER DIVISION			
1. Divisions 1 and 2 - 4.16 kV Emergency Bus Undervoltage				
a. Loss of Voltage - 4.16 kV basis	4	SR 3.3.8.1.1 SR 3.3.8.1.2 SR 3.3.8.1.4	≥ 2621 V and ≤ 2912 V	
b. Loss of Voltage - Time Delay	2	SR 3.3.8.1.3 SR 3.3.8.1.4	≥ 0.4 seconds and ≤ 1.0 seconds	
c. Degraded Voltage - 4.16 kV basis	4	SR 3.3.8.1.1 SR 3.3.8.1.2 SR 3.3.8.1.4	≥ 3764.25 V and ≤ 3837.6 V	
d. Degraded Voltage - Time Delay	2	SR 3.3.8.1.3 SR 3.3.8.1.4	≥ 8.5 seconds and ≤ 9.5 seconds	
2. Division 3 - 4.16 kV Emergency Bus Undervoltage				
a. Loss of Voltage - 4.16 kV basis	4	SR 3.3.8.1.3 SR 3.3.8.1.4	≥ 2984 V and ≤ 3106 V	
b. Loss of Voltage - Time Delay	2	SR 3.3.8.1.3 SR 3.3.8.1.4	≥ 2.0 seconds and ≤ 2.5 seconds	
c. Degraded Voltage - 4.16 kV basis	4	SR 3.3.8.1.3 SR 3.3.8.1.4	≥ 3605 V and ≤ 3763.5 V	
d. Degraded Voltage - Time Delay, No LOCA	2	SR 3.3.8.1.3 SR 3.3.8.1.4	≥ 4.5 minutes and ≤ 5.5 minutes	
e. Degraded Voltage - Time Delay, LOCA	4	SR 3.3.8.1.3 SR 3.3.8.1.4	≥ 3.68 seconds and ≤ 4.4 seconds	

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Sincerely,

/RA/

James Kim, Project Manager
Plant Licensing IV-2 and Decommissioning
Transition Branch
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-416

Enclosure:

Corrected TS pages 3.3-42, 3.3-43, 3.3-66
3.3-79 for Amendment No. 207

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DATE	5/25/2016	5/25/2016	5/25/2016	5/26/2016

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