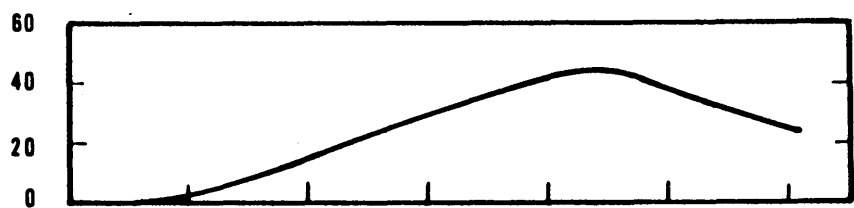
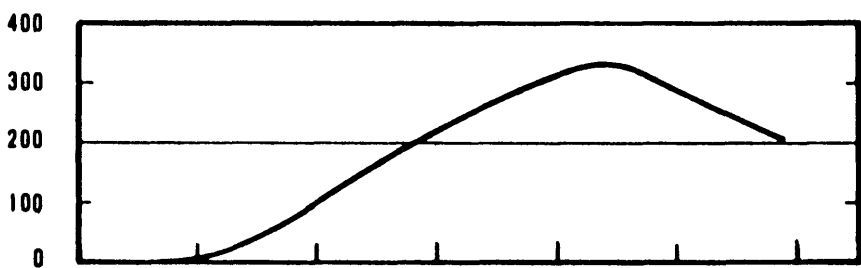


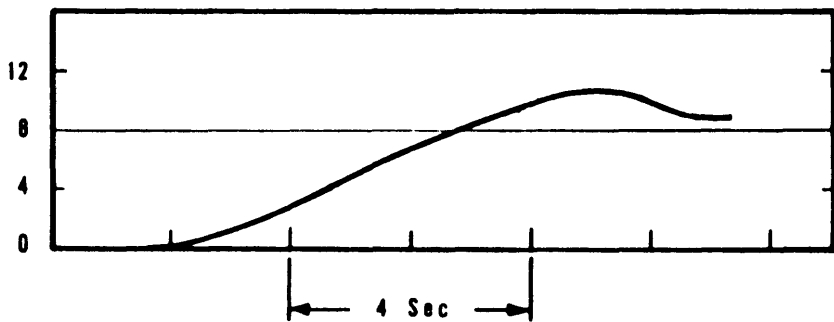
Neutron  
Power, %



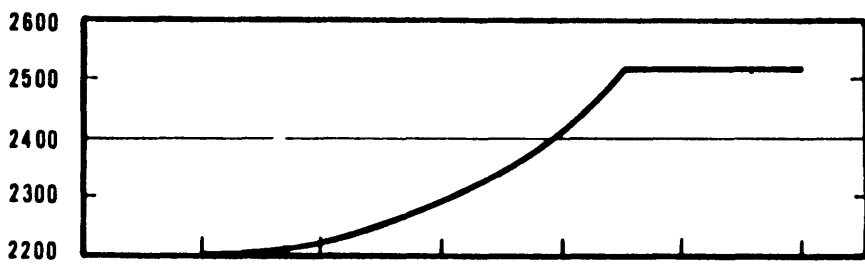
Thermal  
Power, %



Fuel  
Temperature  
Change, F



Average Core  
Moderator  
Temperature  
Change, F



Reactor  
System  
Pressure, psia

P. 14.FIG-1

**GPJ Nuclear**

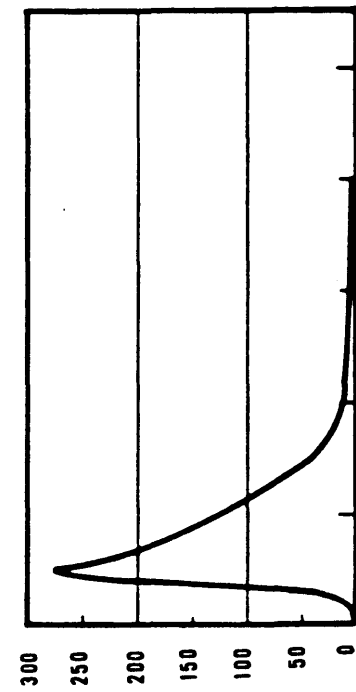
**TMI Unit-1**

**7/82**

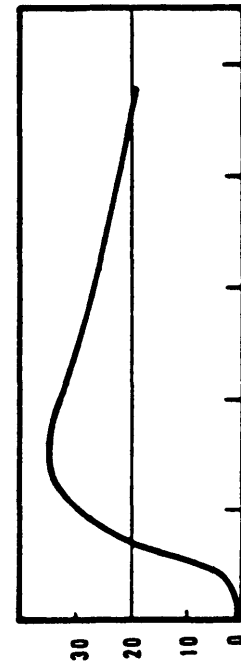
**Update - 1**

Startup Accident from 10<sup>-9</sup> Rated Power  
using a 1.5 ΔK/K Rod Group High Pressure  
Reactor Trip is Actuated

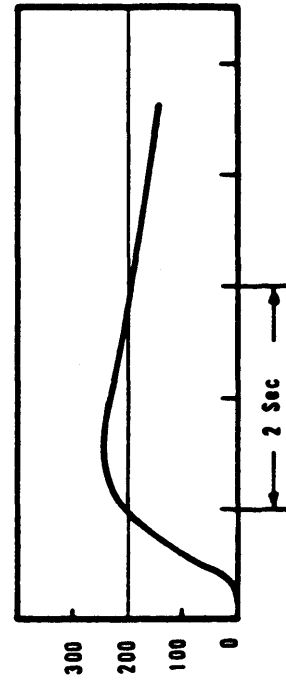
Fig. 14.1-1



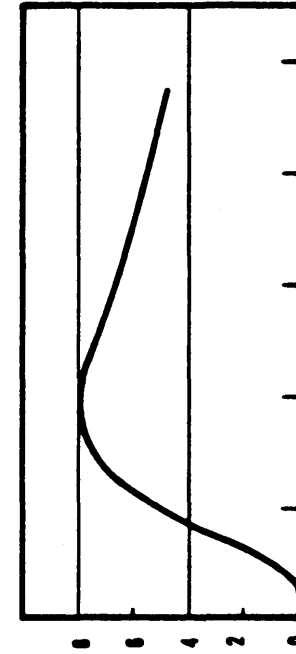
Neutron  
Power, %



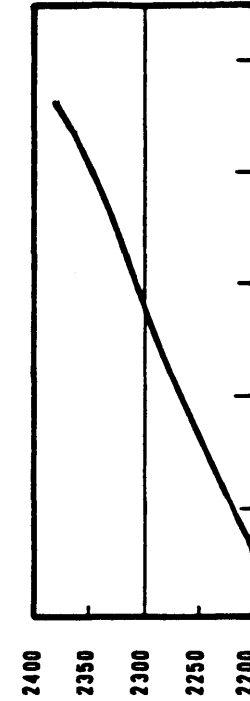
Thermal  
Power, %



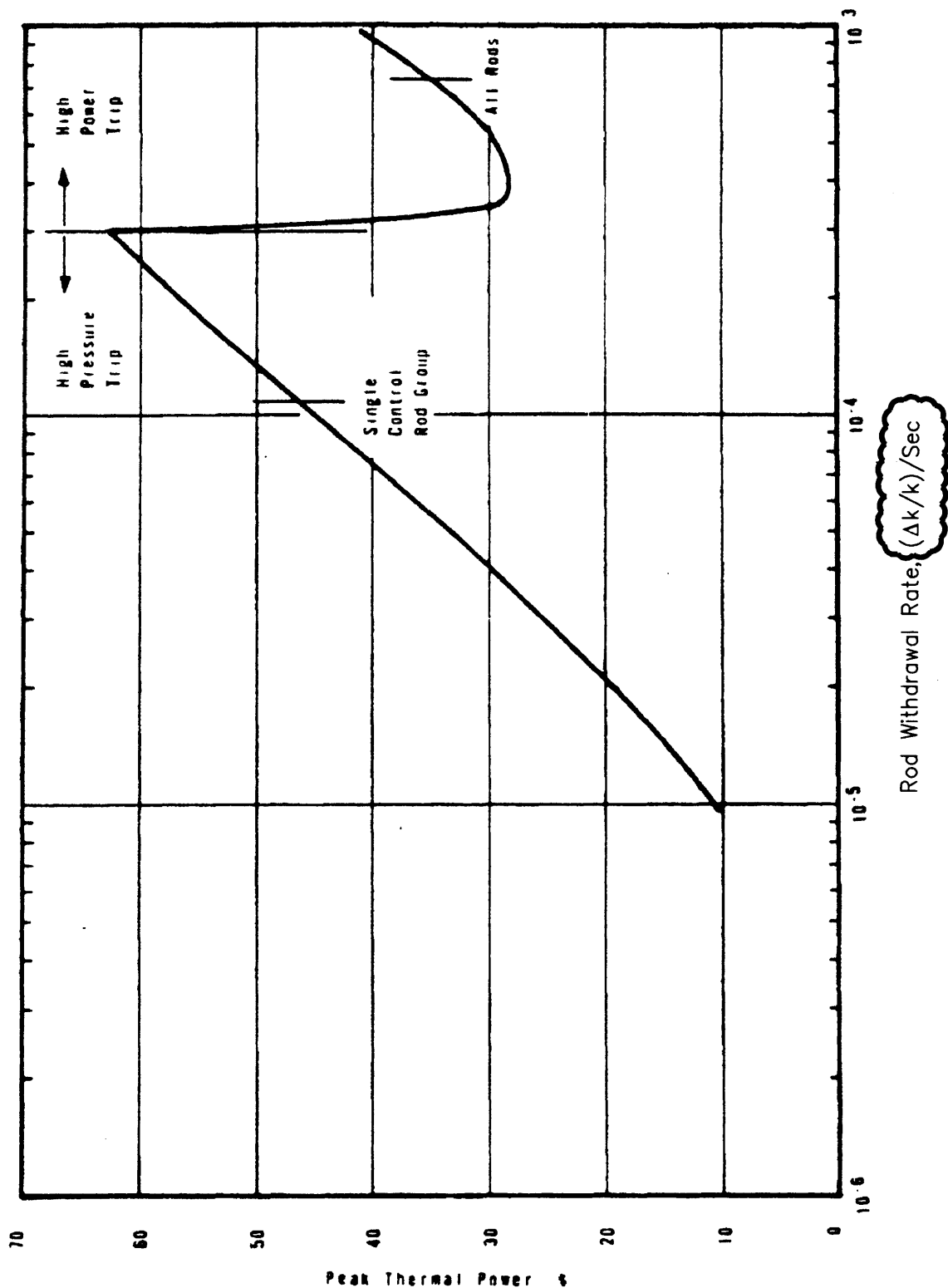
Fuel  
Temperature  
Change, °F




Average Core  
Moderator  
Temperature  
Change, °F



Reactor  
System  
Pressure, psia



p. 14.FIG-3



TMI - UNIT 1

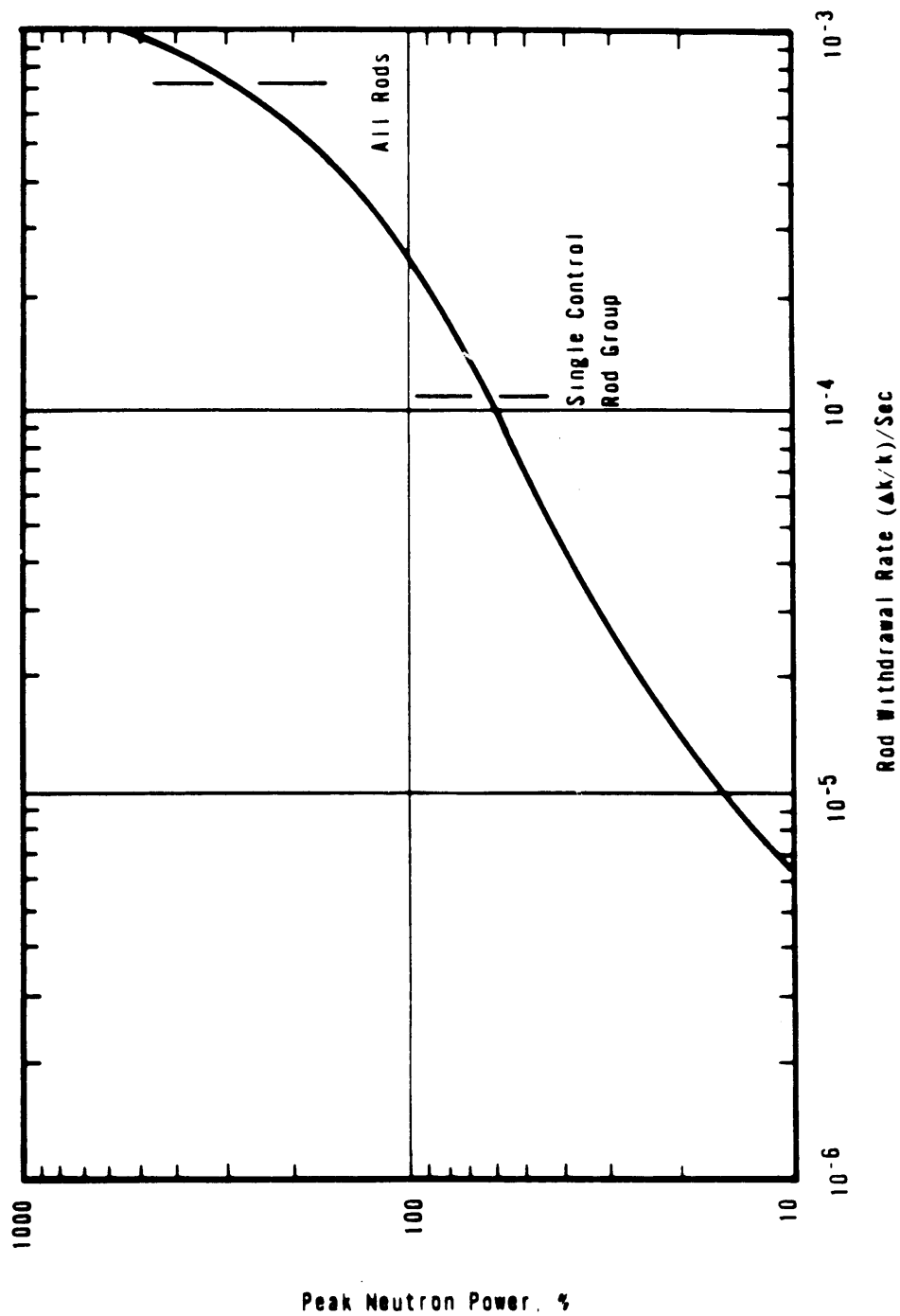
UPDATE - 14

4/98

Peak Thermal Power vs. Rod Withdrawal Rate for a Startup Accident from  $10^{-9}$  Rated Power

CAD FILE: 6555RI4.DWG

FIG 14.1-3



p. 14.FIG-4

**GPU Nuclear**

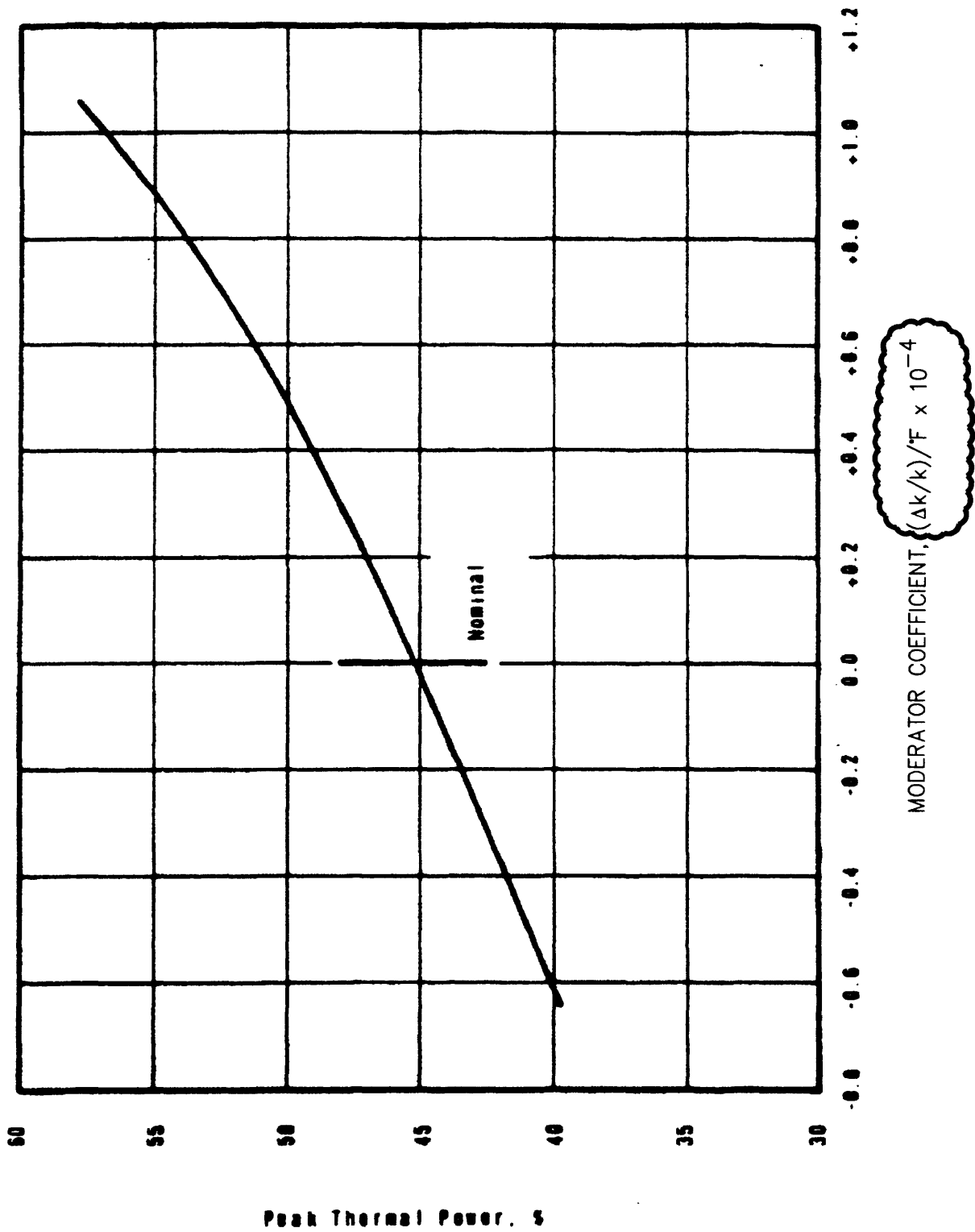
TMI Unit-1

Peak Neutron Power vs. Rod  
Withdrawal Rate for a Startup  
Accident from  $10^{-9}$  Rated Power

Update - 1

7/82

Fig. 14.1-4



p.14.FIG-5

TMI - UNIT 1

**GPU**

**NUCLEAR**

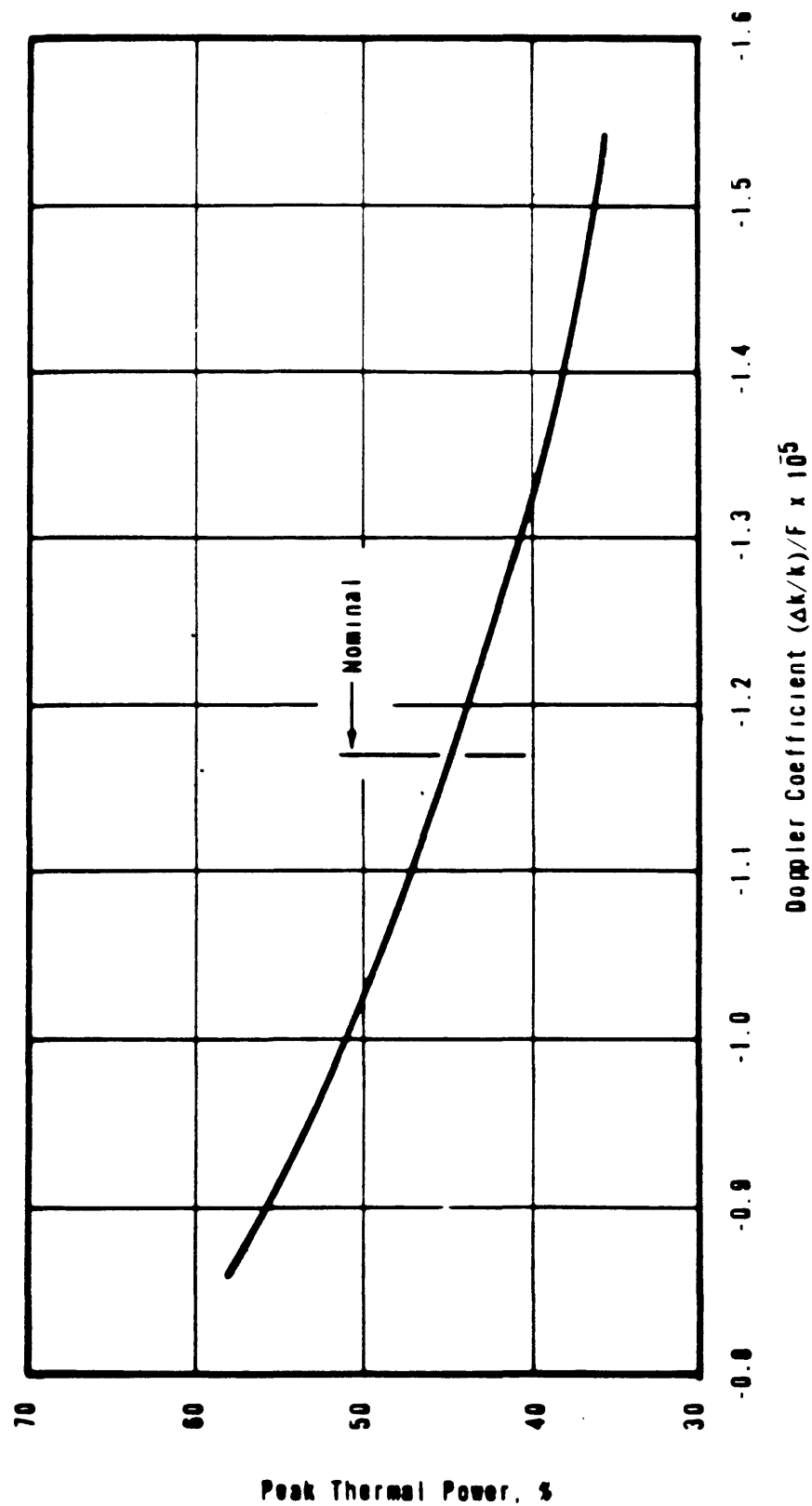
UPDATE - 14

04/98

Peak Thermal Power vs. Moderator Coefficient for a Startup Accident using a 1.5%  $\Delta k/k$  Rod Group at  $1.09 \times 10^{-4} (\Delta k/k)/^{\circ}\text{F}$  FROM  $10^{-9}$  Rated Power

CAD FILE: 6556RI4.DWG

FIG 14.1-5



p.14.FIG-6

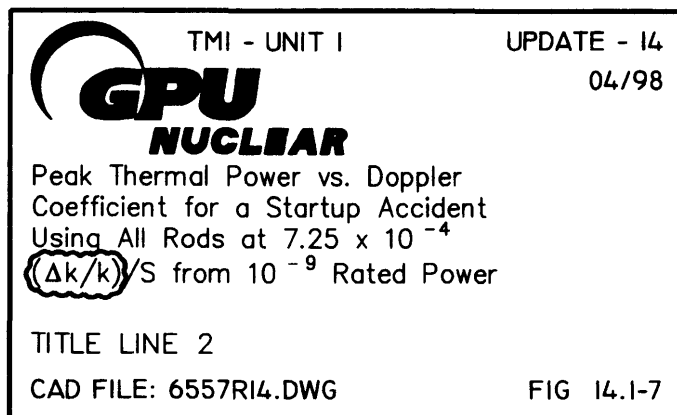
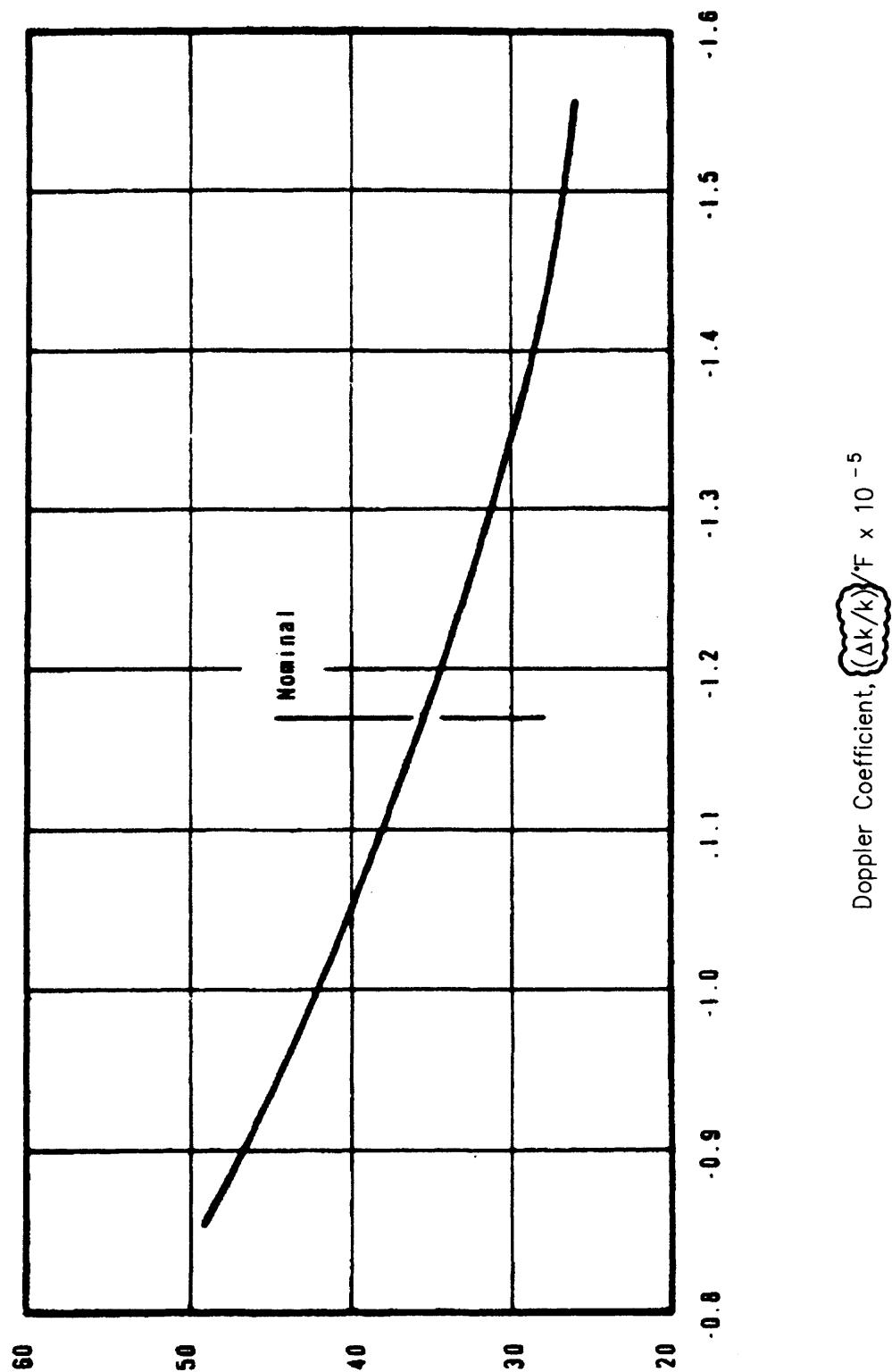
**GPU Nuclear**

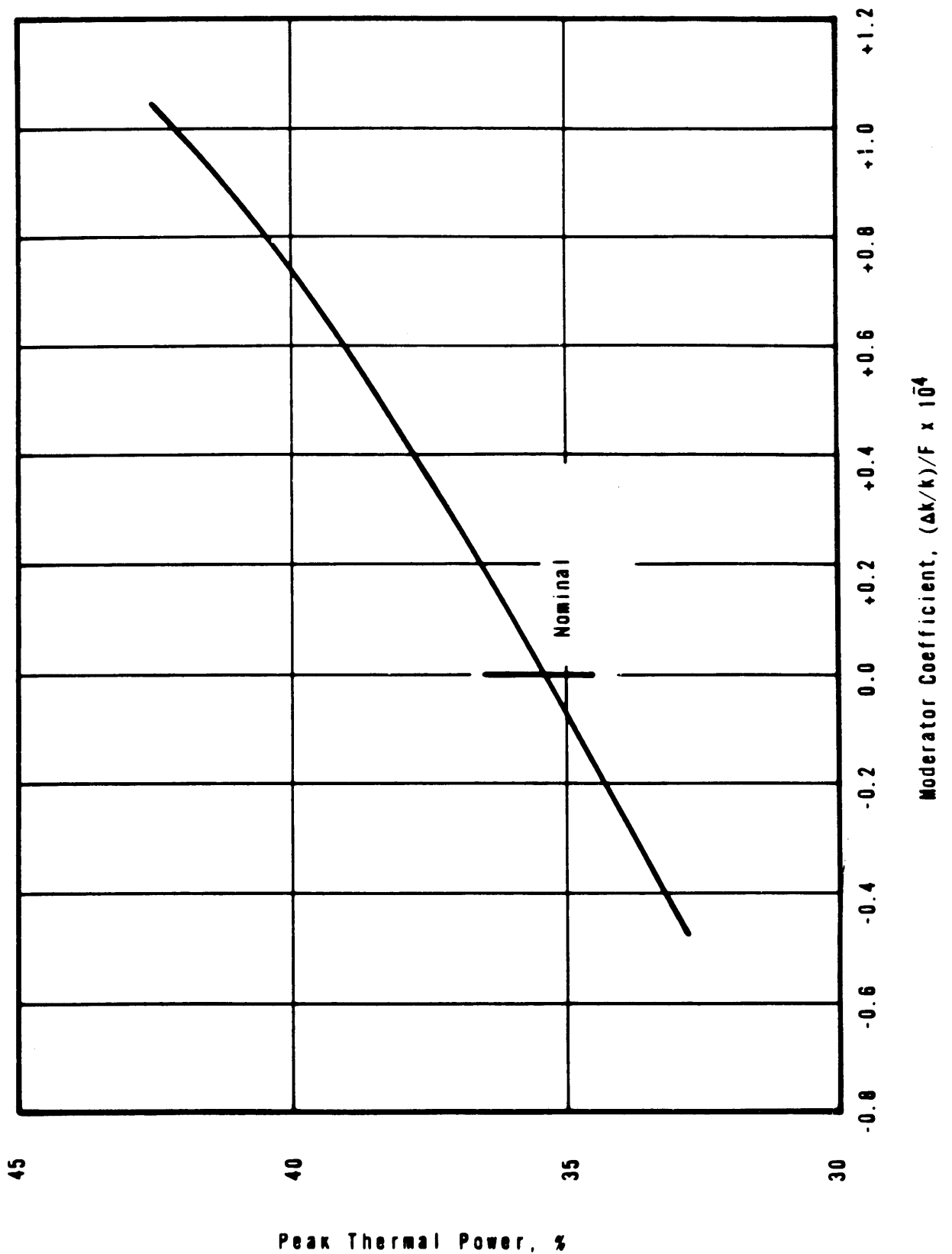
Update - 8R

TMI Unit 1  
 Peak Thermal Power Versus Doppler  
 Coefficient for a Startup Accident  
 Using a 1.5%  $\Delta K/K$  Rod Group at 1.09  
 $\times 10^{-4}(\Delta K/K)/S$  From  $10^{-9}$   
 Rated Power

3/90

Fig. 14.1-6





Peak Thermal Power, %

**GPU Nuclear**

Update -1

**TMI Unit-1**

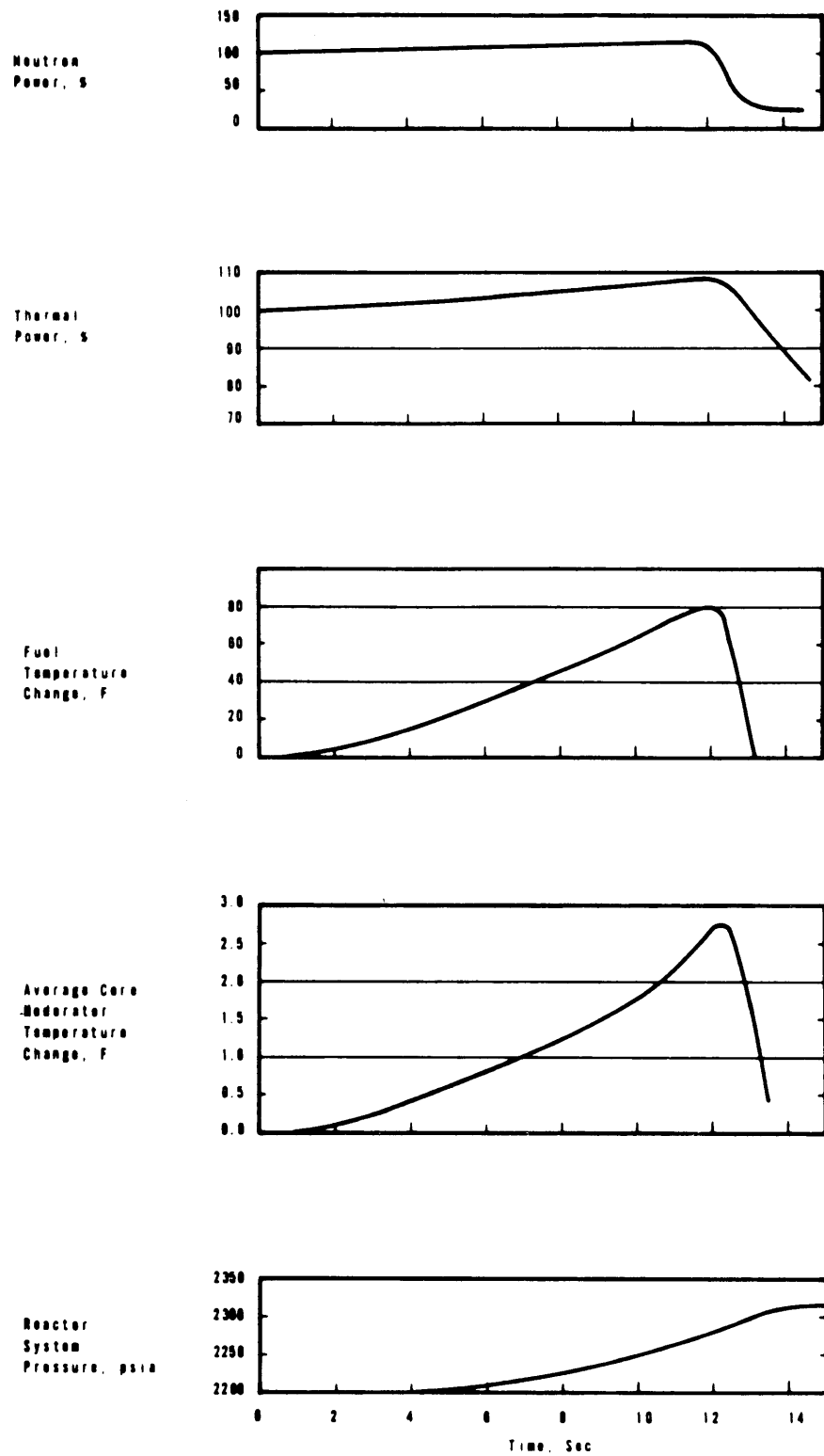
7/82

Peak Thermal Power vs. Moderator  
Coefficient for a Startup Accident  
Using all Rods at  $7.25 \times 10^{-4}$   
 $(\Delta K/K)/S$  From  $10^{-9}$  Rated Power

p. 14.FIG-8

Fig. 14.1-8





**GPU Nuclear**

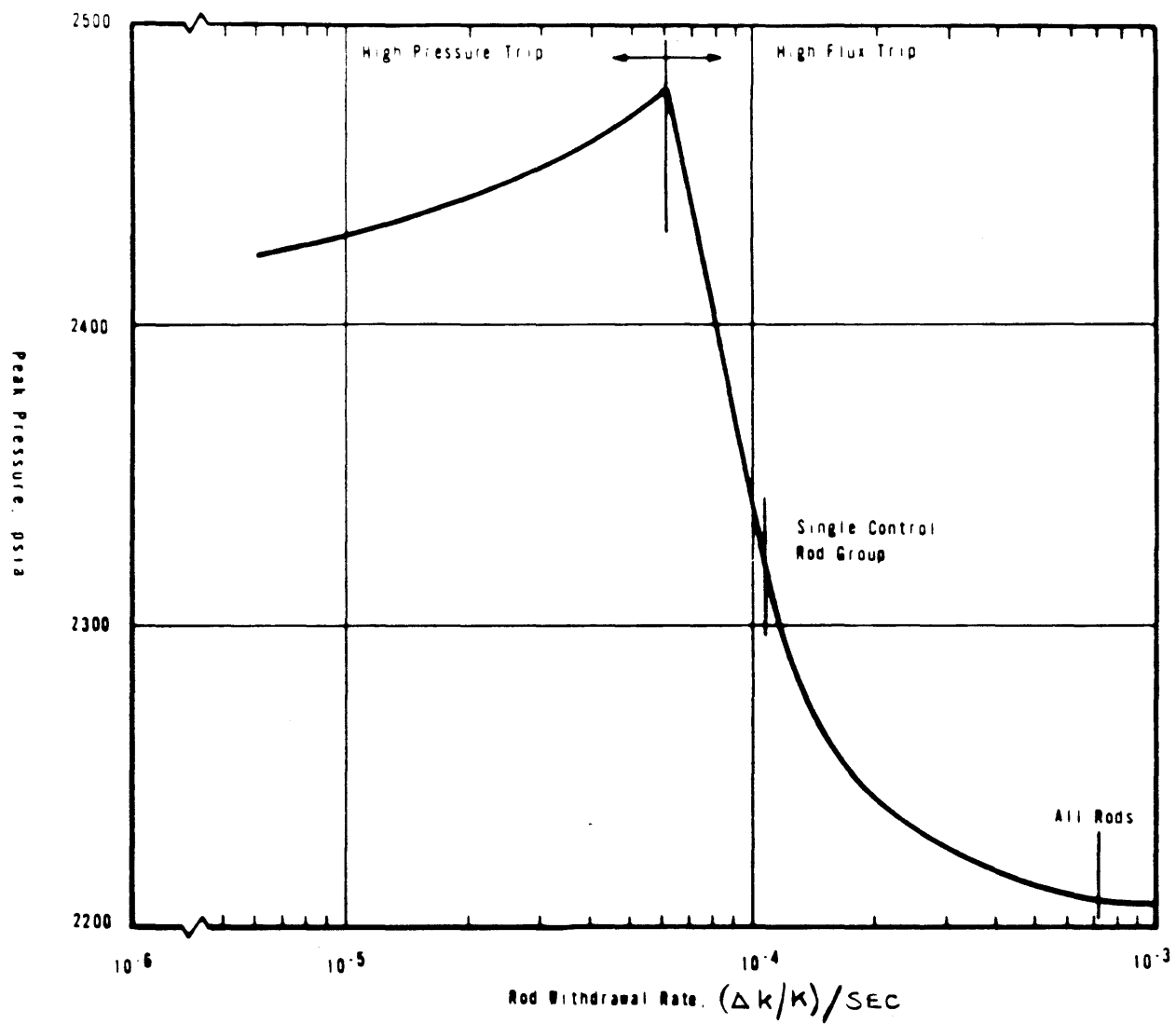
Update - 1

**TMI Unit-1**

7/82

Rod Withdrawal Accident from Rated Power Using a 1.5%  $\Delta K/K$  Rod Group at  $1.09 \times 10^{-4}$  ( $\Delta K/K$ )/S; High Flux Reactor Trip is Actuated

Fig. 14.1-9



**GPU Nuclear**

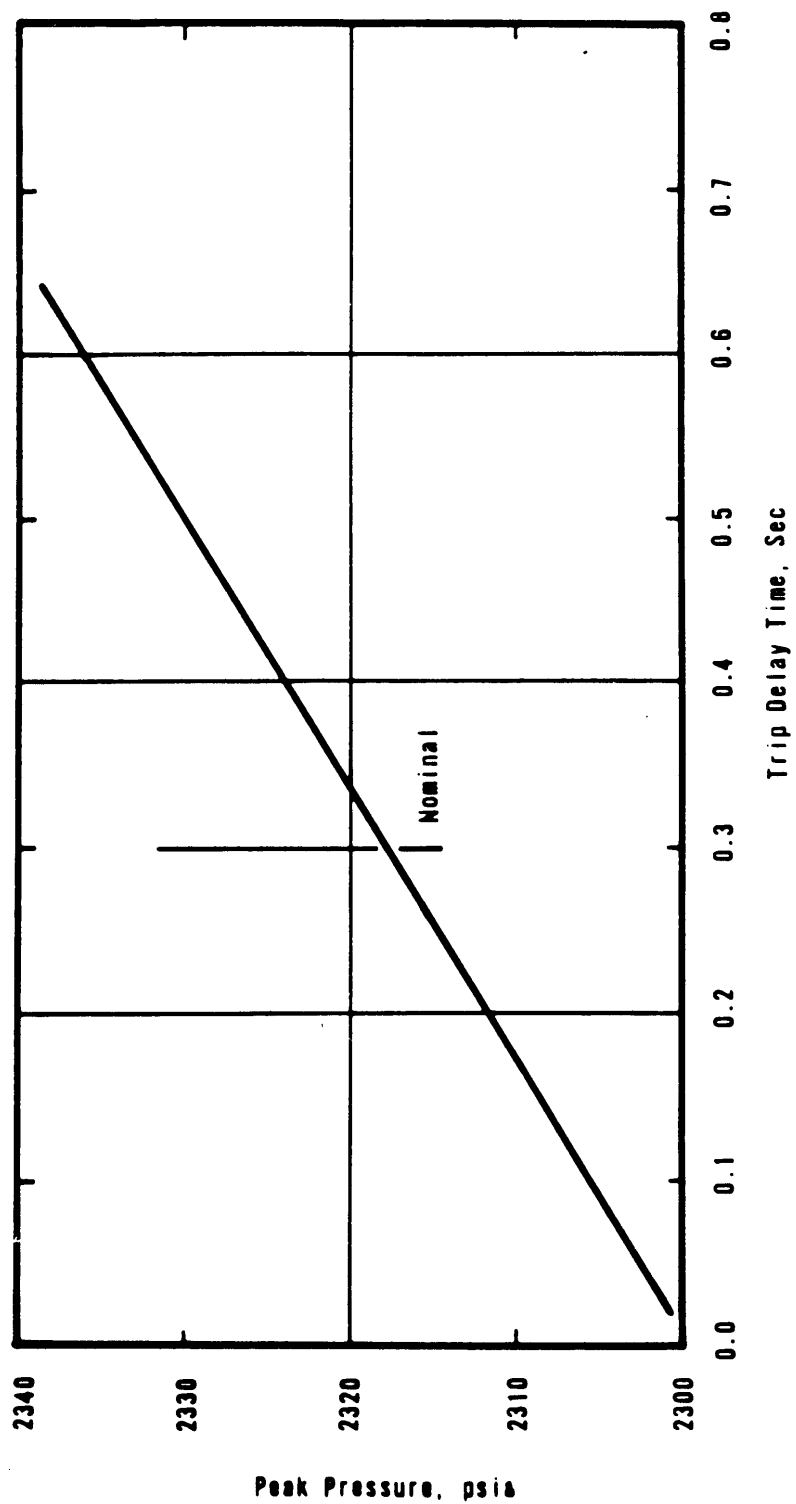
Update - 8R

TMI Unit 1

3/90

Peak Pressure vs. Rod Withdrawal  
Rate for a Rod Withdrawal Accident  
from Rated Power

Fig. 14.1-10



**GPU Nuclear**

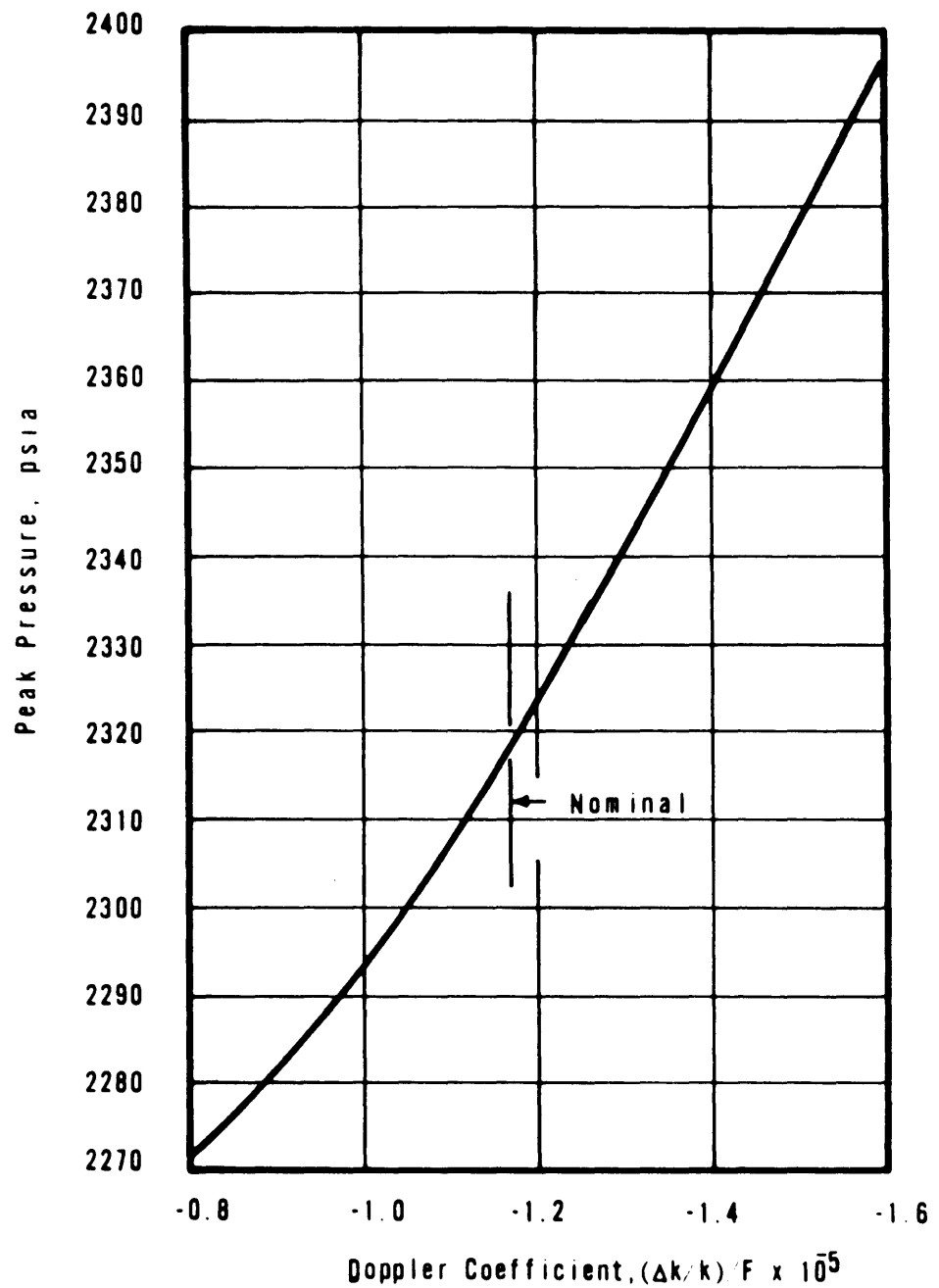
Update - 1

**TMI Unit-1**

7/82

Peak Pressure vs. Trip Delay Time  
For a Rod Withdrawal Accident from  
Rated Power Using a 1.5%  $\Delta K/K$   
Rod Group

Fig. 14.1-11



**GPU Nuclear**

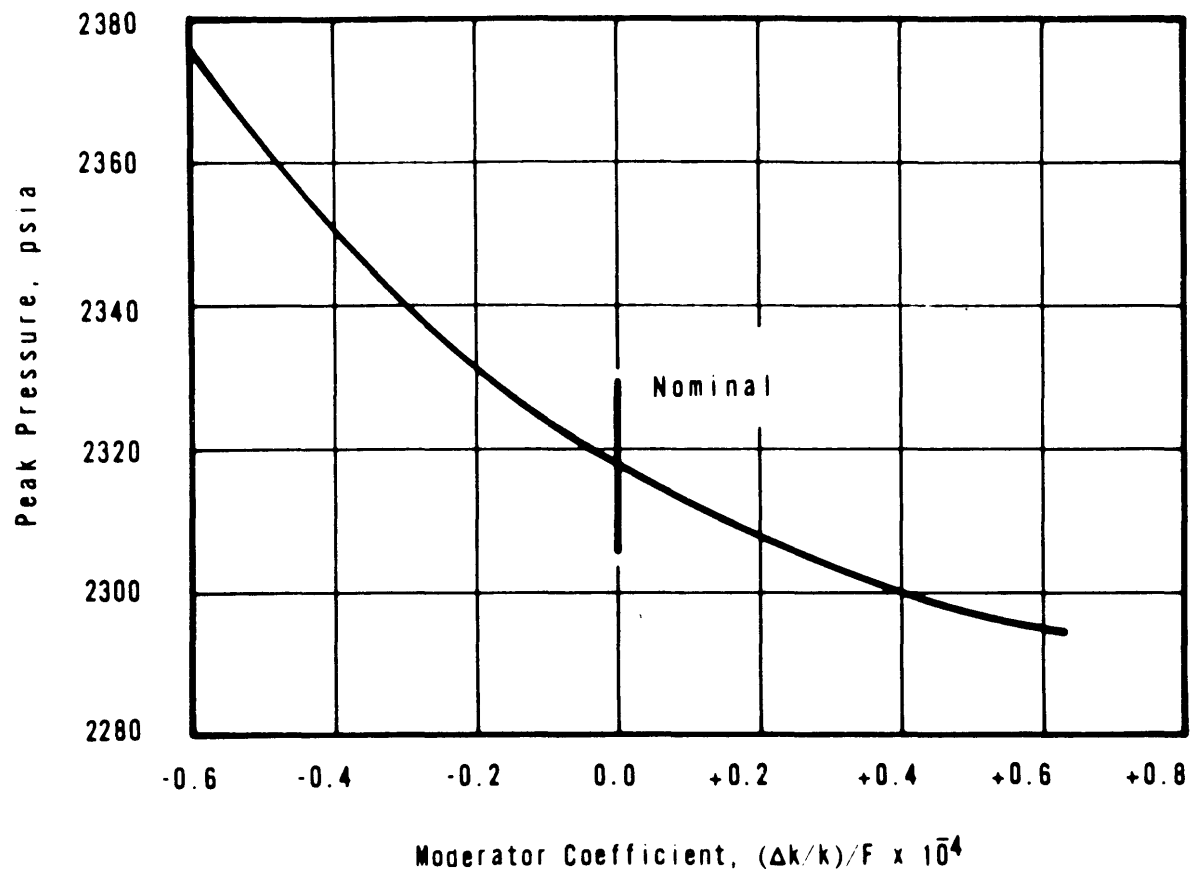
Update - 1

**TMI Unit-1**

7/82

Peak Pressure vs. Doppler Coefficient  
for a Rod Withdrawal Accident from  
Rated Power Using a 1.5%  $\Delta K/K$   
Rod Group

Fig. 14.1-12



**GPU Nuclear**

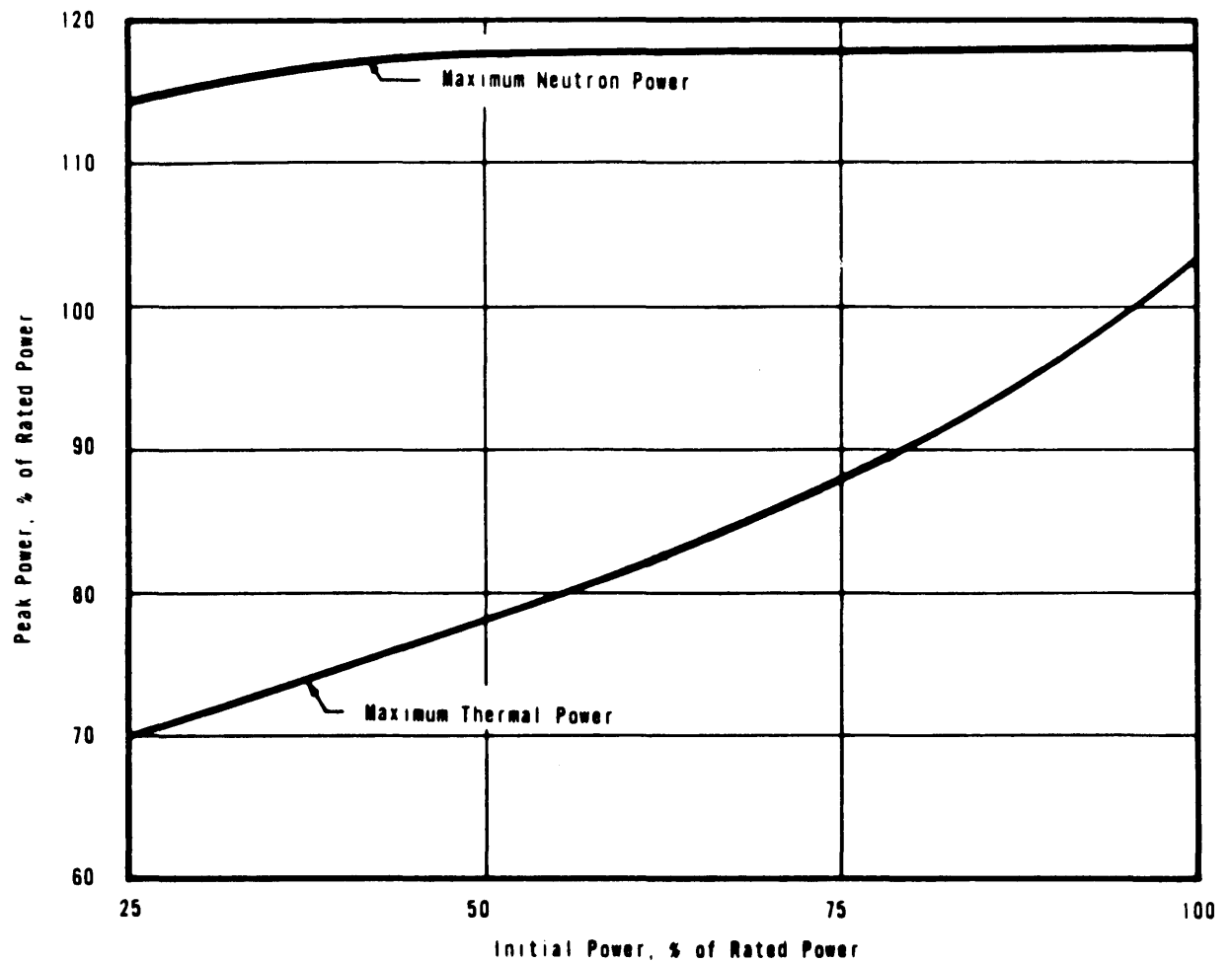
Update - 1

**TMI Unit-1**

7/82

Peak Pressure vs. Moderator Coefficient  
for a Rod Withdrawal Accident from  
Rated Power Using a 1.5%  $\Delta K/K$  Rod  
Group

Fig. 14.1-13



**GPU Nuclear**

Update - 1

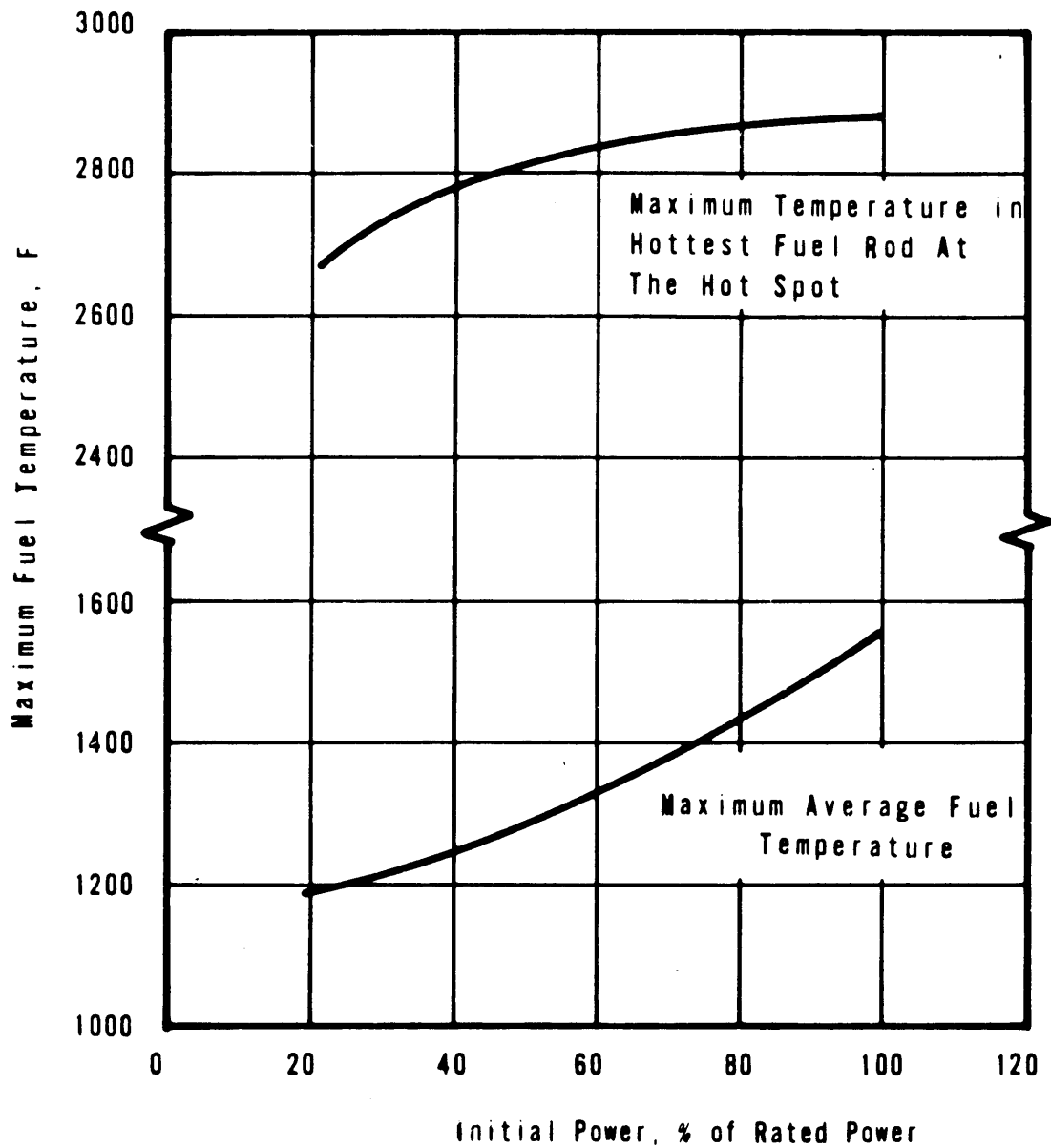
**TMI Unit-1**

7/82

Maximum Neutron and Thermal Power  
for an All-Rod Withdrawal Accident  
from Various Initial Power Levels

p. 14.FIG-14

Fig. 14.1-14



**GPU Nuclear**

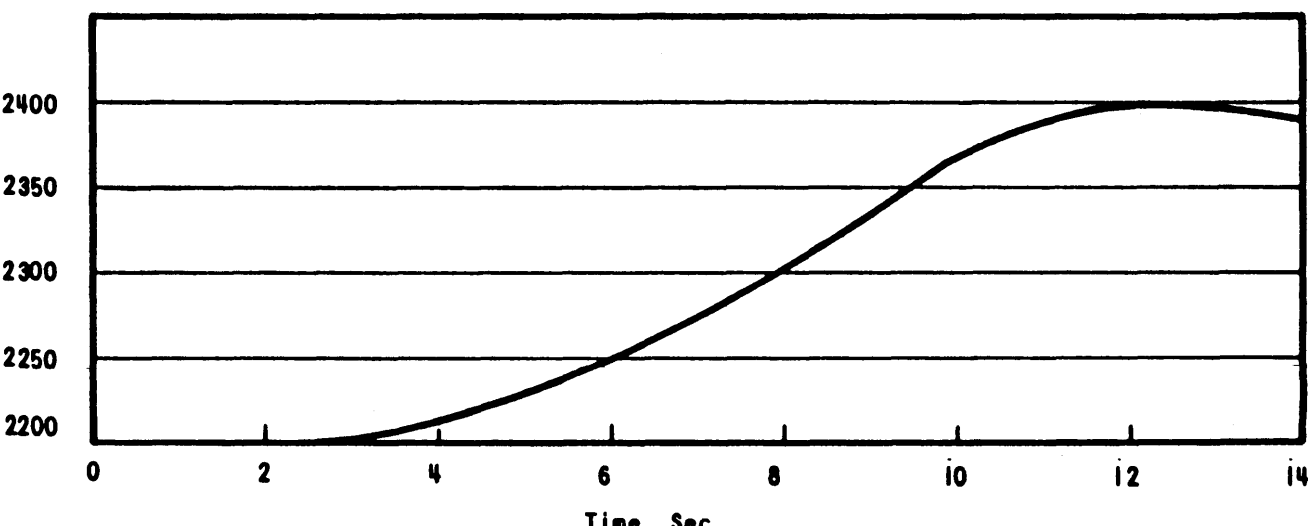
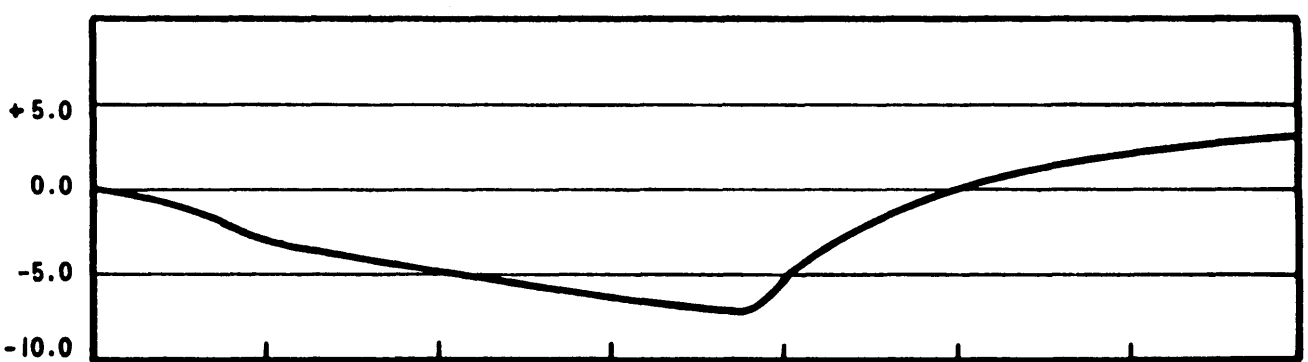
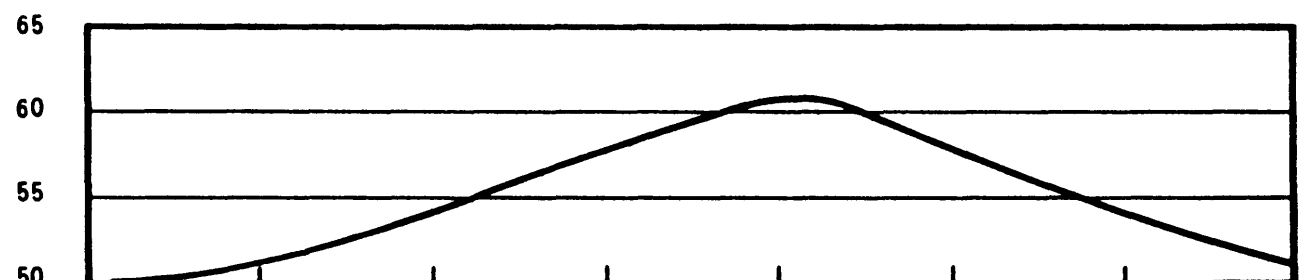
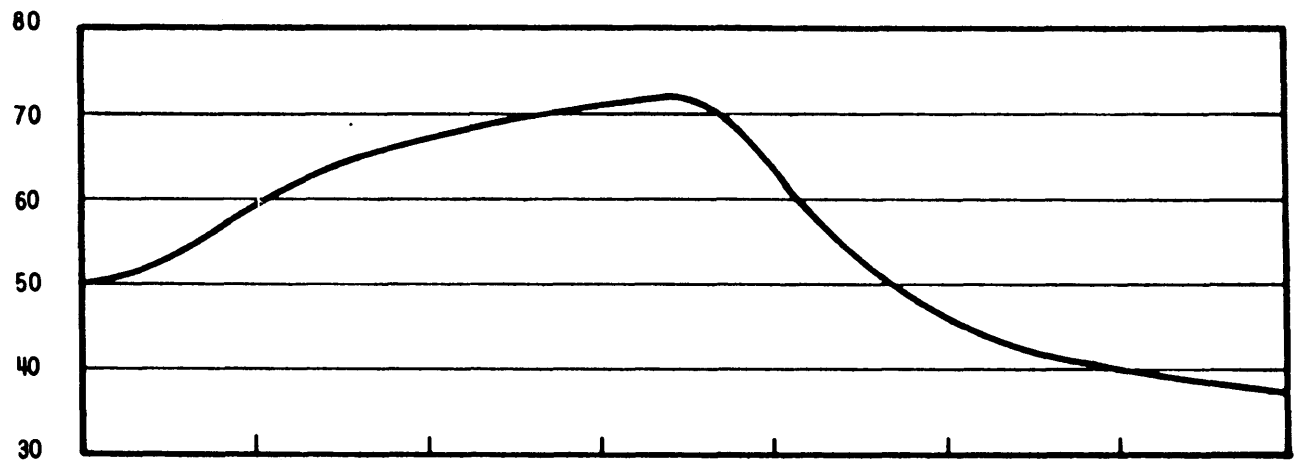
Update - 1

**TMI Unit-1**

7/82

Peak Fuel Temperature in Average  
Rod and Hot Spot for an All-Rod  
Withdrawal Accident from Various  
Initial Power Levels

Fig. 14.1-15



p. 14.FIG-16

Neutron  
Power, %

Thermal  
Power, %

Change in  
Average  
Moderator  
Temperature, F

Reactor  
Coolant  
Pressure, psia

Time, Sec

**EPRI Nuclear**

TMI Unit-1

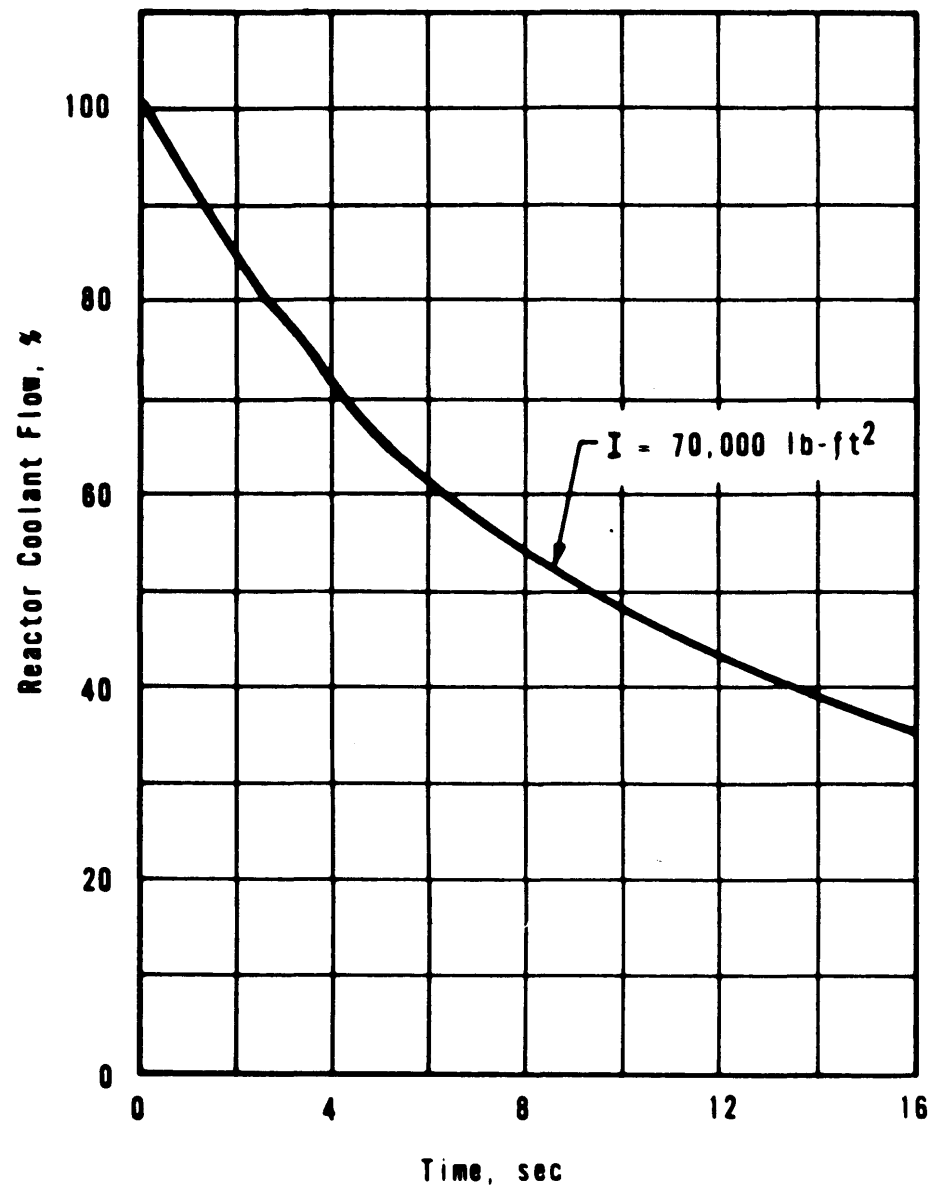
Pump Startup from 50% Power and  
50% Flow

Update -1

7/82

Fig. 14.1-16





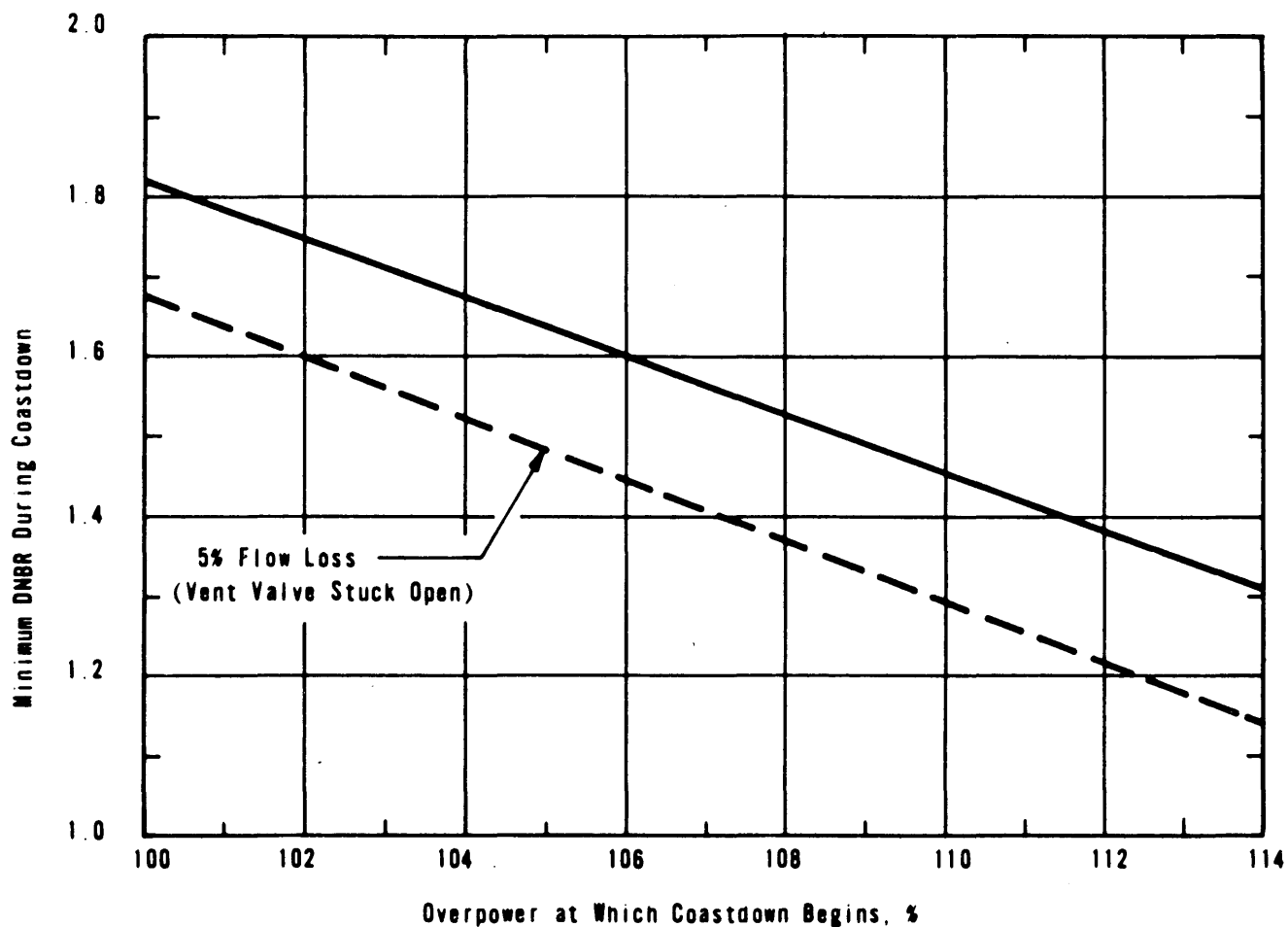
**GPU Nuclear**

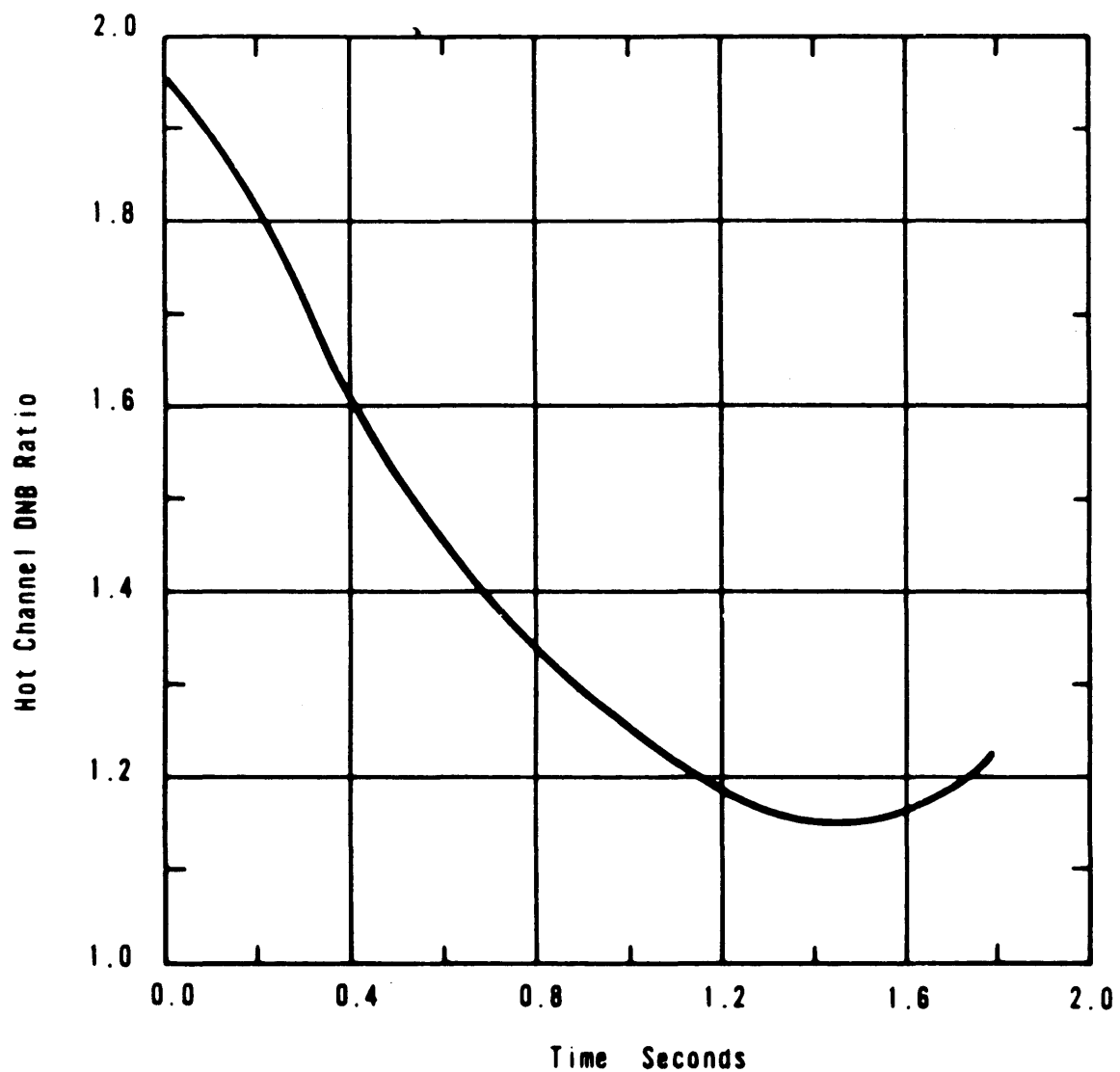
**TMI Unit-1**

Percent Reactor Coolant Flow as a  
Function of Time After Loss of Pump  
Power

Update - 1

7/82





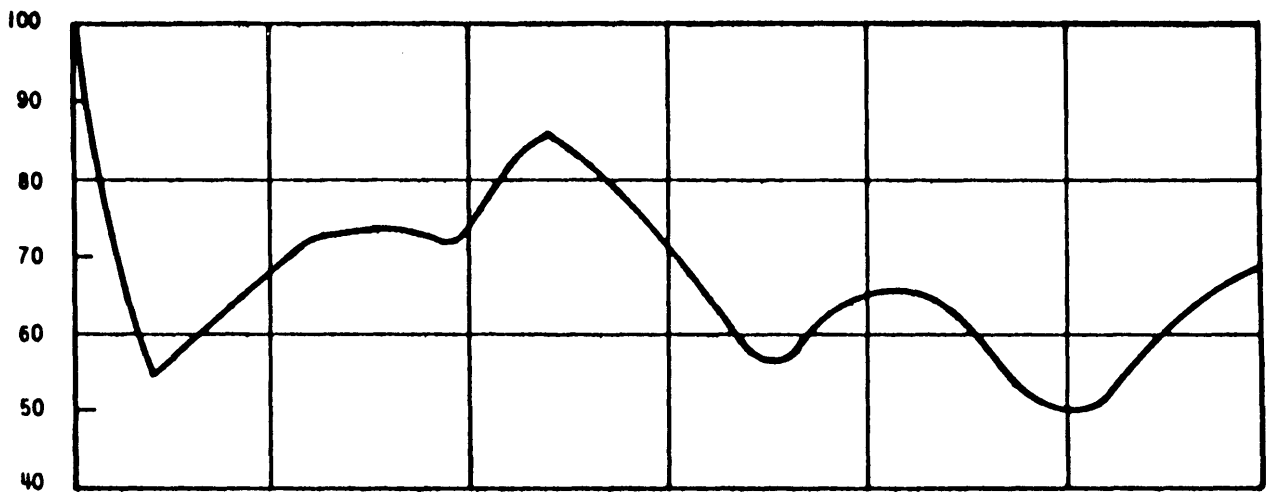
**GPU Nuclear**

**TMI Unit-1**

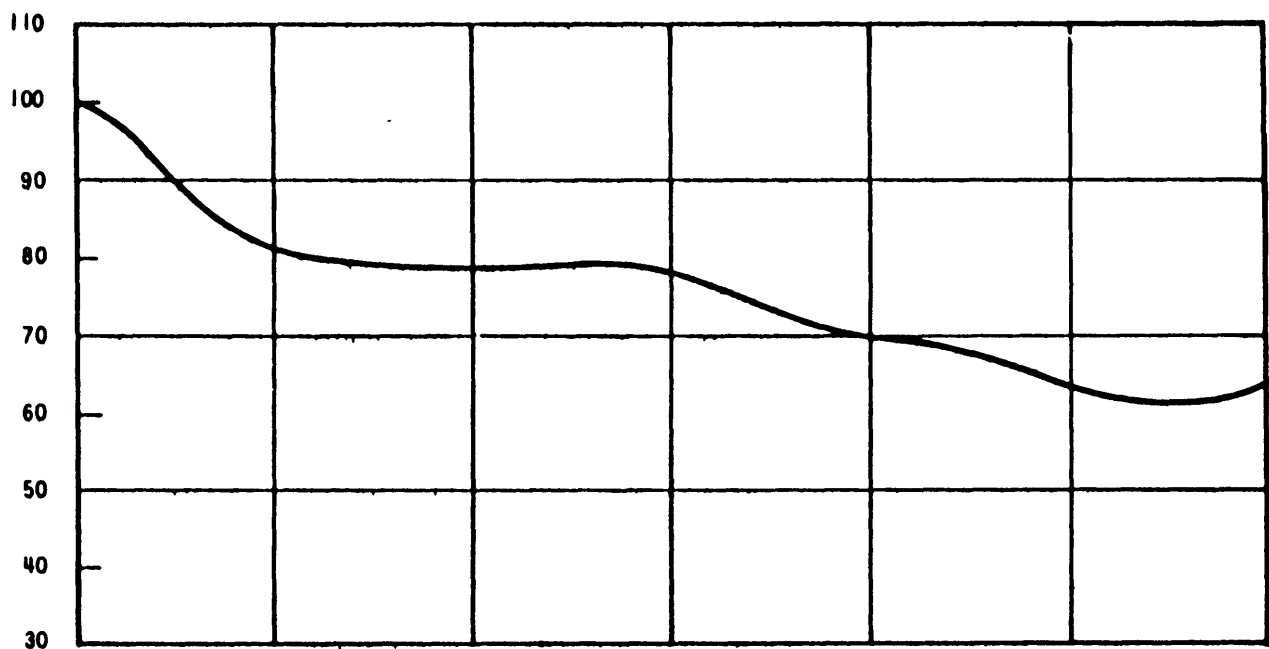
**DNB Ratio Versus Time for a Locked  
Rotor Accident from 102% of Rated Power**

**Update -1**

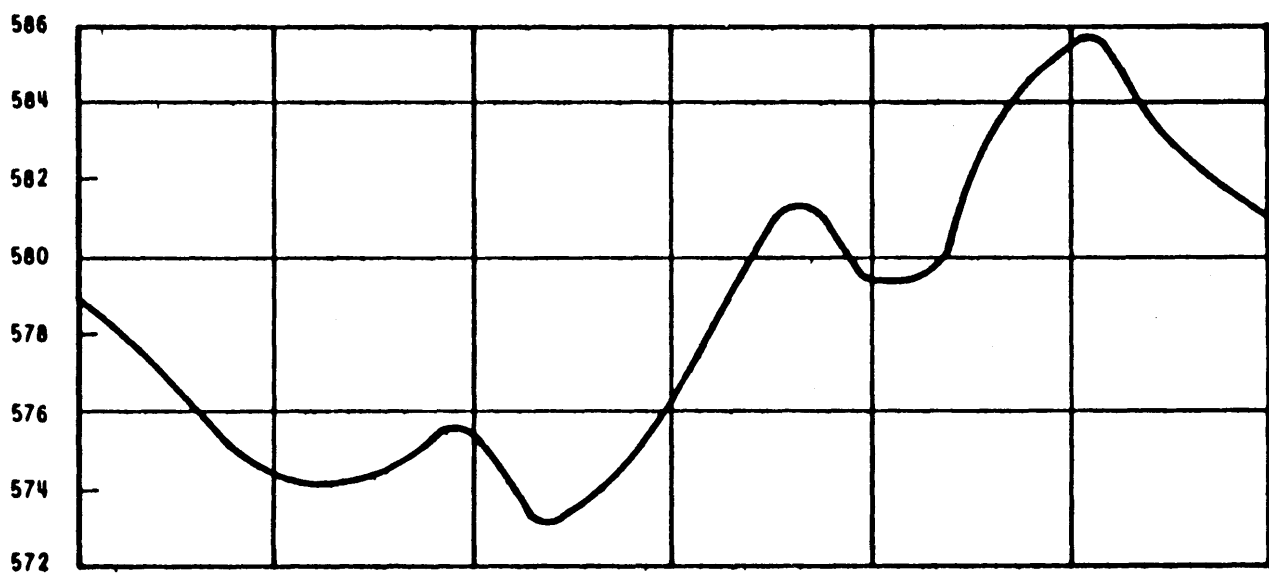
**7/82**



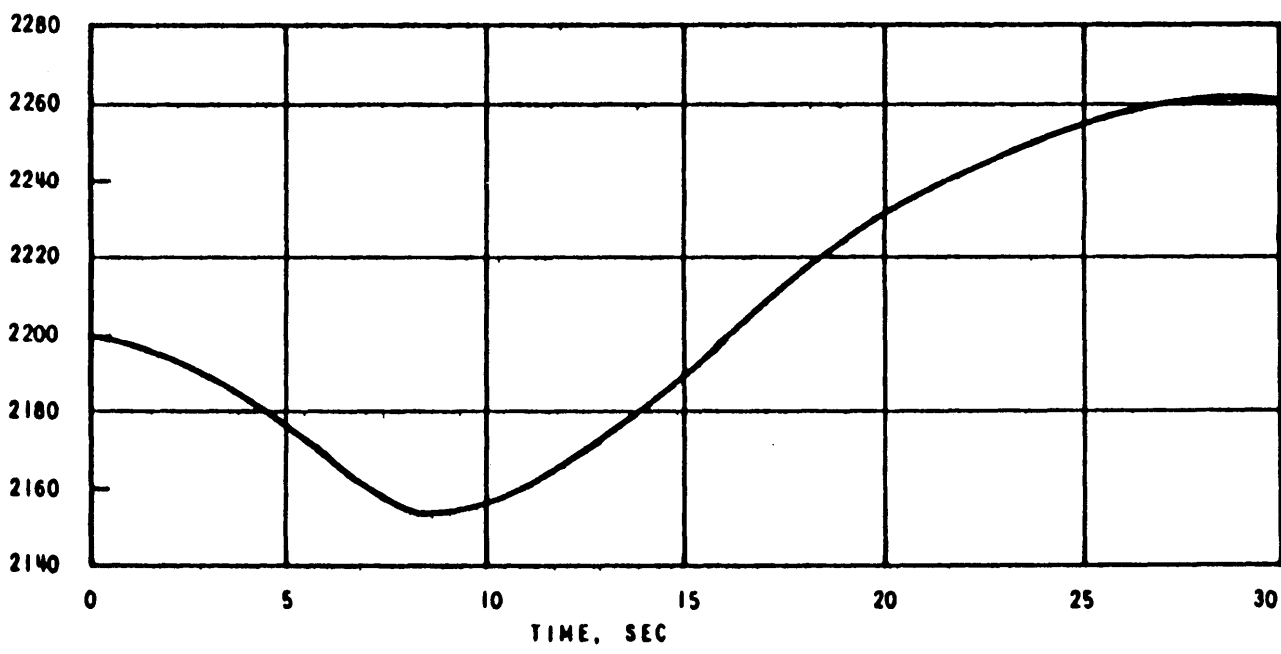
NEUTRON  
POWER  
%



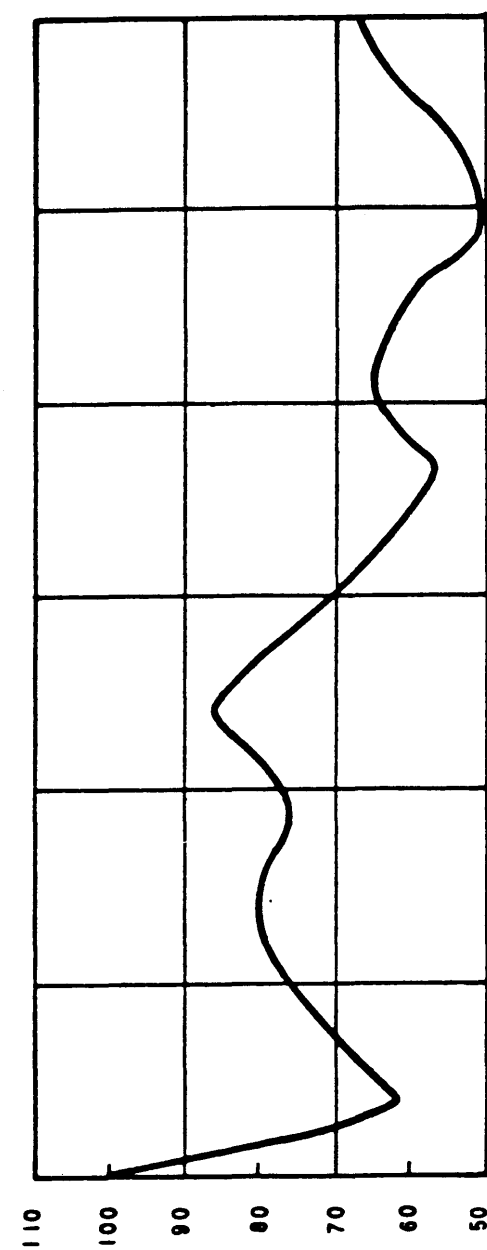
THERMAL  
POWER  
%



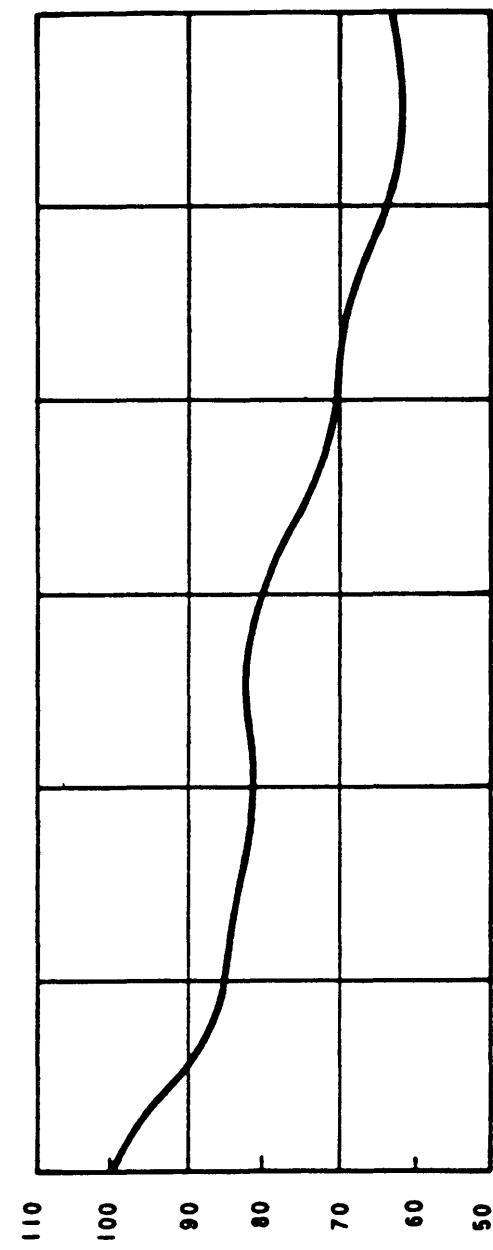
AVERAGE CORE  
MODERATOR  
TEMPERATURE, F



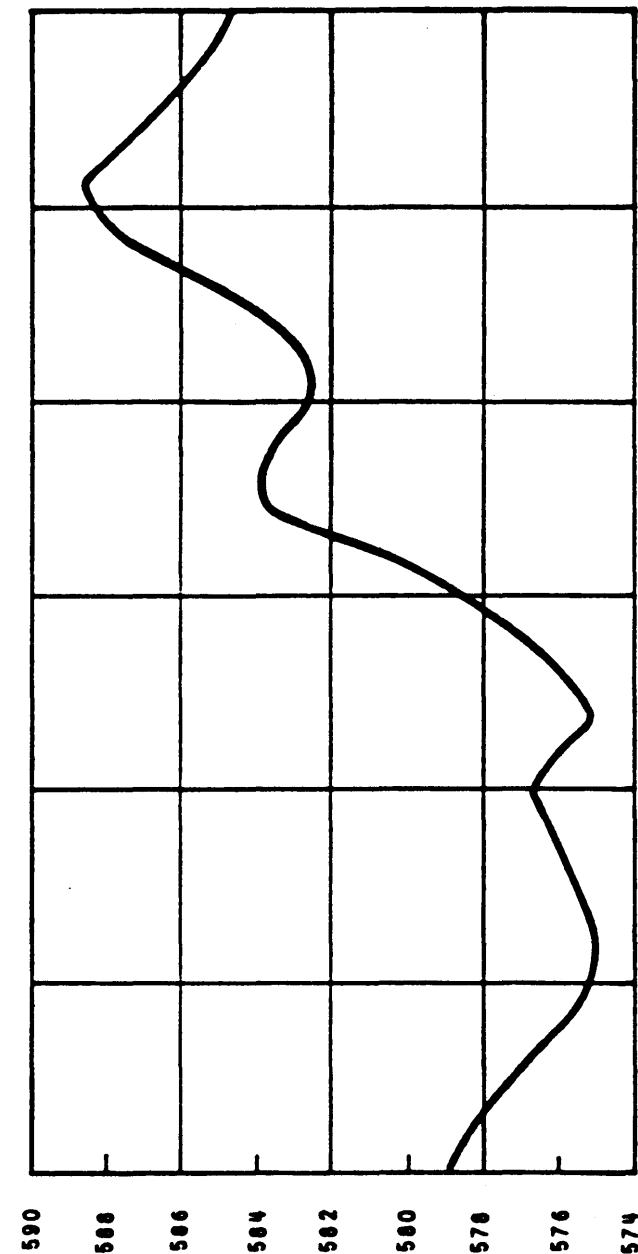
REACTOR  
SYSTEM  
PRESSURE  
psia



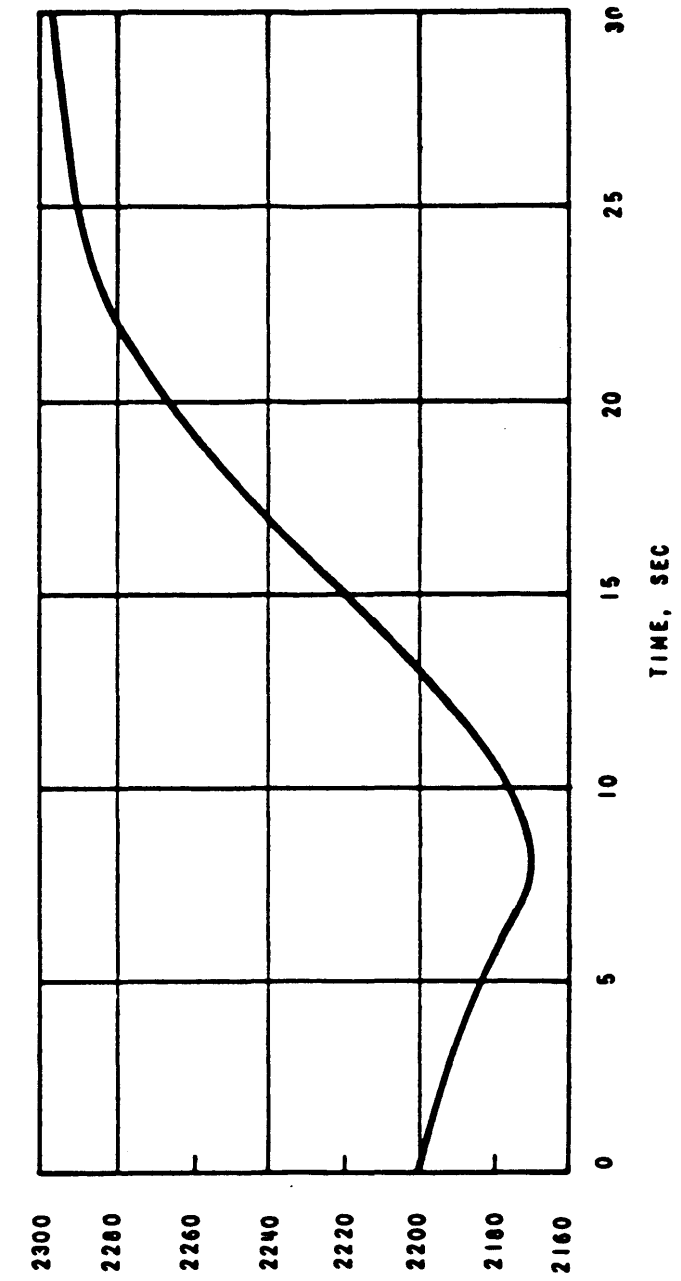
NEUTRON  
POWER  
%



THERMAL  
POWER  
%



AVERAGE CORE  
MODERATOR  
TEMPERATURE, F



REACTOR  
SYSTEM  
PRESSURE  
psia

**GPU Nuclear**

**TMI Unit-1**

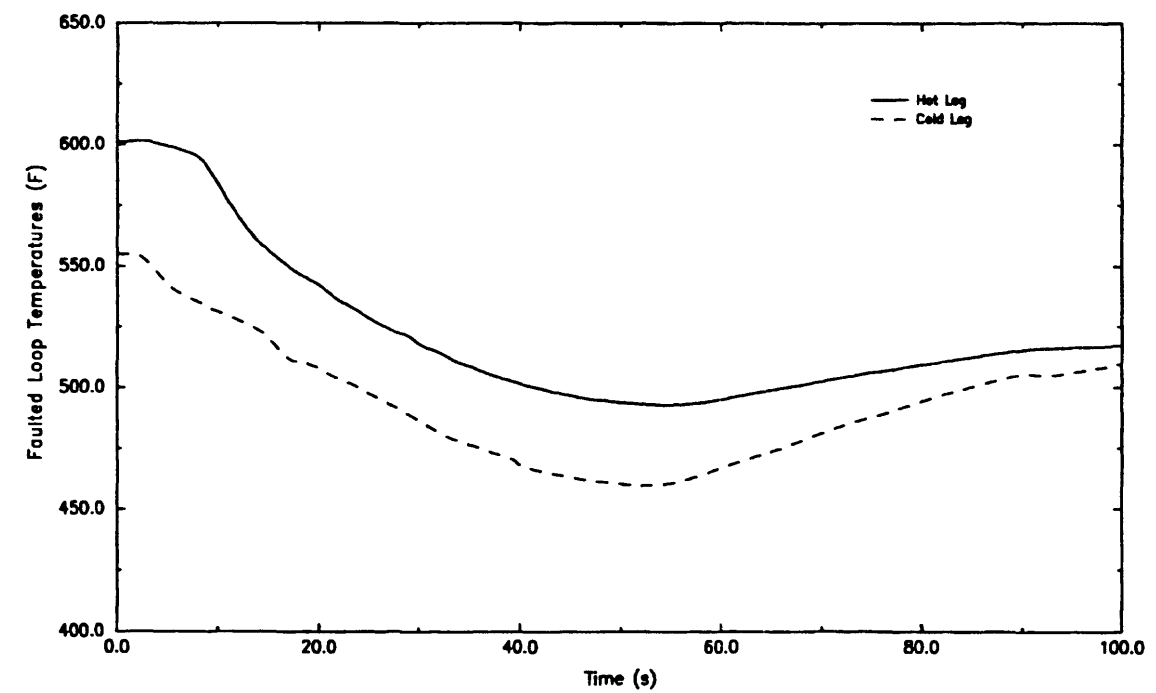
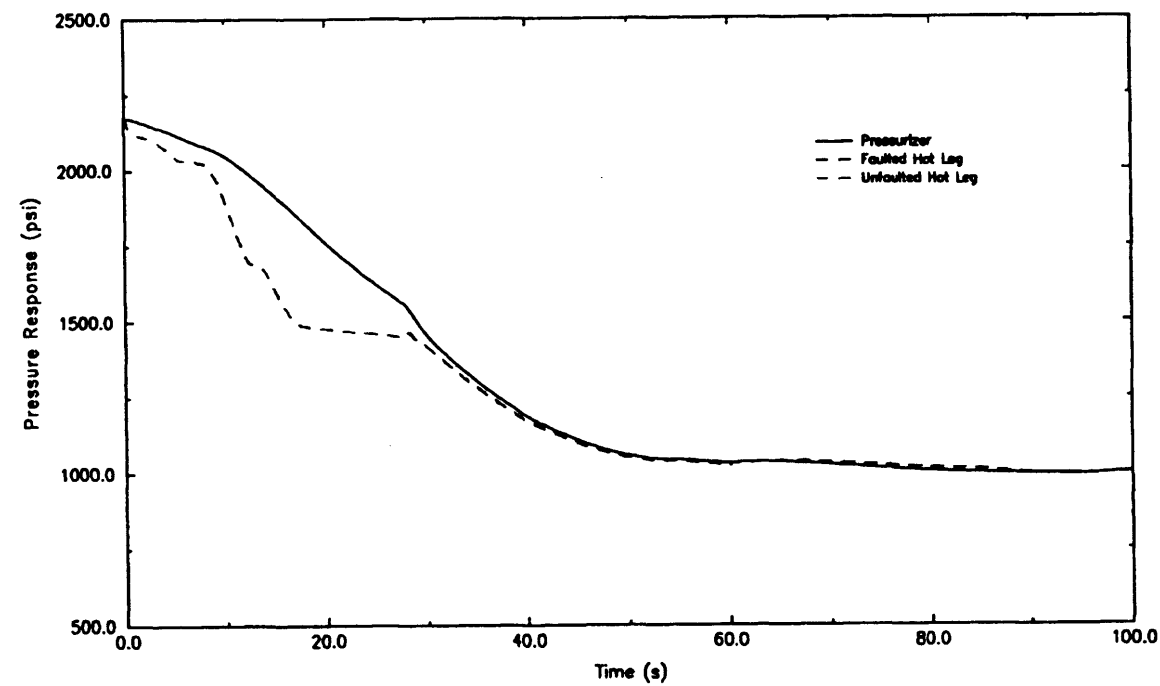
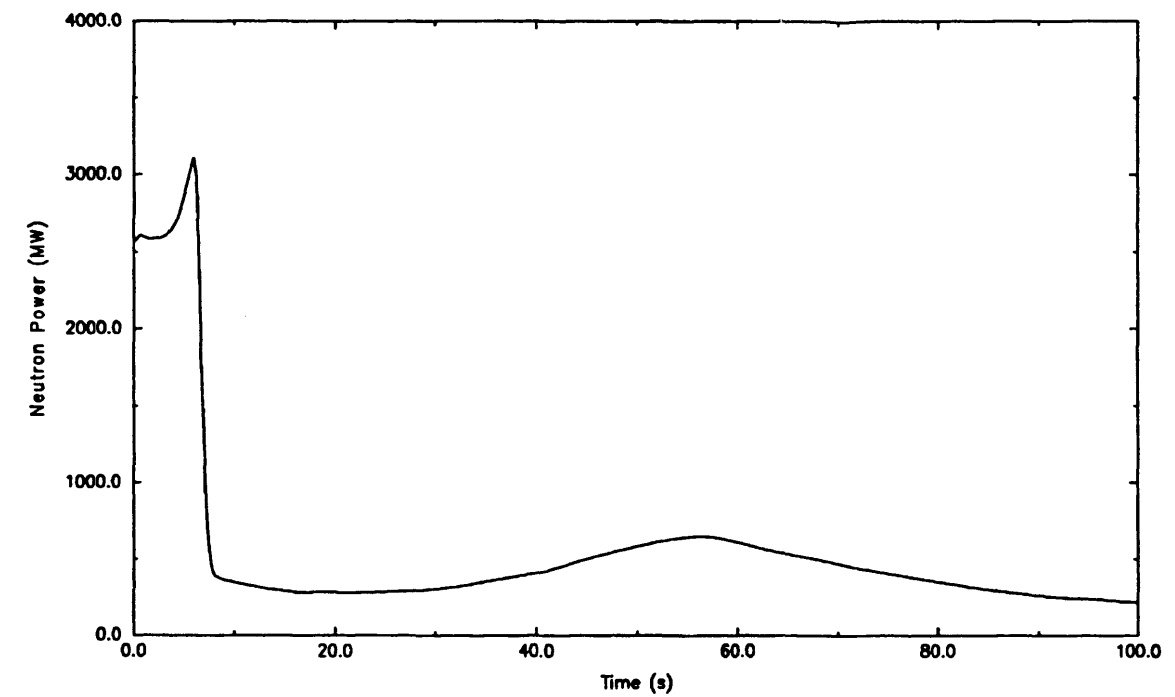
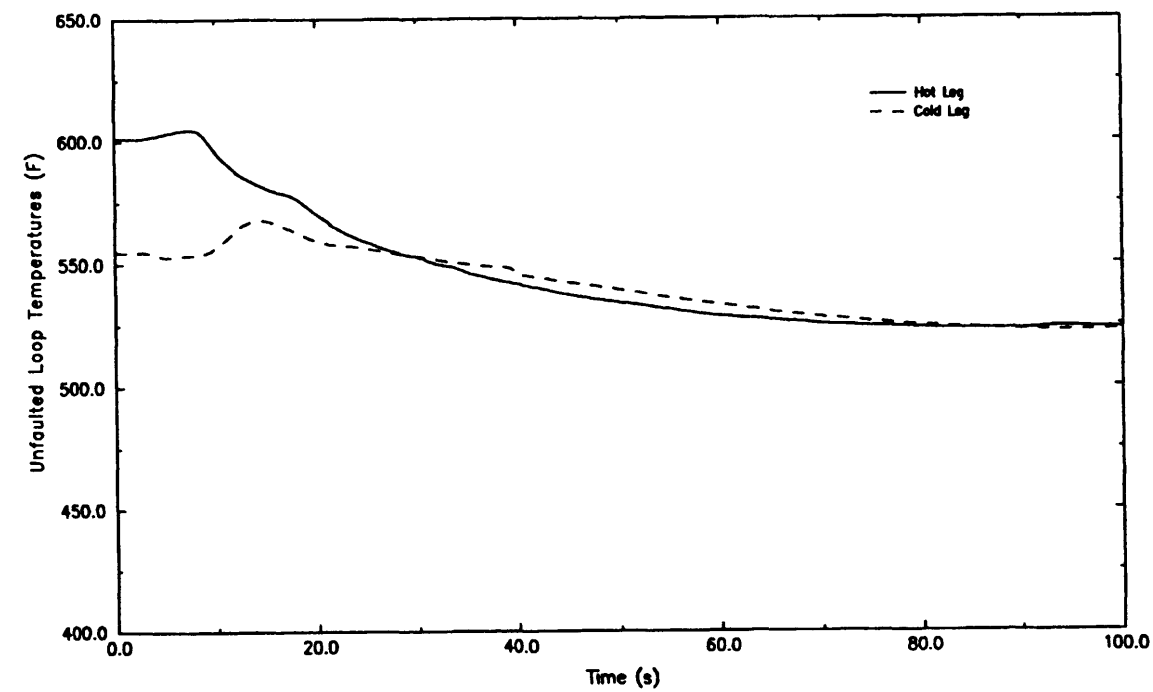
0.36%  $\Delta K/K$  Rod Drop from Rated  
Power with Automatic Runback to 60%  
Demand in 12s

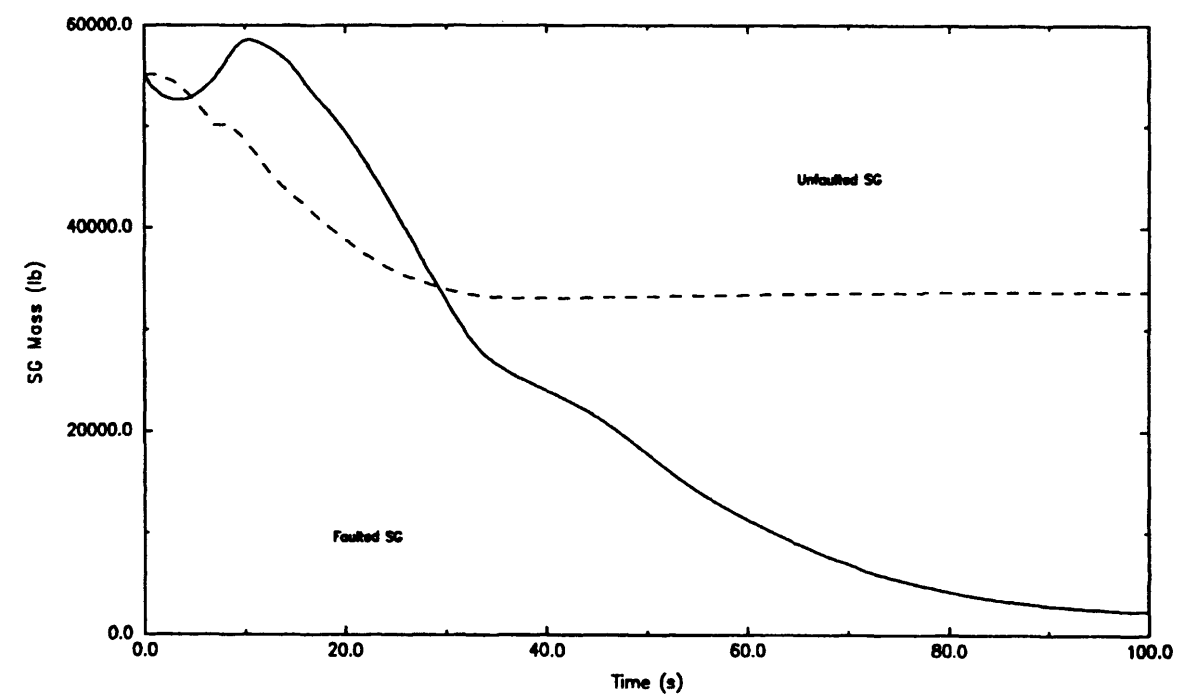
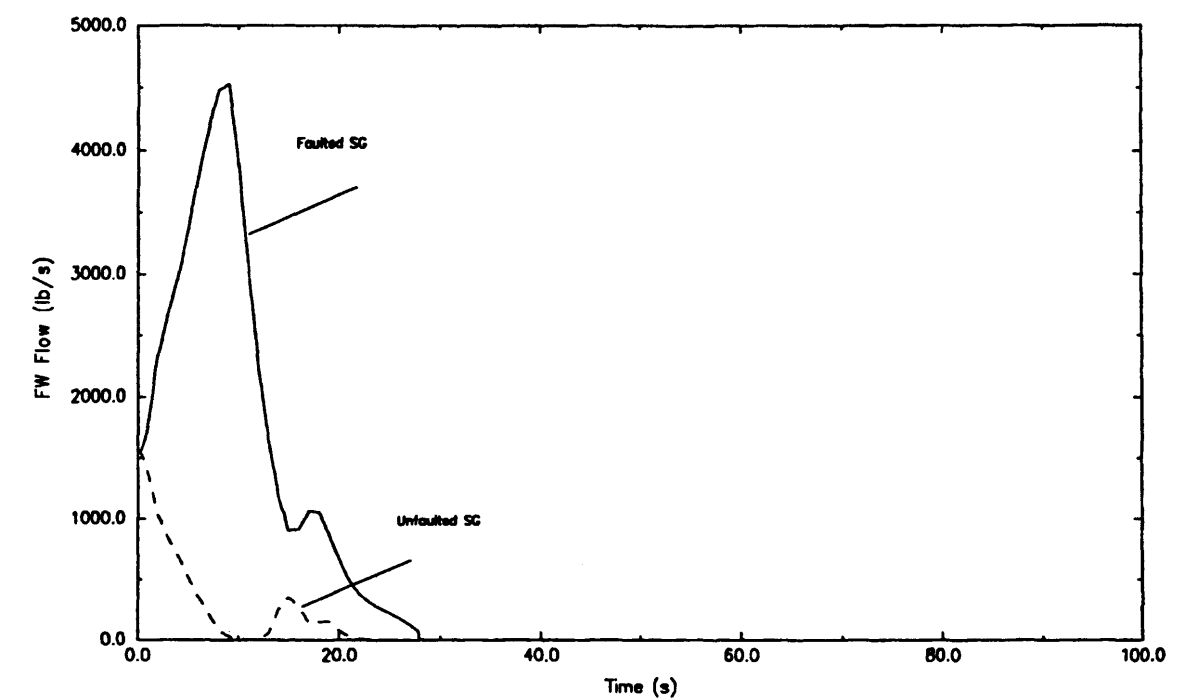
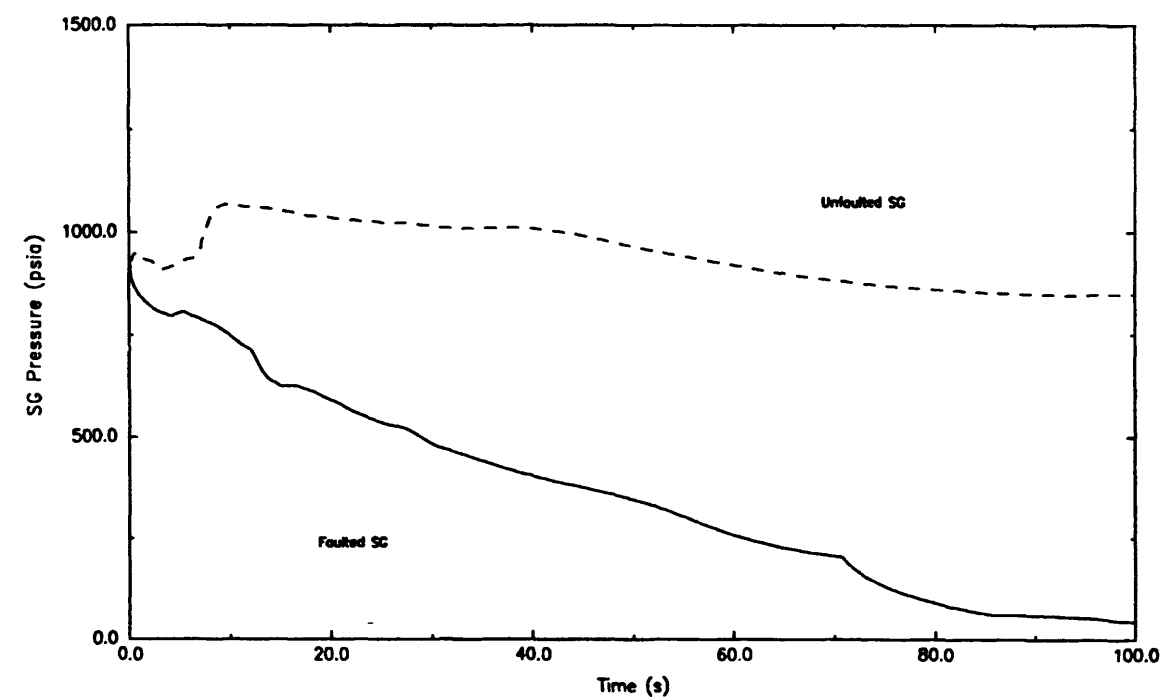
Update - 1

7/82

Fig. 14.1-21

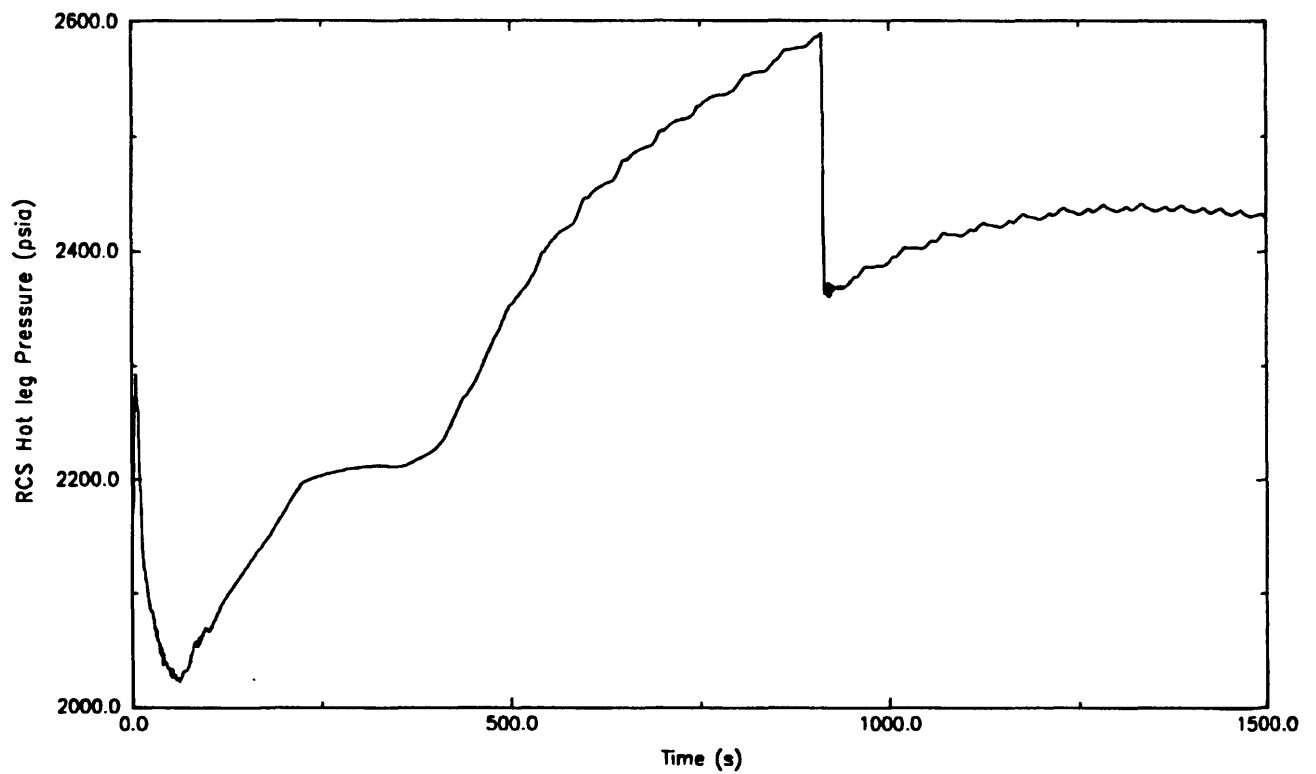
p. 14.FIG-21





p. 14.FIG-22B

<b>AmerGen</b>	UPDATE - 15
	4/00
TMI - UNIT 1	
DOUBLE-ENDED RUPTURE OF 24 IN. STEAM LINE BETWEEN STEAM GENERATOR AND STEAM STOP VALVE (WITH FEED WATER ISOLATION)	
CAD FILE: 6713R15.DWG	Fig. 14.1-22B



**AmerGen**

TMI - UNIT 1

UPDATE - 15

4/00

RCS HOT LEG PRESSURE (psia) RESPONSE  
FOR LOSS OF ALL AC POWER (STATION  
BLACKOUT)

CAD FILE: 6705R15.DWG

Fig. 14.1-23



# TMI UFSAR

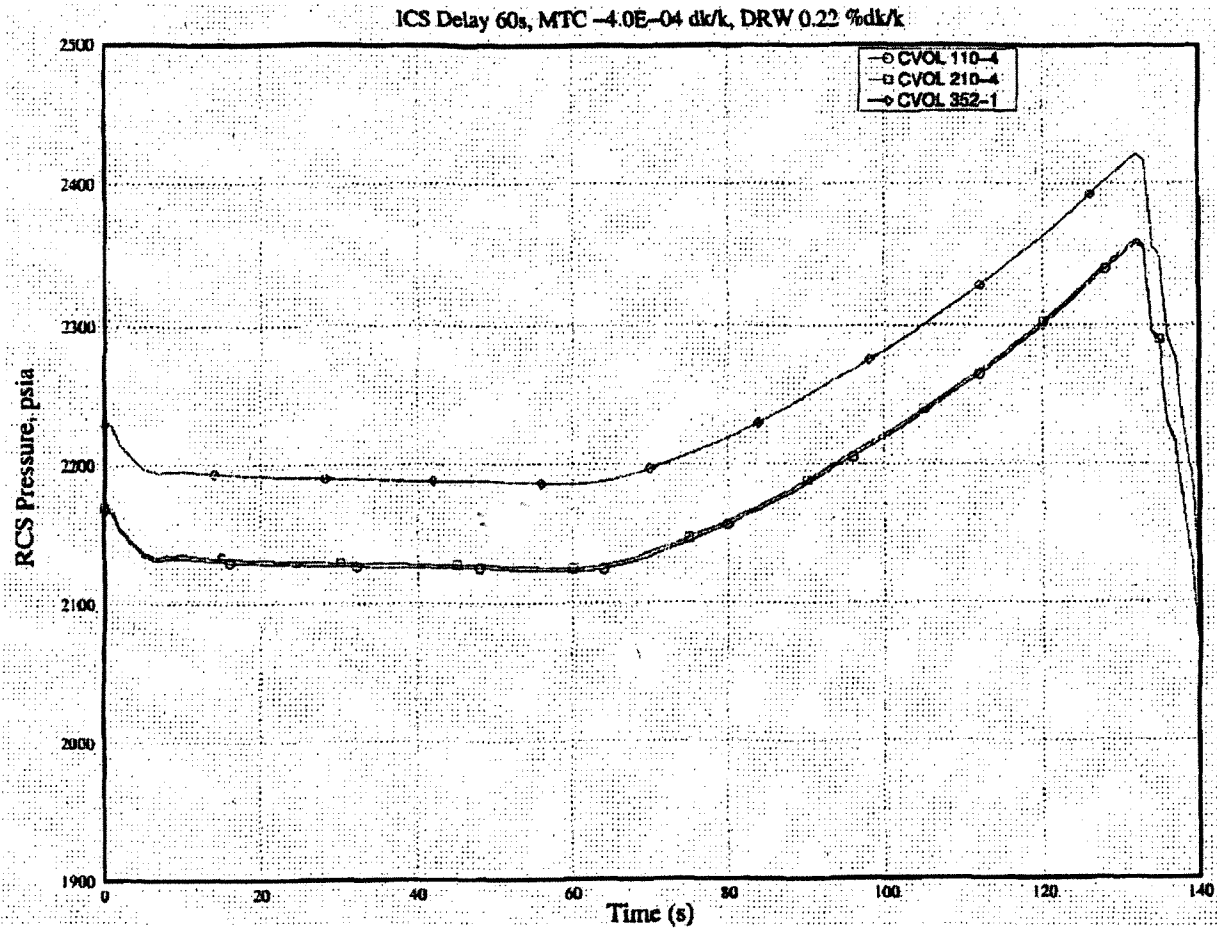


Figure 14.1-24 Dropped Rod w/ Bayonet Failure – RCS Pressure Response (NOTE 1)

## NOTES:

1. CVOL 110-4 and 210-4 represent hot leg pressures.  
CVOL 352-1 represents the core exit pressure.

p.14.FIG-23A

TMI - UNIT 1

REV. 19, APRIL 2008

Dropped Rod w/ Bayonet Failure  
RCS Pressure Response

FIGURE. 14.1-24

# TMI UFSAR

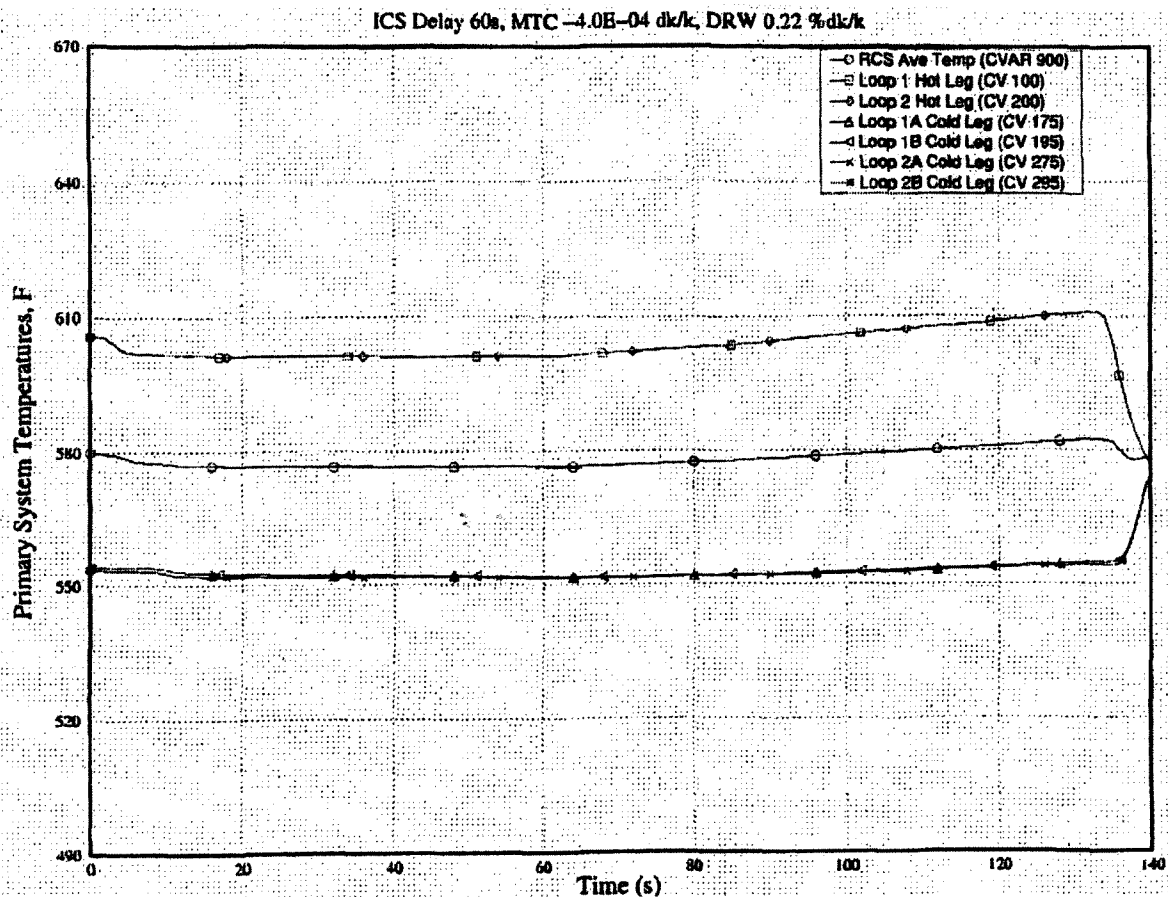


Figure 14.1-25 Dropped Rod w/ Bayonet Failure – Reactor Power Response

p.14.FIG-23B

TMI - UNIT 1

REV. 19, APRIL 2008

Dropped Rod w/ Bayonet Failure  
Reactor Power Response

FIGURE. 14.1-25

# TMI UFSAR

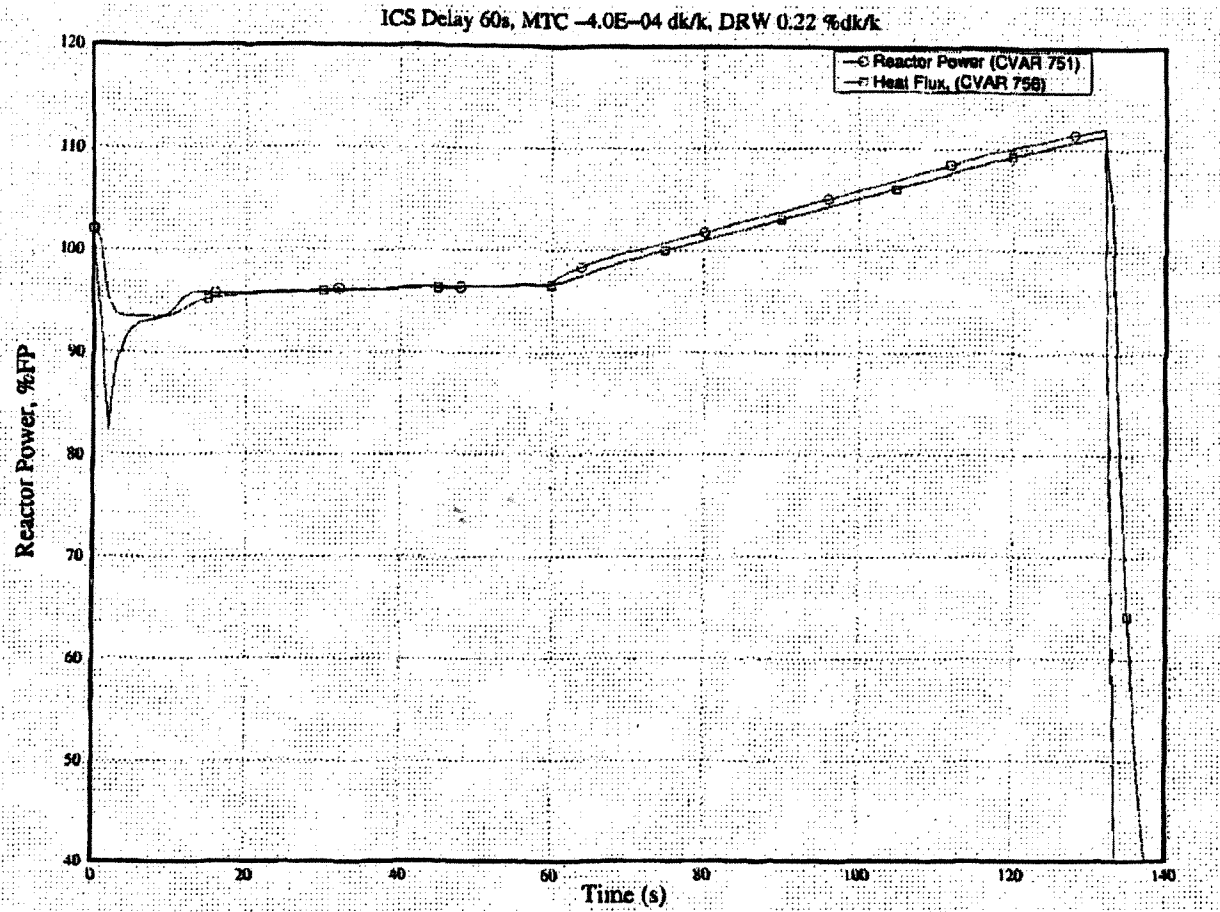


Figure 14.1-26 Dropped Rod w/ Bayonet Failure – Primary Temperature Response

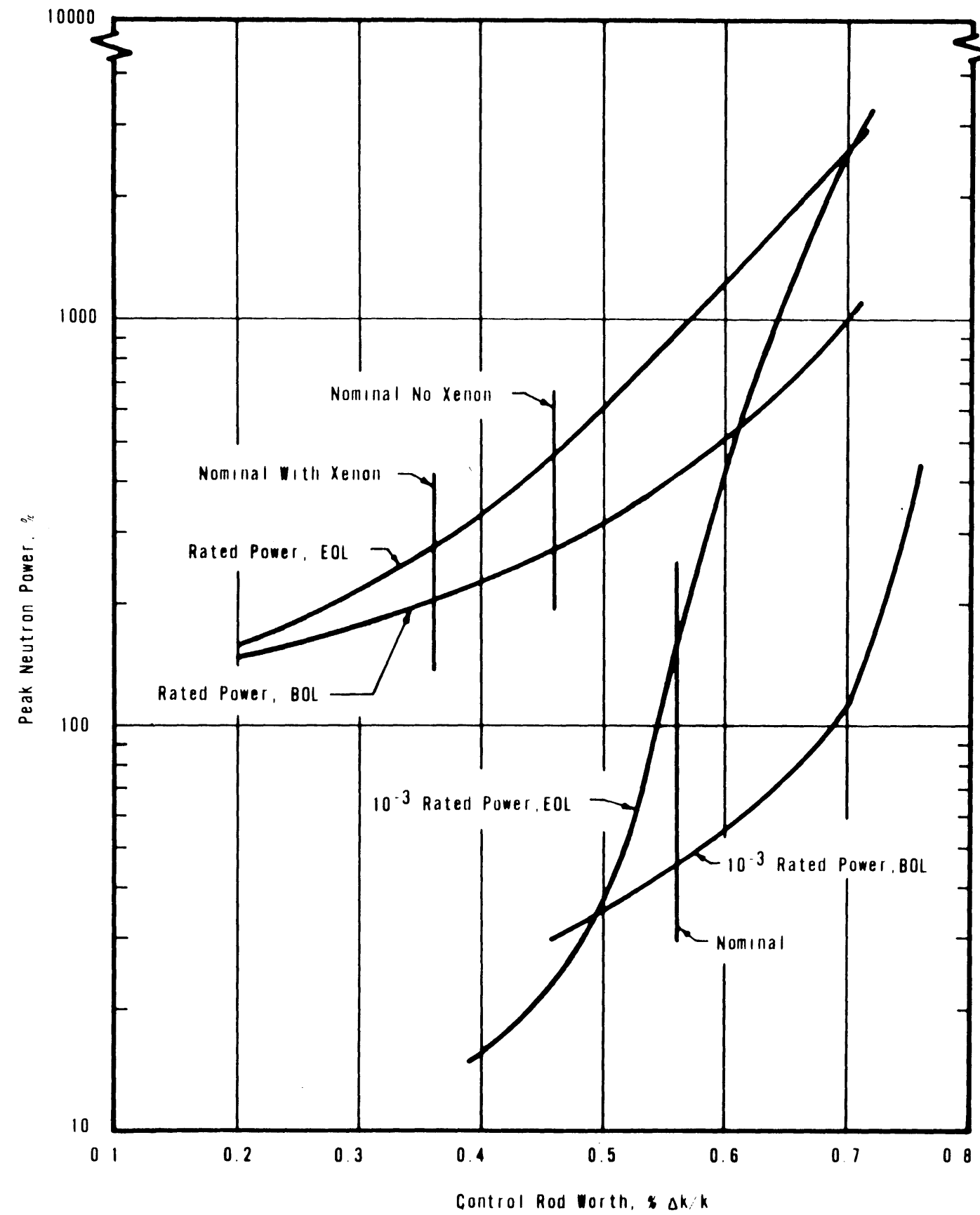
p.14.FIG-23C

TMI - UNIT 1

REV. 19, APRIL 2008

Dropped Rod w/ Bayonet Failure  
Primary Temperature Response

FIGURE. 14.1-26



**GPU Nuclear**

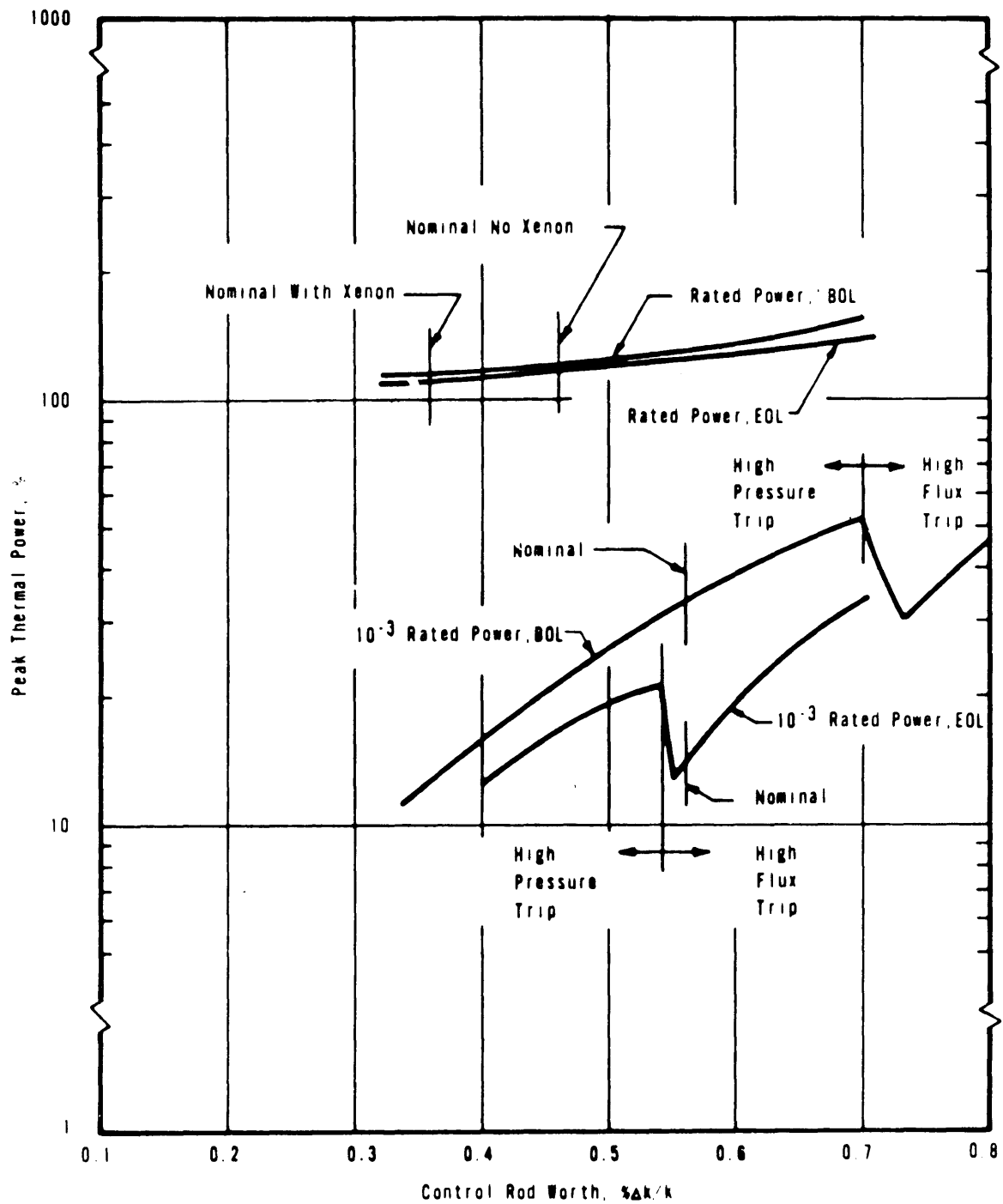
**TMI Unit-1**

Peak Neutron Power Variation With  
Ejected Control Rod Worth

Update -1

7/82

Fig. 14.2-1



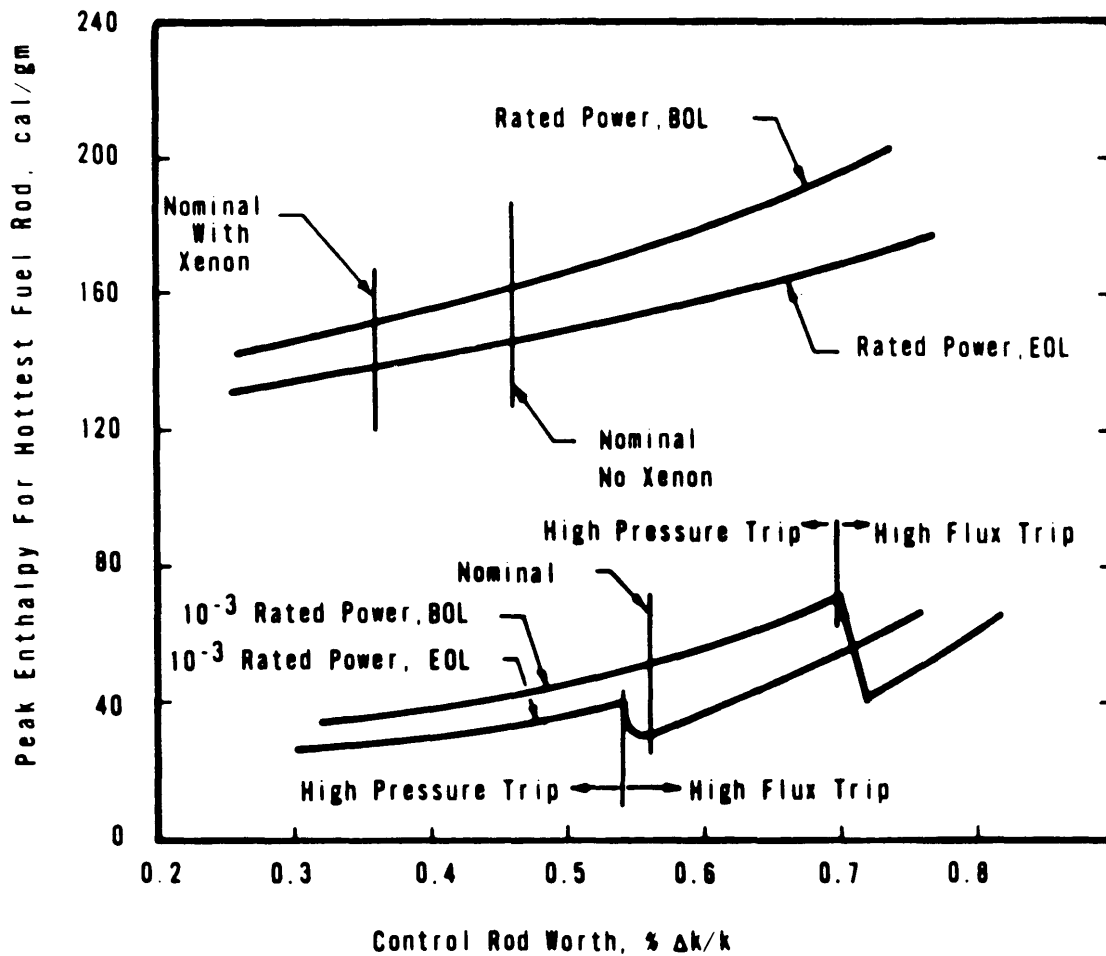
**GPU Nuclear**

Update -1

TMI Unit-1

7/82

Peak Thermal Power as a Function  
of Ejected Control Rod Worth



**GPU Nuclear**

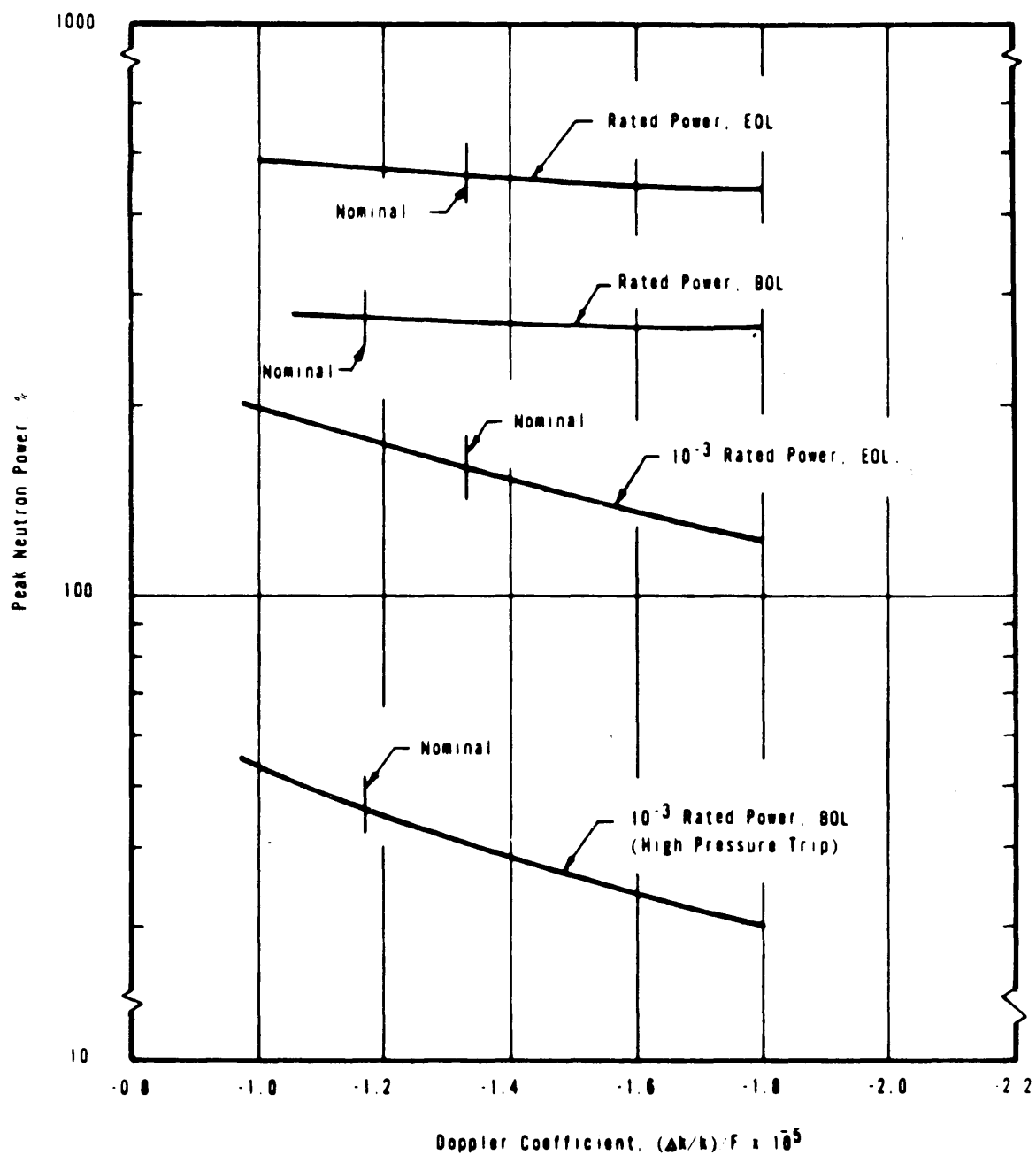
**TMI Unit-1**

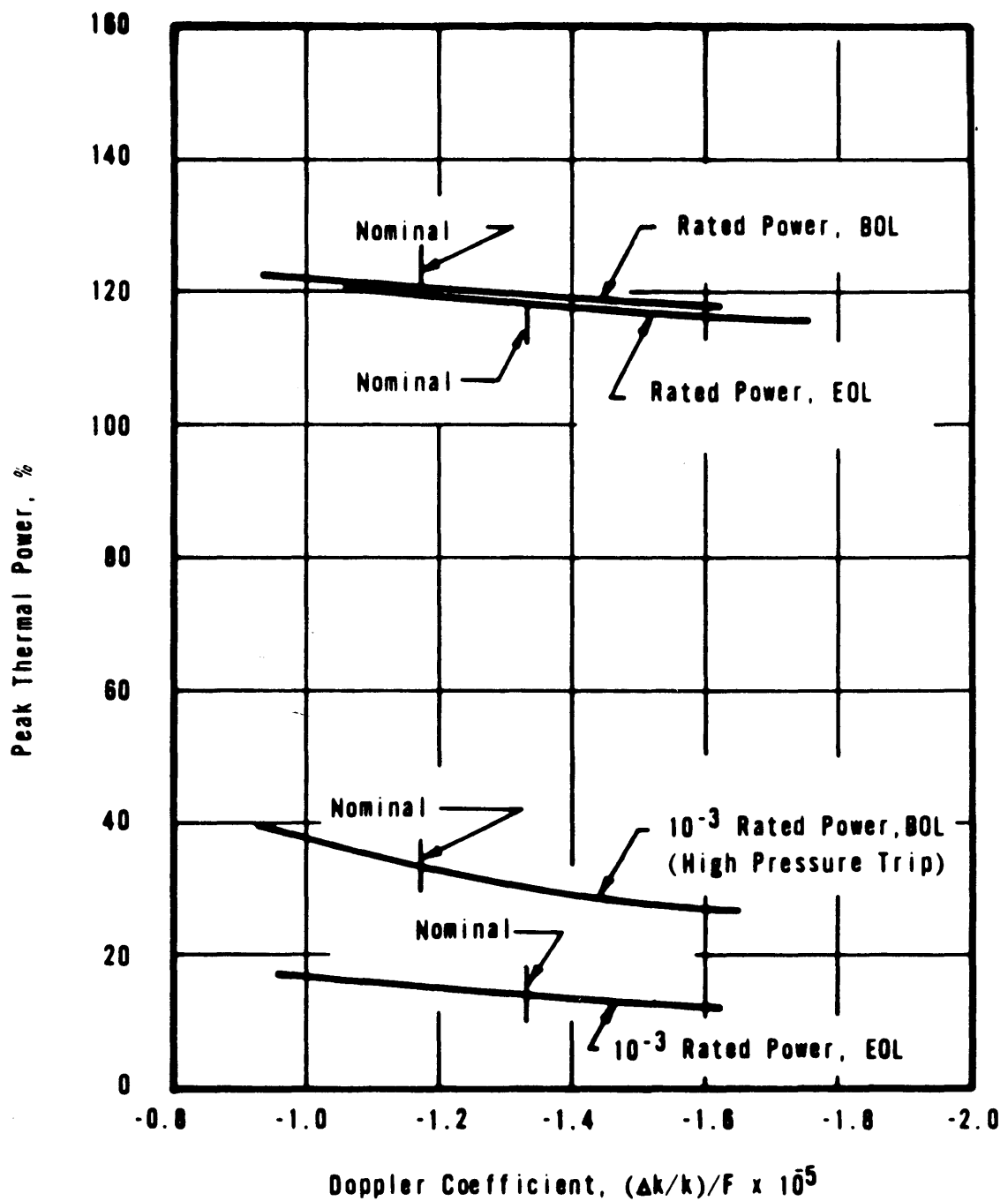
Peak Enthalpy of Hottest Fuel Rod  
Versus Ejected Control Rod Worth

Update -1

7/82

Fig. 14.2-3





**GPU Nuclear**

Update - 1

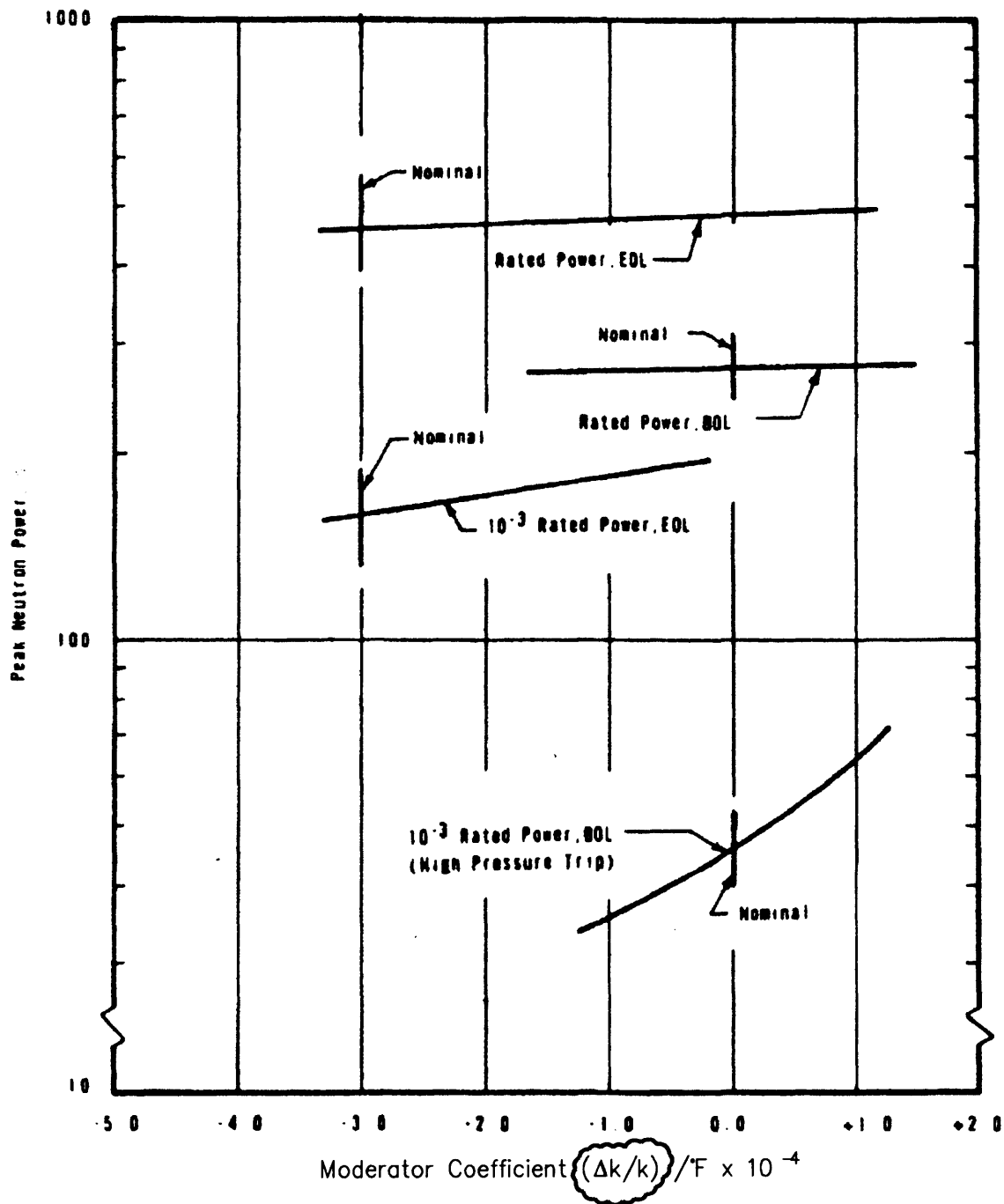
**TMI Unit-1**

7/82


Effect on Peak Thermal Power of Varying the Doppler Coefficient for an Ejected Rod Worth of 0.56%  $\Delta K/K$  at  $10^{-3}$  Rated Power and 0.46%  $\Delta K/K$  at Rated Power

Fig. 14.2-5





p. 14.FIG-28



TMI - UNIT I

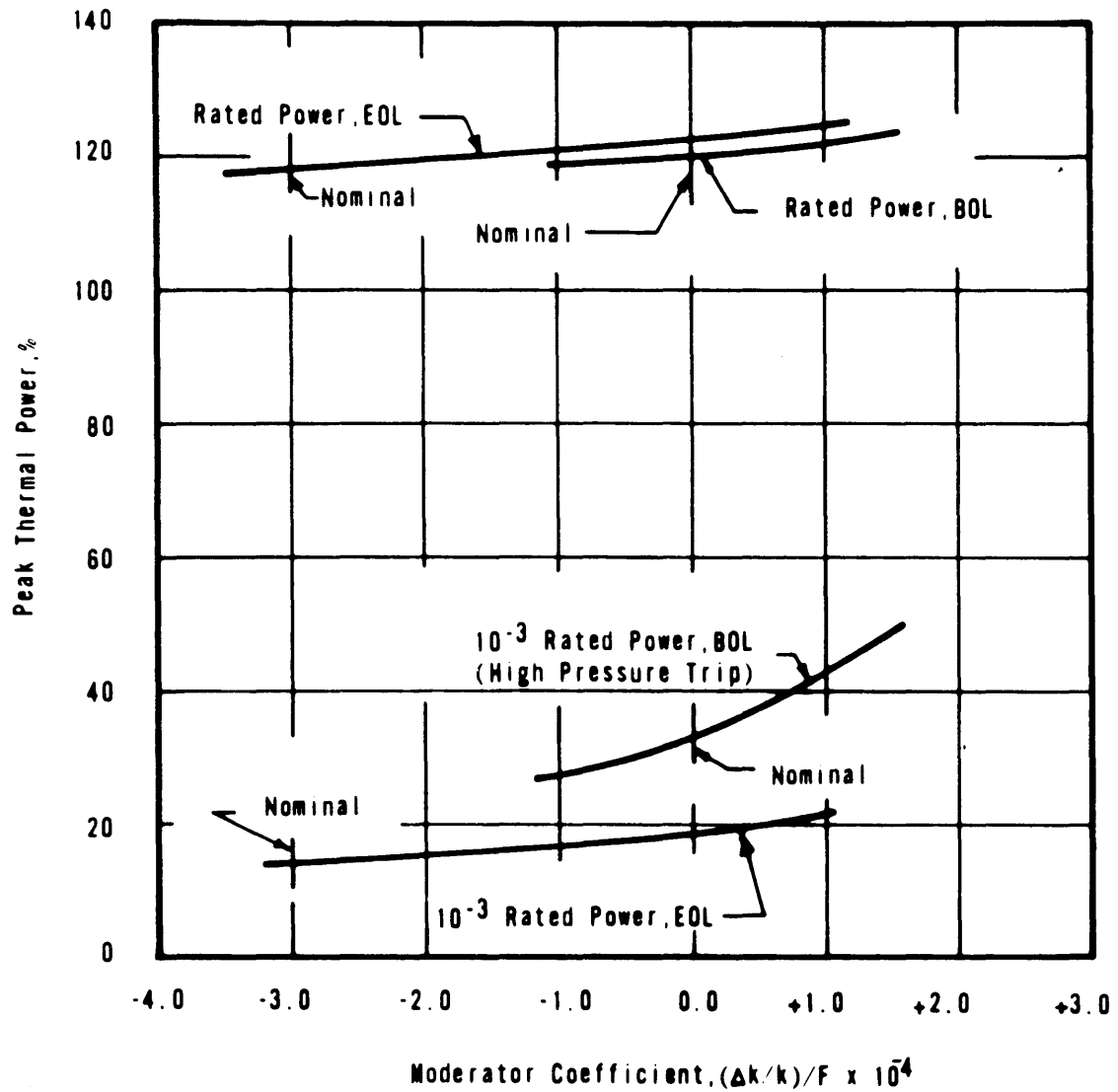
UPDATE - 14

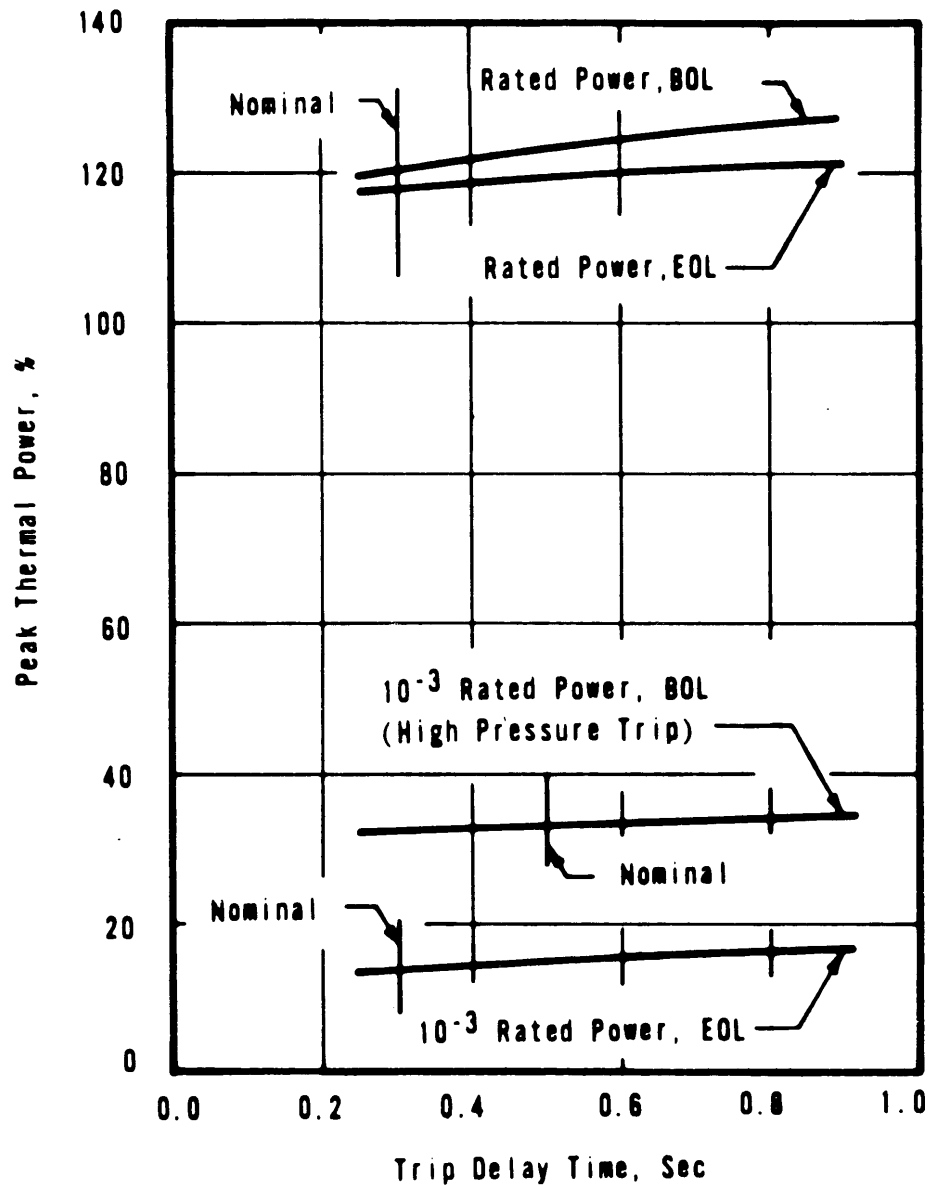
04/98

Effect on Peak Neutron Power of Varying the Moderator Coefficient for an Ejected Rod Worth of 0.56%  $\Delta k/k$  at  $10^{-3}$  Rated Power and 0.46%  $\Delta k/k$  at Rated Power

CAD FILE: 6558RI4.DWG

FIG 14.2-6





**GPU Nuclear**

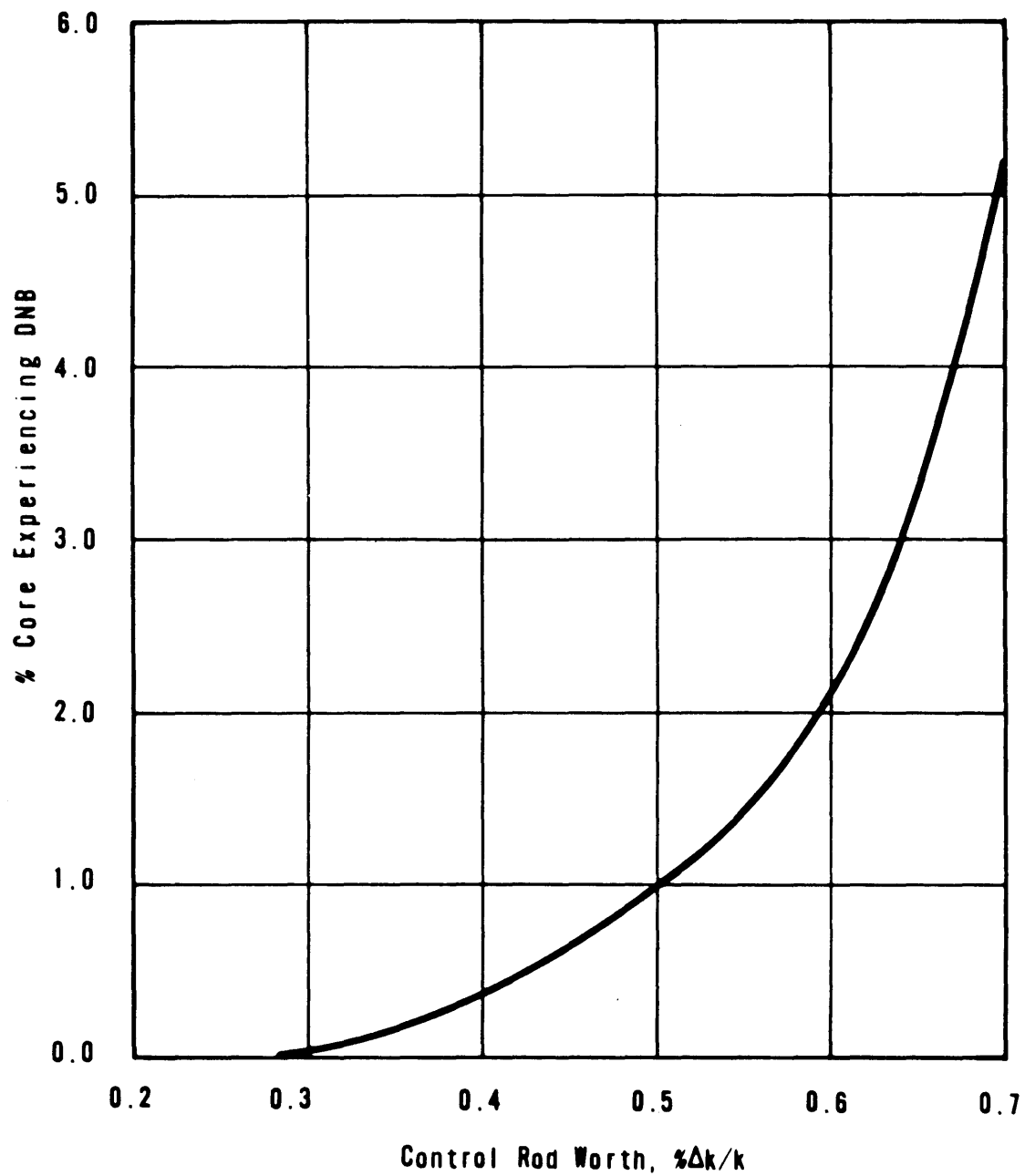
Update -1

TMI Unit-1

7/82

Effect on Peak Thermal Power of Varying the Trip Delay Time for an Ejected Rod Worth of 0.56%  $\Delta K/K$  at 10<sup>-3</sup> Rated Power and 0.46%  $\Delta K/K$  at Rated Power

Fig. 14.2-8



**GPU Nuclear**

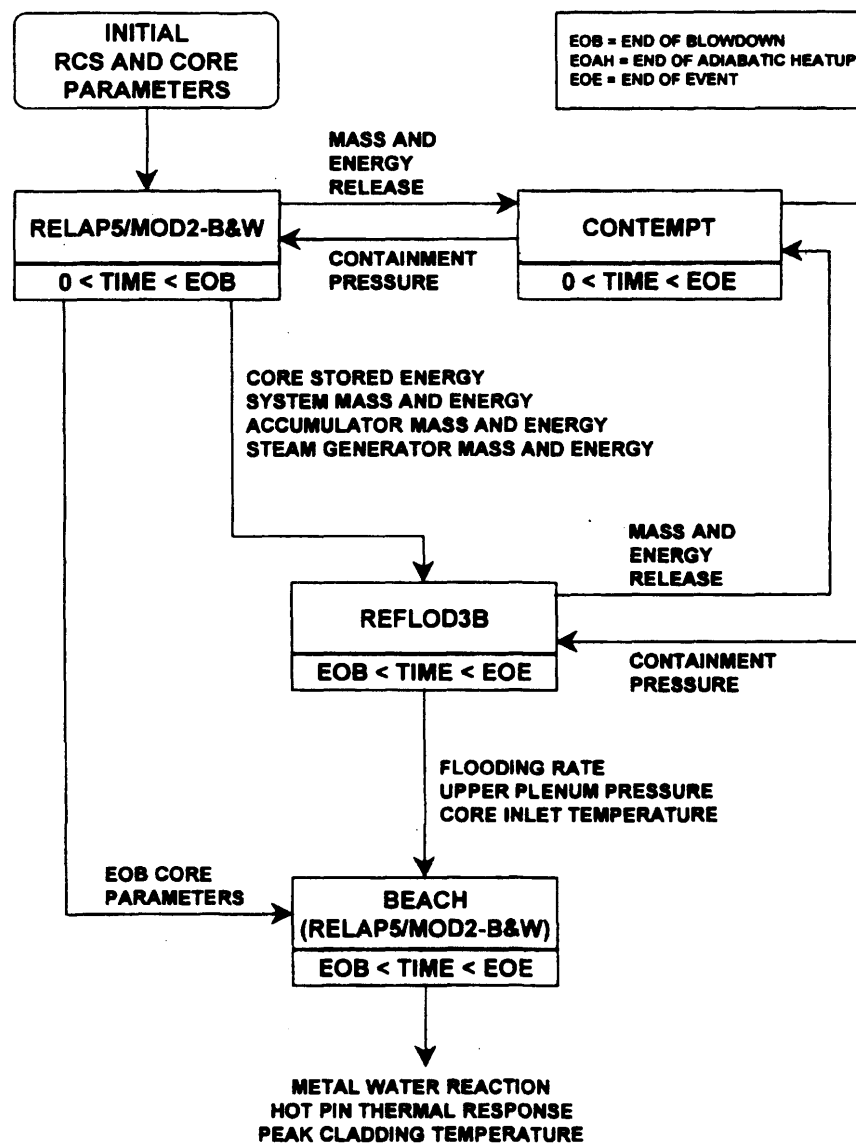
Update - 1


**TMI Unit-1**

7/82

Percent Core Experiencing DNB as a  
Function of Ejected Control Rod  
Worth at Rated Power, BOL

Fig. 14.2-9





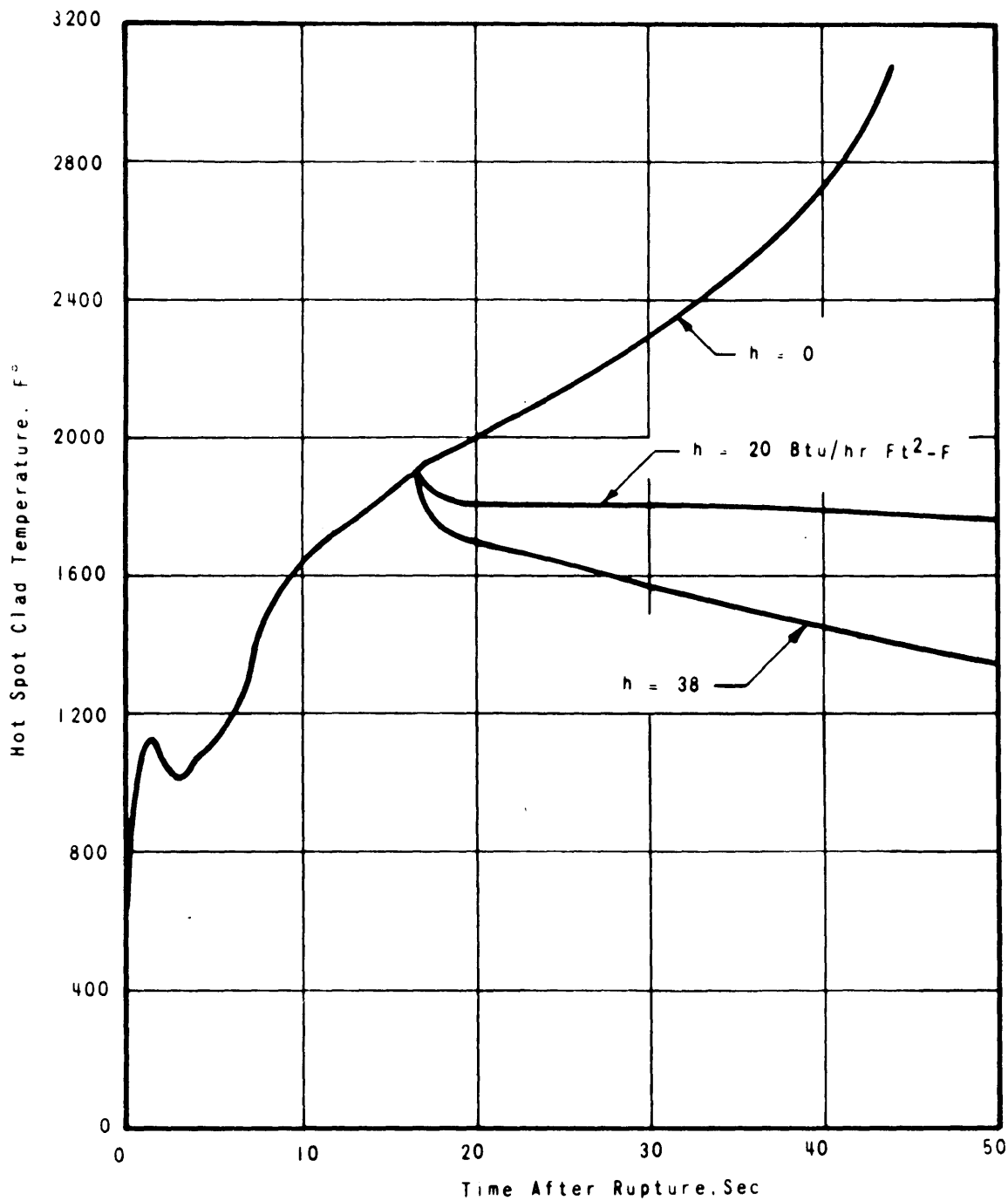
TMI - UNIT I

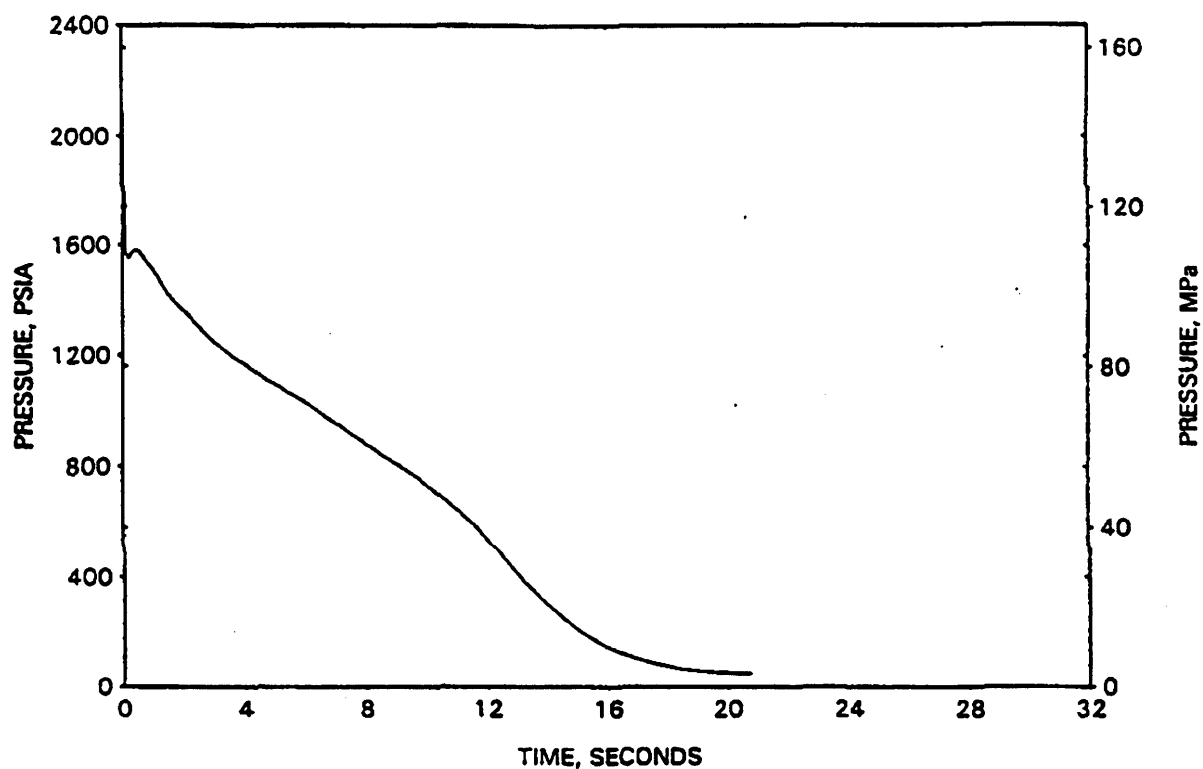
UPDATE - 14  
4/98

**LARGE BREAK ANALYSIS CODE INTERFACES**

CAD FILE: 6573RI4.DWG

FIG 14.2-10





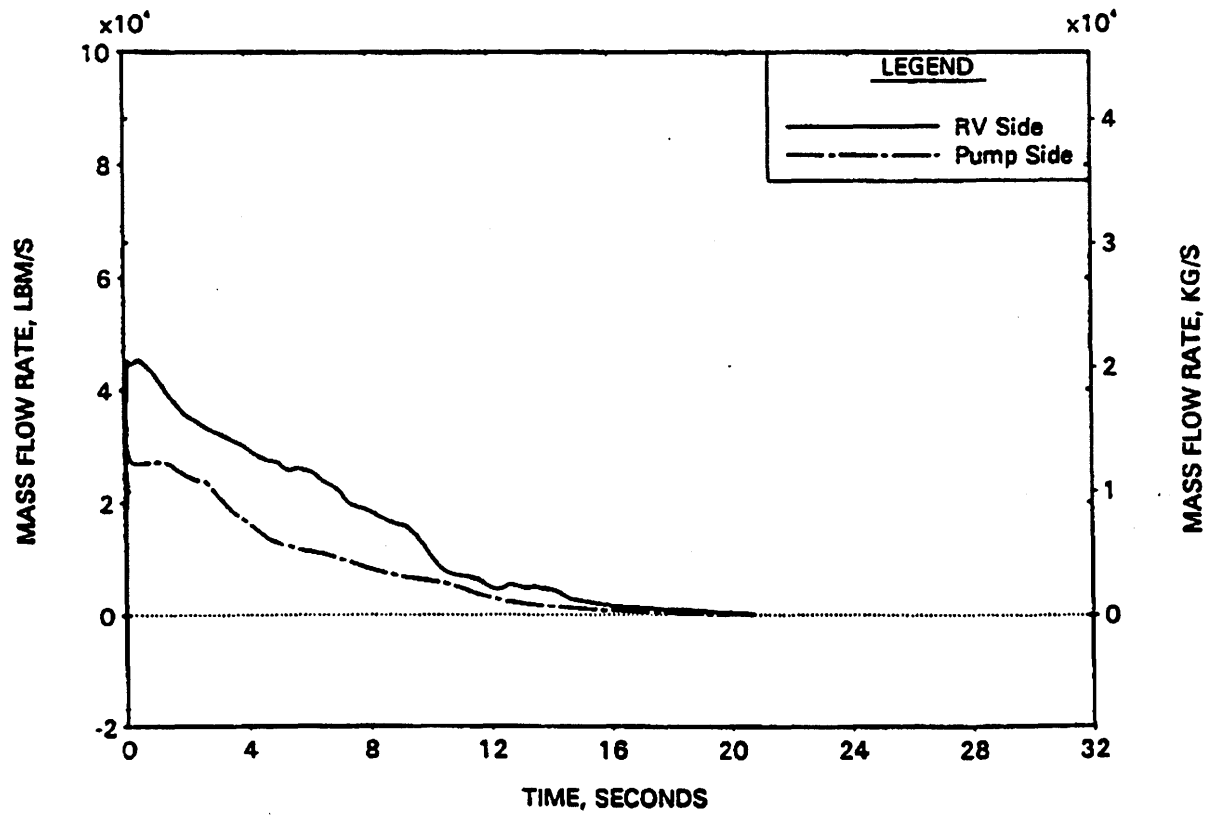
**AmerGen**  
TMI - UNIT 1

UPDATE - 15  
4/00

LBLOCA LIMIT CASE (BOL) - REACTOR  
VESSEL UPPER PLENUM PRESSURE

CAD FILE: 6574R15.DWG

FIG 14.2-12



**AmerGen**

TMI - UNIT 1

UPDATE - 15

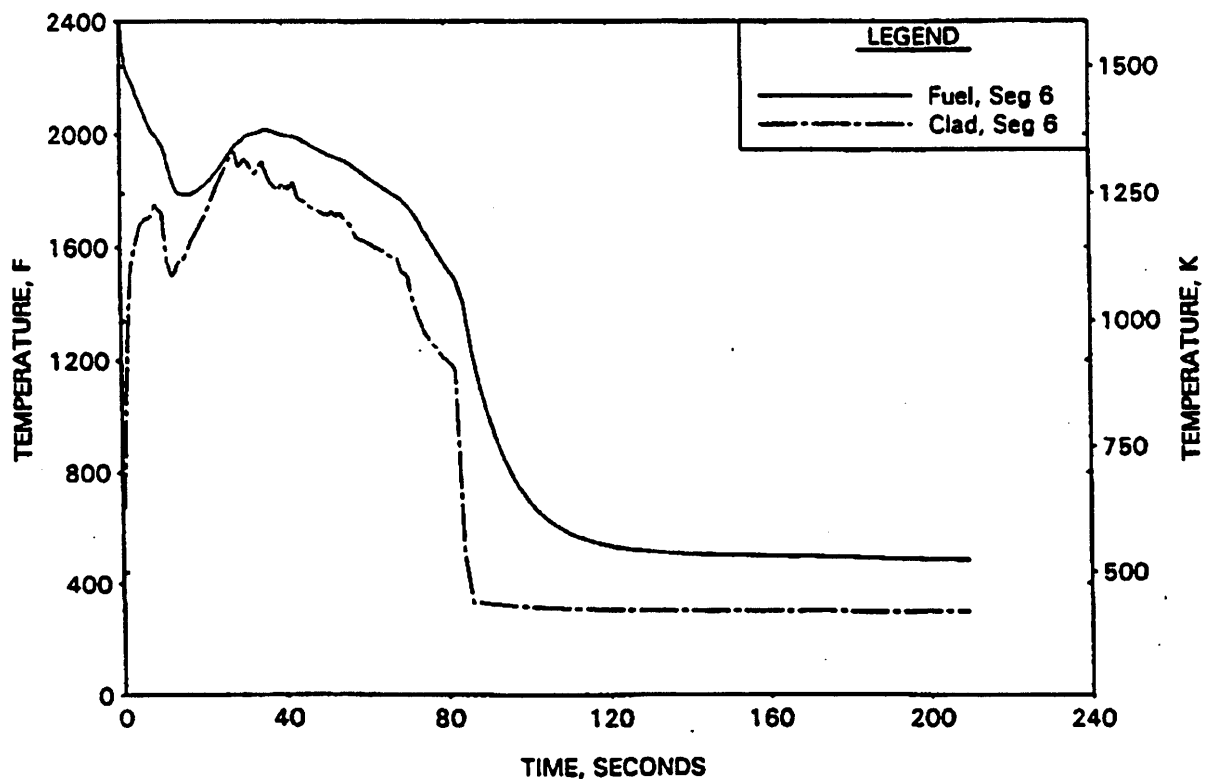
4/00

LBLOCA LIMIT CASE (BOL) - BREAK MASS  
FLOW RATE

CAD FILE: 6575R15.DWG

FIG 14.2-13





**AmerGen**

TMI - UNIT 1

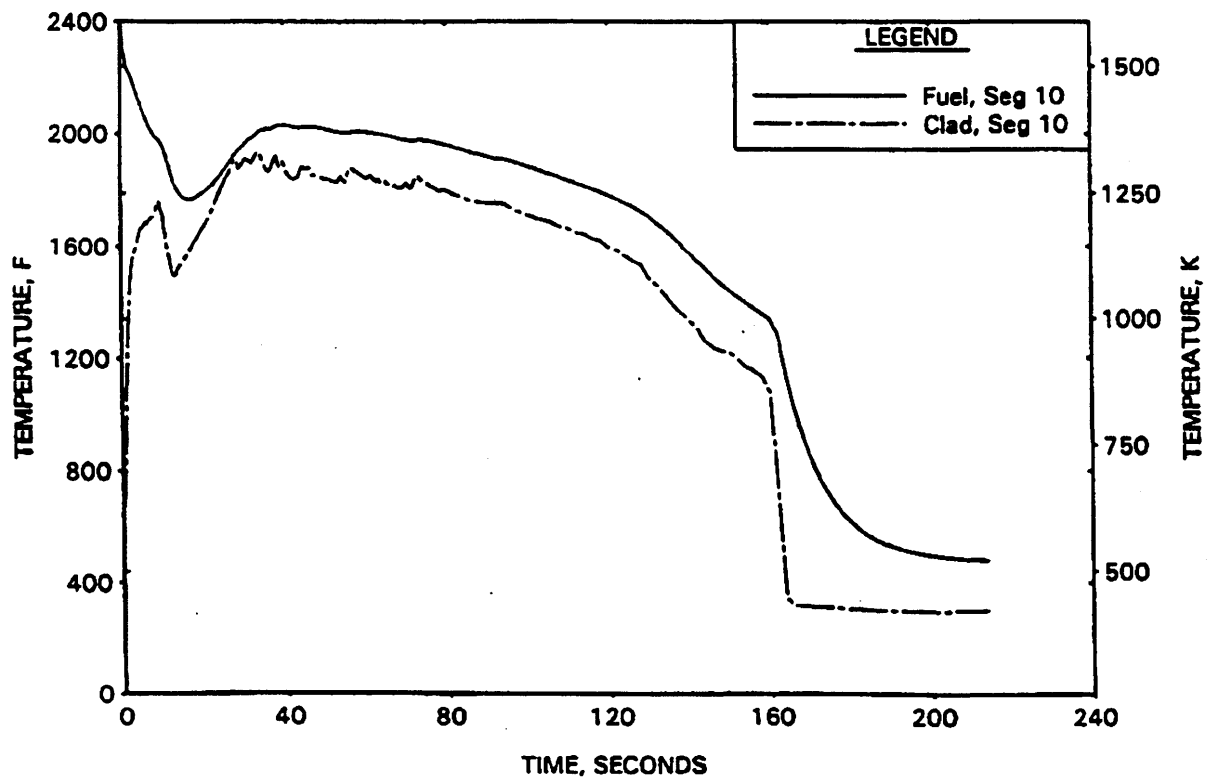
UPDATE - 15

4/00

LBLOCA LIMIT CASE (BOL - 2.506-FT) -  
HOT CHANNEL FUEL AND CLAD  
TEMPERATURE AT PEAK UNRUPTURED  
LOCATION

CAD FILE: 6576R15.DWG

FIG 14.2-14



**AmerGen**

TMI - UNIT 1

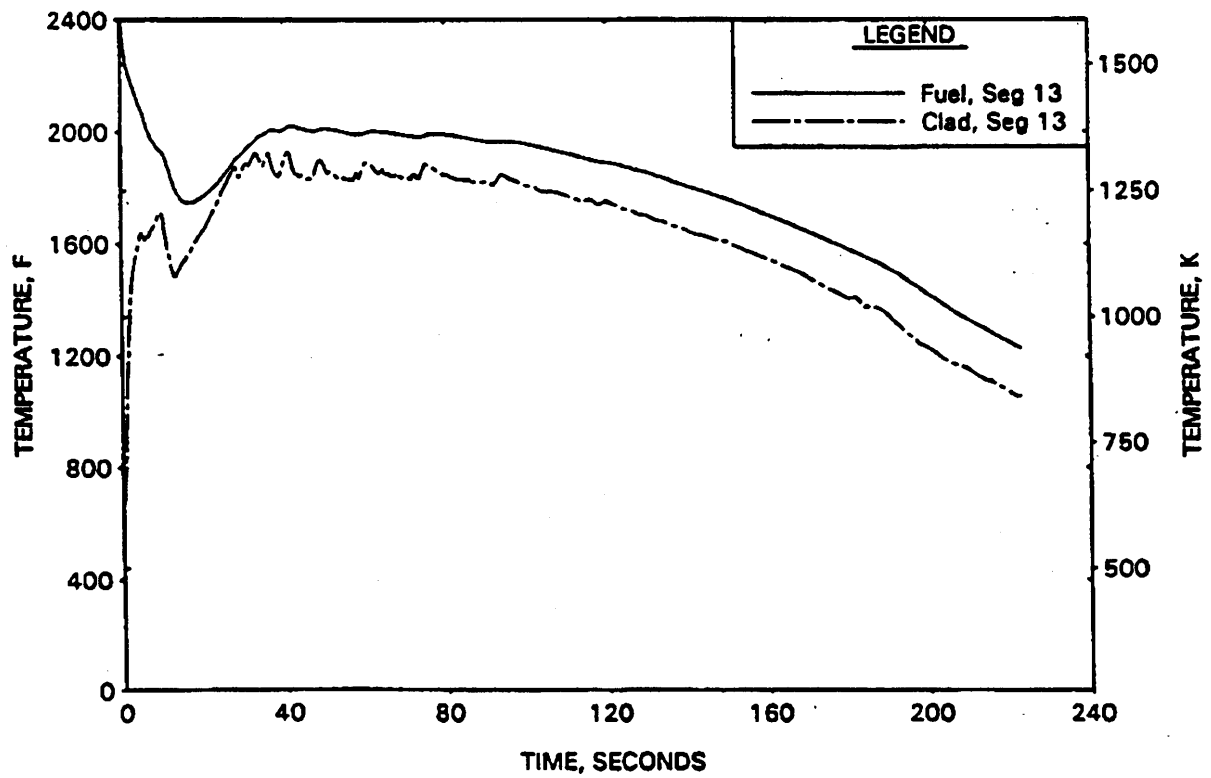
UPDATE - 15

4/00

LBLOCA LIMIT CASE (BOL - 4.264-FT) -  
HOT CHANNEL FUEL AND CLAD  
TEMPERATURE AT PEAK UNRUPTURED  
LOCATION

CAD FILE: 6577R15.DWG

FIG 14.2-15



**AmerGen**

TMI - UNIT 1

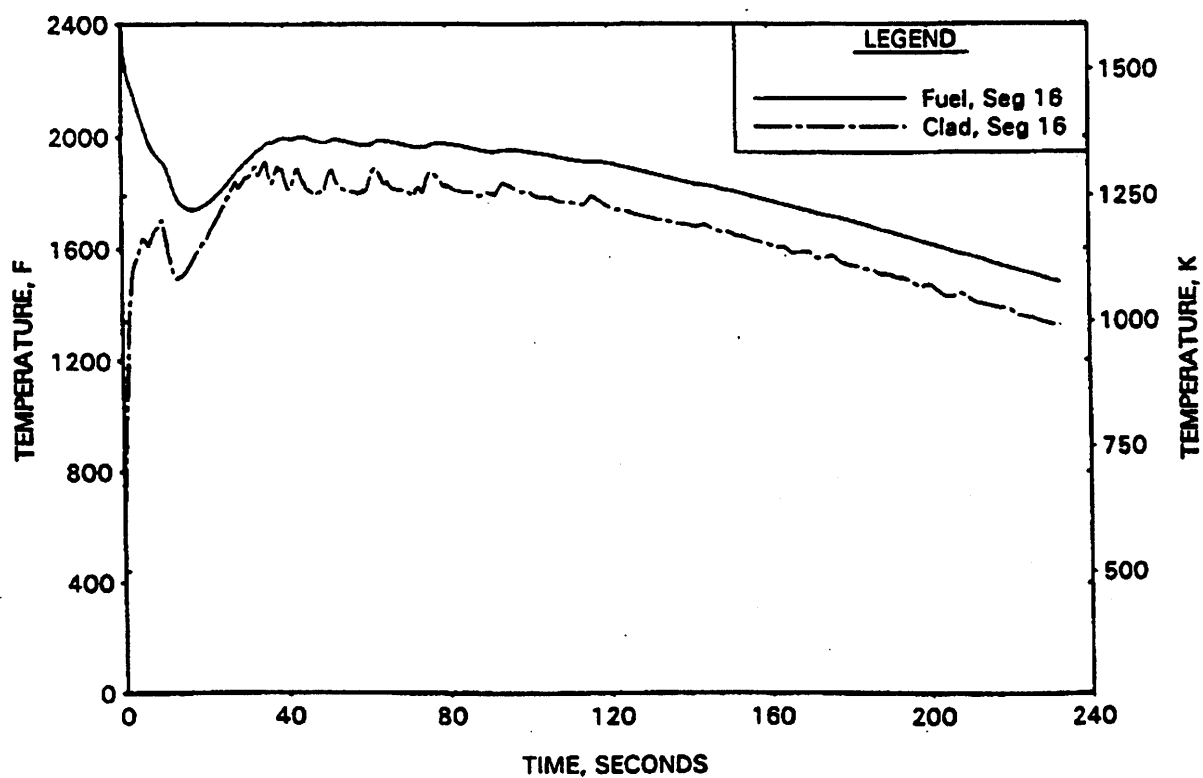
UPDATE - 15

4/00

LBLOCA LIMIT CASE (BOL - 6.021-FT) -  
HOT CHANNEL FUEL AND CLAD  
TEMPERATURE AT PEAK UNRUPTURED  
LOCATION

CAD FILE: 6578R15.DWG

FIG 14.2-16



**AmerGen**

TMI - UNIT 1

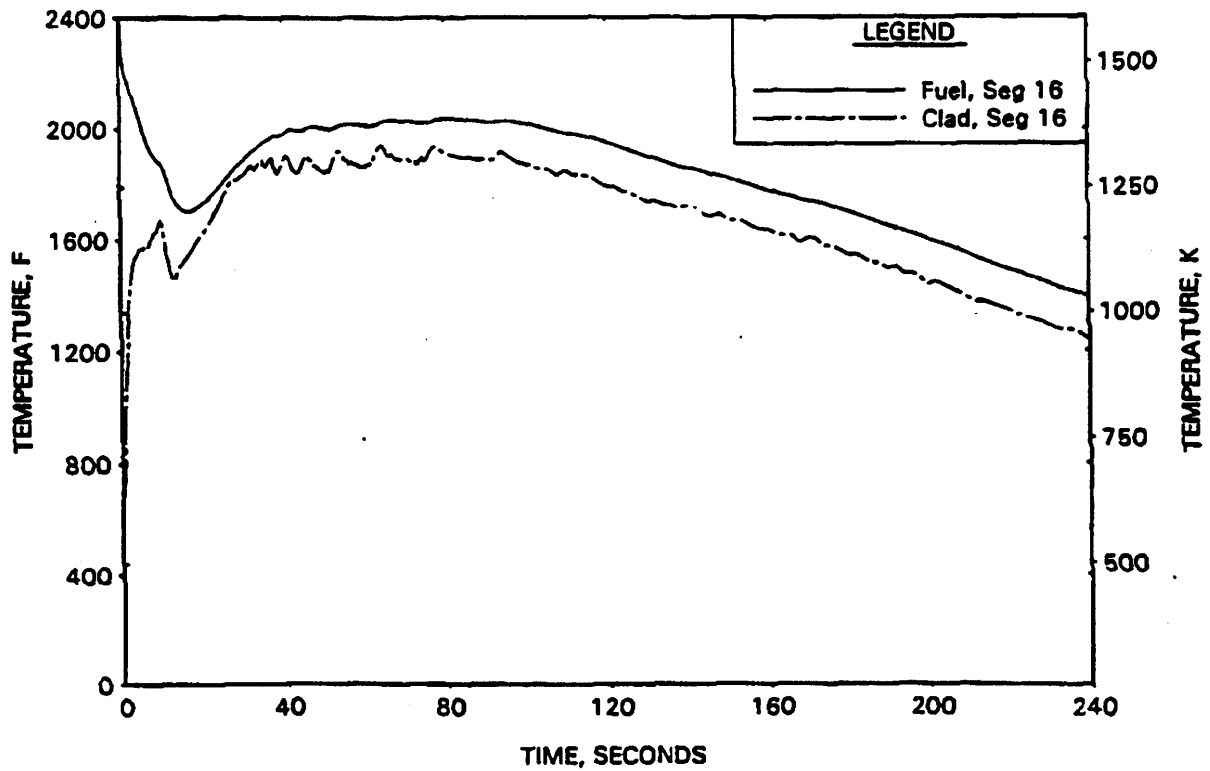
UPDATE - 15

4/00

LBLOCA LIMIT CASE (BOL - 7.779-FT) -  
HOT CHANNEL FUEL AND CLAD  
TEMPERATURE AT PEAK UNRUPTURED  
LOCATION

CAD FILE: 6579R15.DWG

FIG 14.2-17



**AmerGen**

TMI - UNIT 1

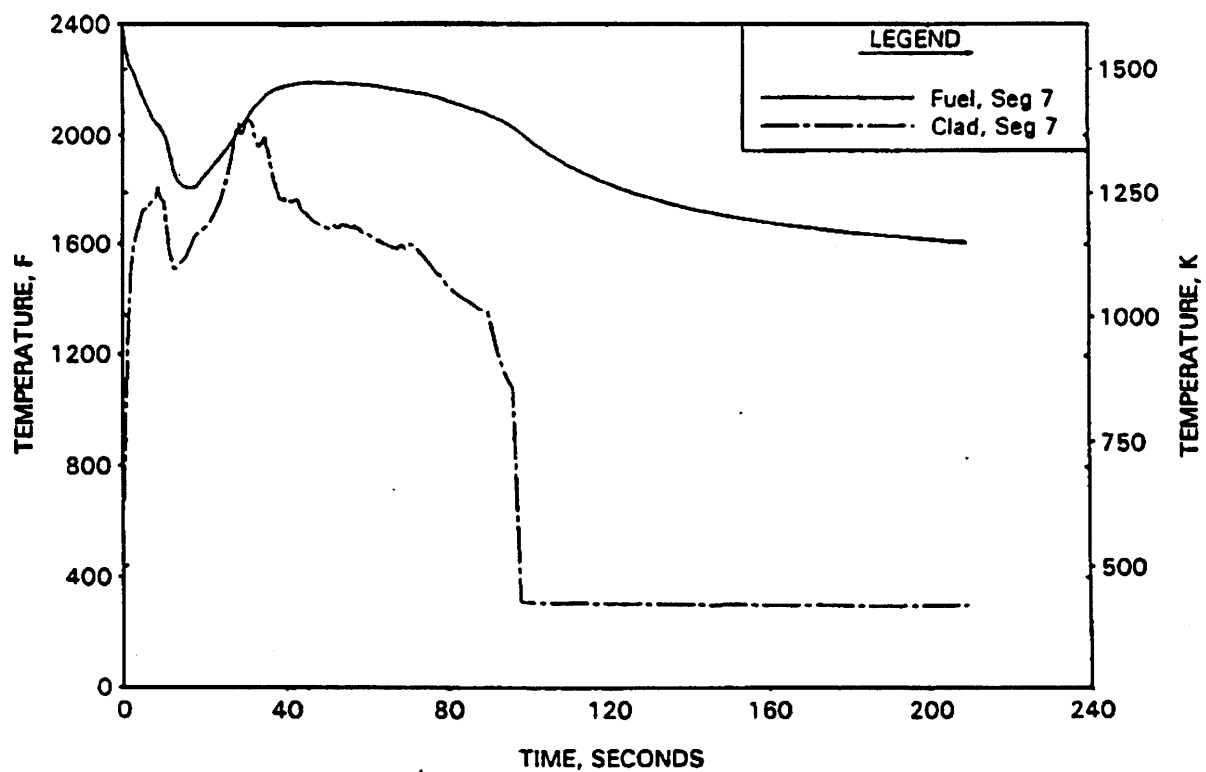
UPDATE - 15

4/00

LBLOCA LIMIT CASE (BOL - 9.536-FT) -  
HOT CHANNEL FUEL AND CLAD  
TEMPERATURE AT PEAK UNRUPTURED  
LOCATION

CAD FILE: 6580R15.DWG

FIG 14.2-18



**AmerGen**

TMI - UNIT 1

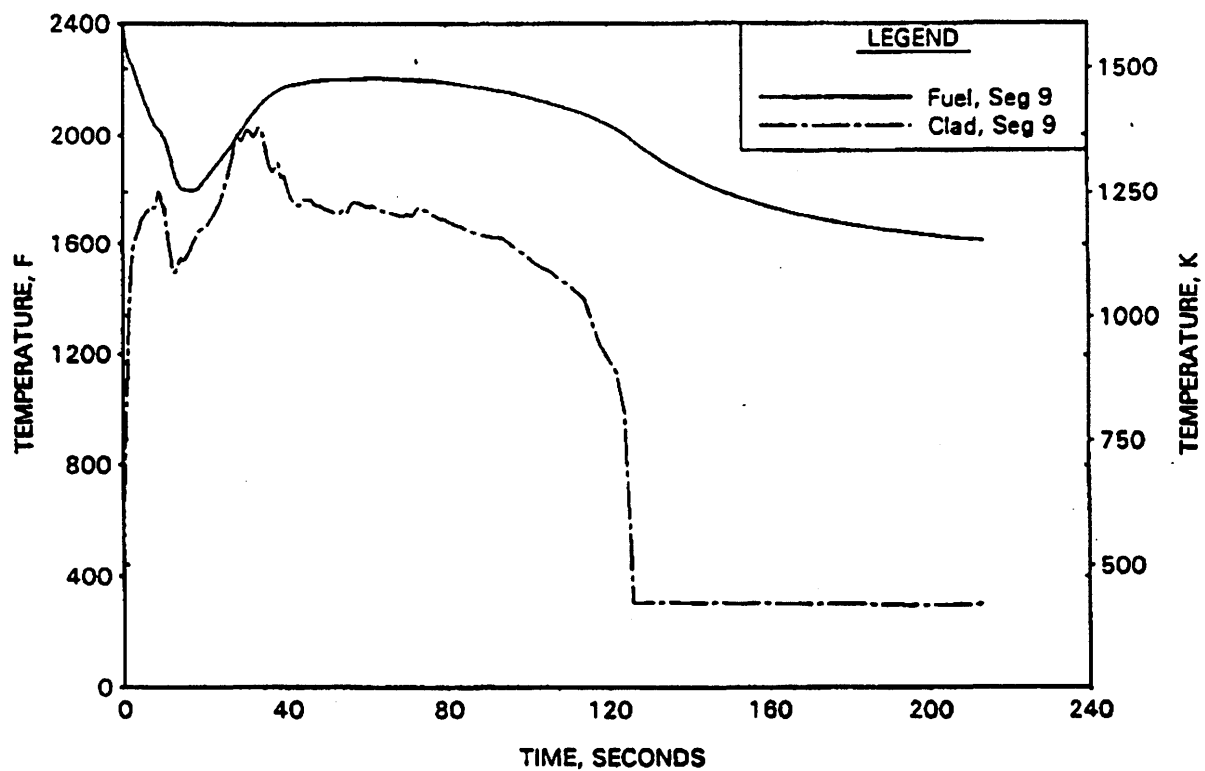
UPDATE - 15

4/00

LBLOCA LIMIT CASE (BOL - 2.506-FT) -  
HOT CHANNEL FUEL AND CLAD  
TEMPERATURE AT PEAK UNRUPTURED  
LOCATION

CAD FILE: 6581R15.DWG

FIG 14.2-19



**AmerGen**

TMI - UNIT 1

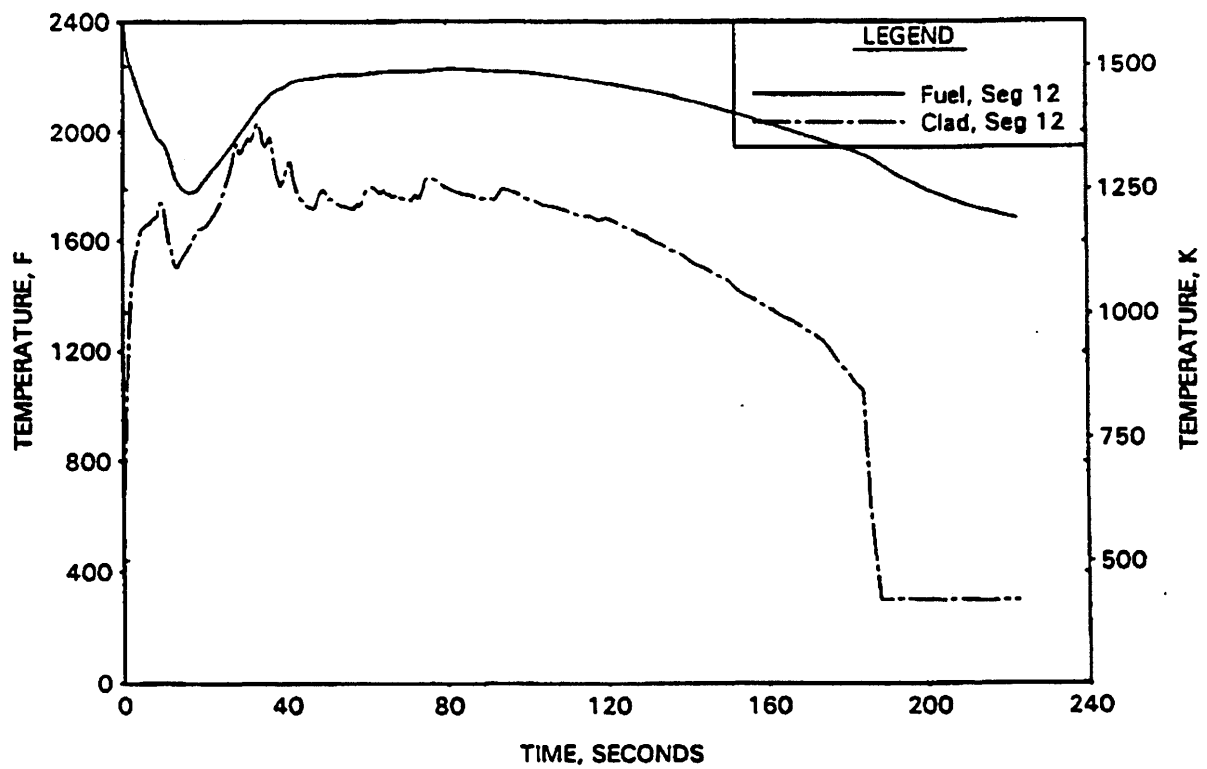
UPDATE - 15

4/00

LBLOCA LIMIT CASE (BOL - 4.264-FT) -  
HOT CHANNEL FUEL AND CLAD  
TEMPERATURE AT PEAK RUPTURED  
LOCATION

CAD FILE: 6582R15.DWG

FIG 14.2-20



**AmerGen**

TMI - UNIT 1

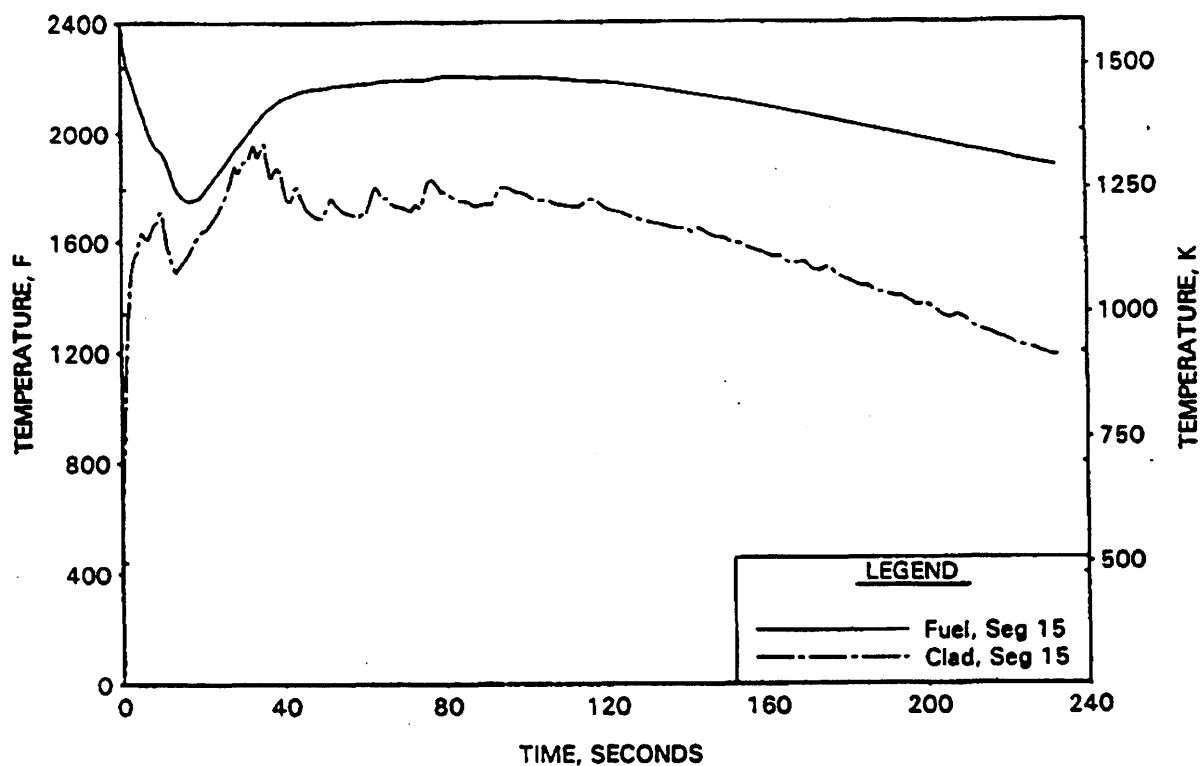
UPDATE - 15  
4/00

LBLOCA LIMIT CASE (BOL - 6.021-FT) -  
HOT CHANNEL FUEL AND CLAD  
TEMPERATURE AT PEAK RUPTURED  
LOCATION

CAD FILE: 6583R15.DWG

FIG 14.2-21





**AmerGen**

TMI - UNIT 1

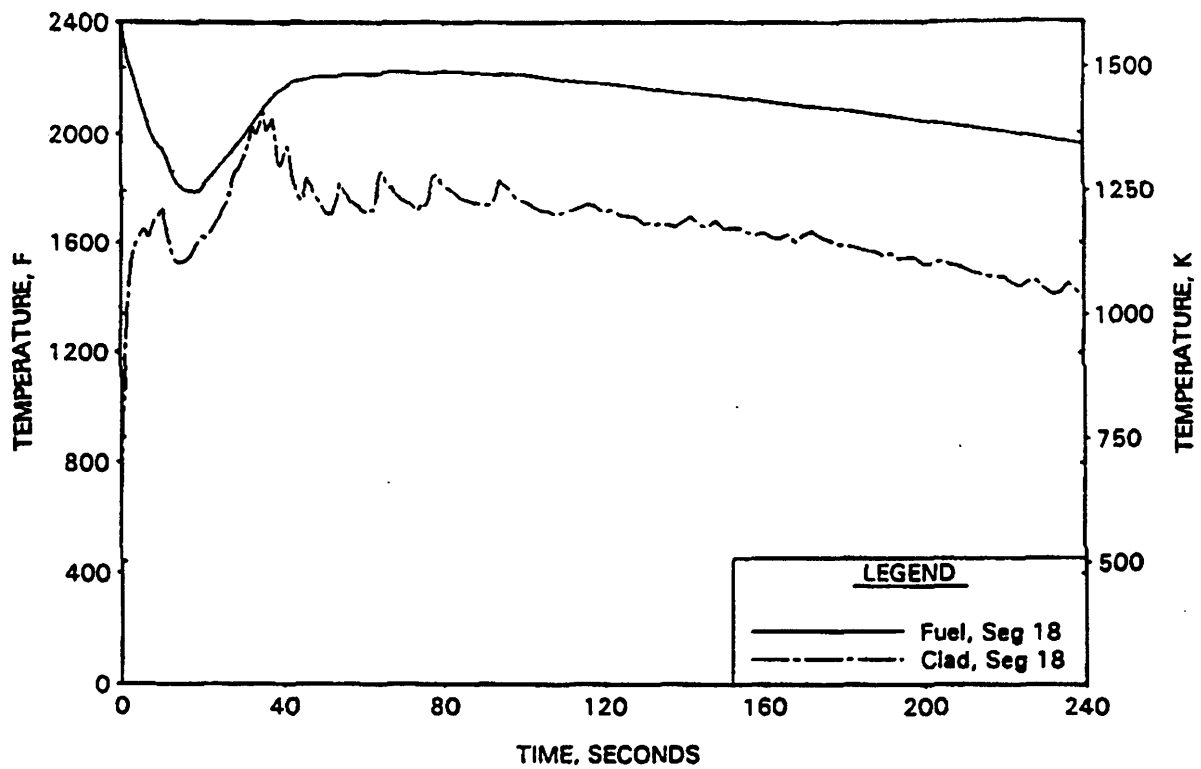
UPDATE - 15

4/00

LBLOCA LIMIT CASE (BOL - 7.779-FT) -  
HOT CHANNEL FUEL AND CLAD  
TEMPERATURE AT PEAK RUPTURED  
LOCATION

CAD FILE: 6584R15.DWG

FIG 14.2-22



**AmerGen**

TMI - UNIT 1

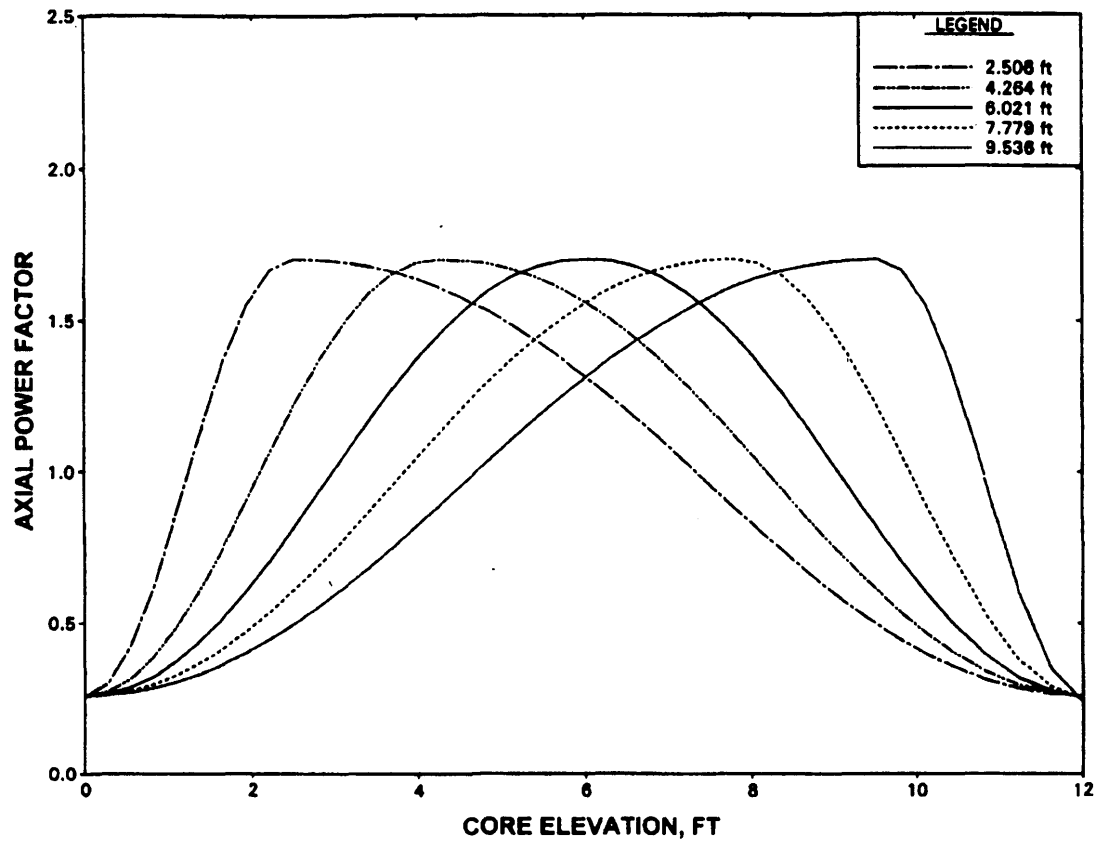
UPDATE - 15

4/00

LBLOCA LIMIT CASE (BOL - 9.536-FT) -  
HOT CHANNEL FUEL AND CLAD  
TEMPERATURE AT PEAK RUPTURED  
LOCATION

CAD FILE: 6585R15.DWG

FIG 14.2-23



TMI - UNIT I

**GPU**

**NUCLEAR**

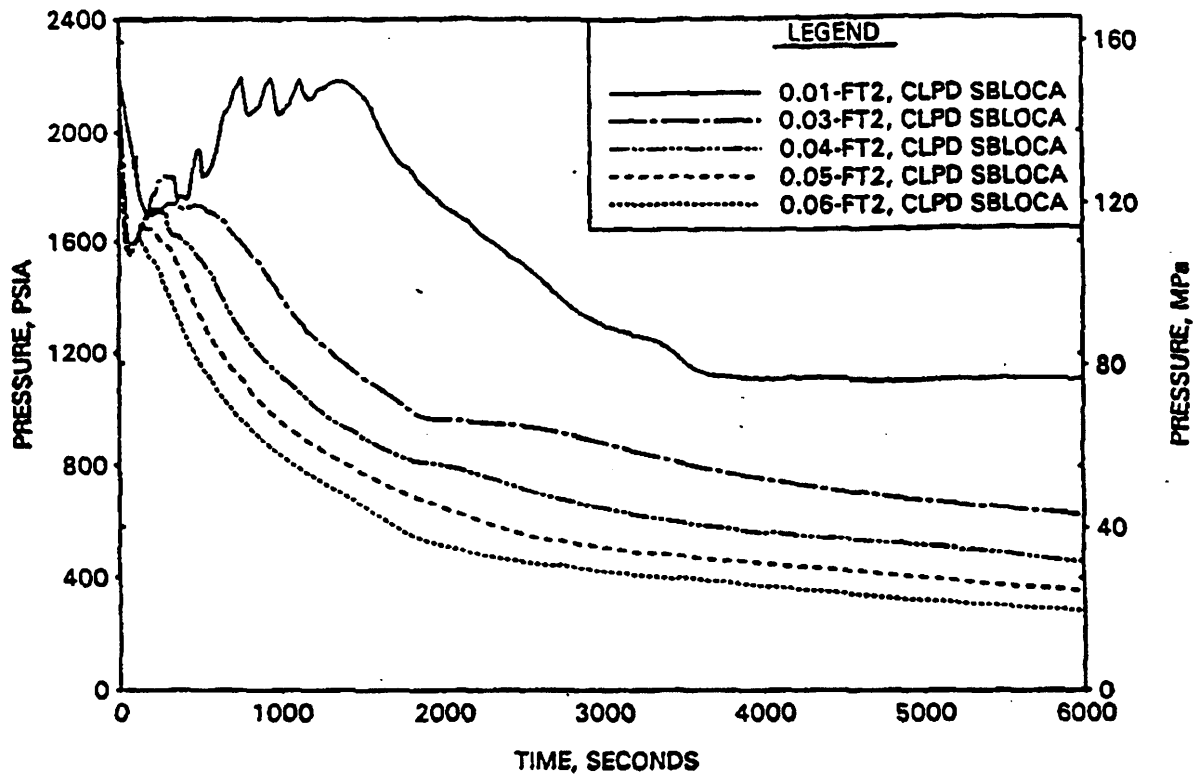
UPDATE - 14

4/98

LOCA LIMIT AXIAL POWER SHAPES

CAD FILE: 6586RI4.DWG

FIG 14.2-24



**AmerGen**

TMI - UNIT 1

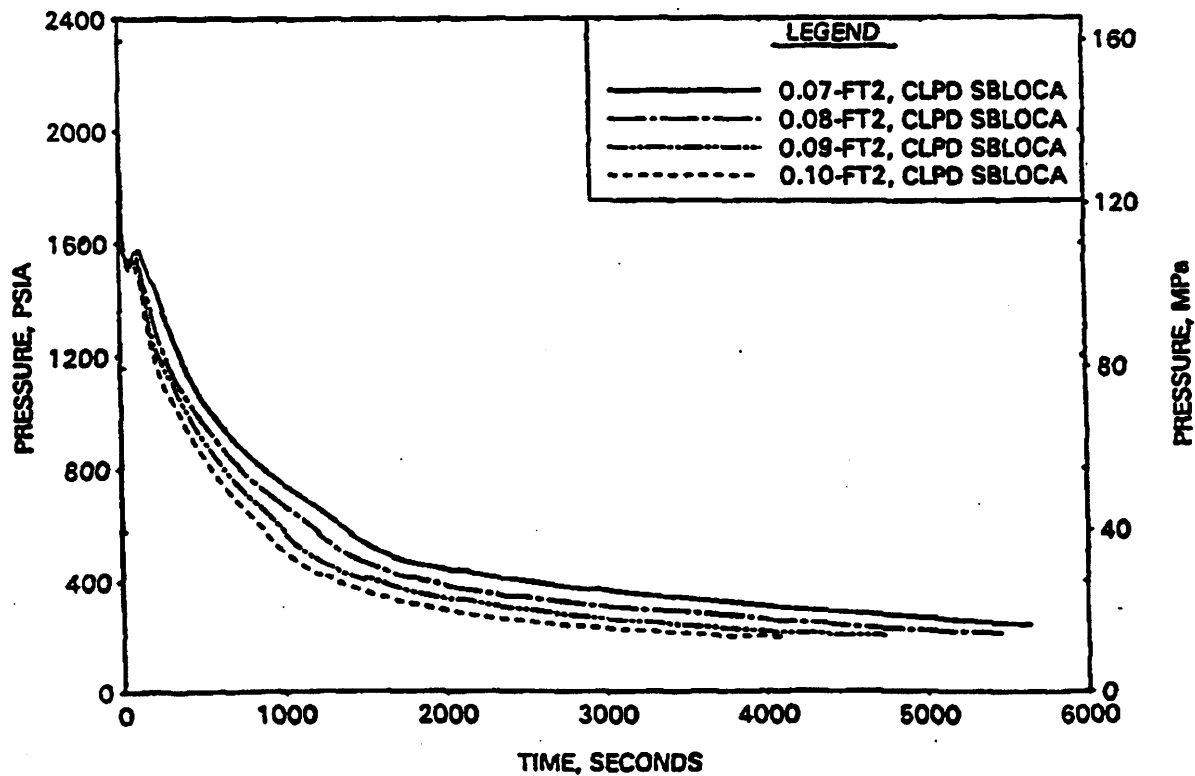
UPDATE - 15

4/00

SBLOCA COMPARISON OF RCS PRESSURE  
(0.01 - 0.06 FT<sup>2</sup> BREAKS)

CAD FILE: 6587R15.DWG

FIG 14.2-25



**AmerGen**

TMI - UNIT 1

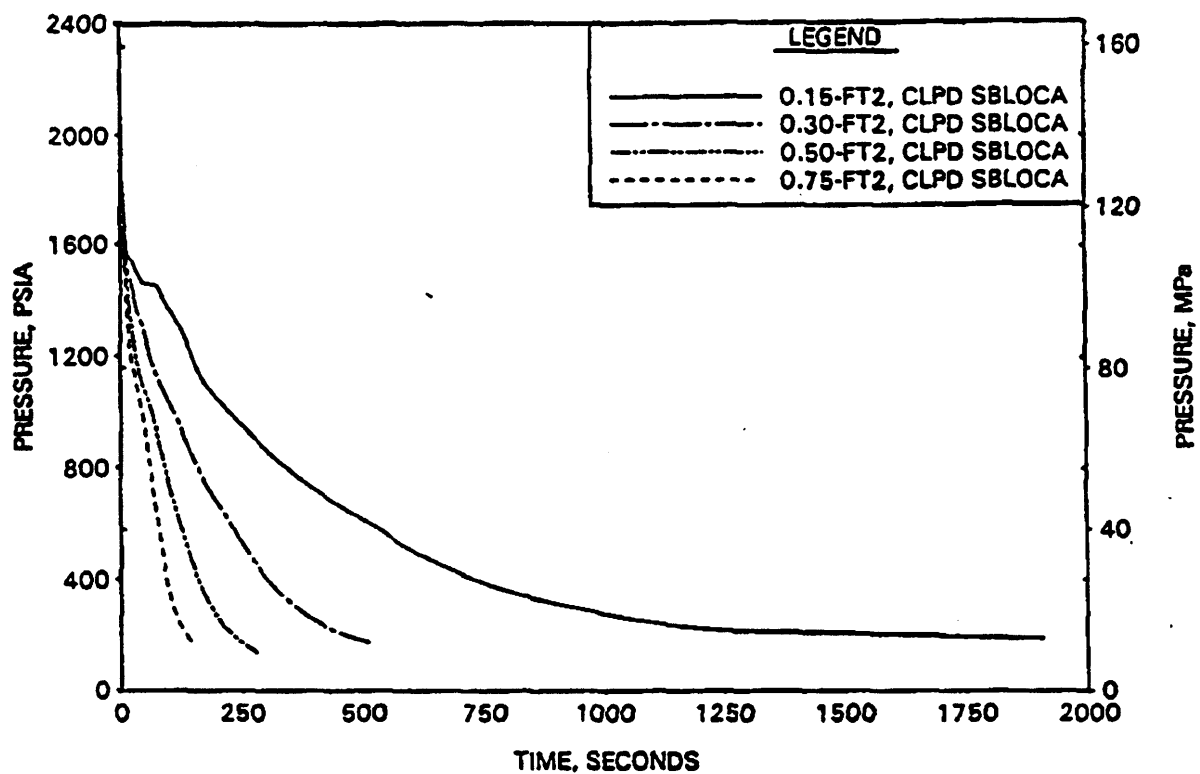
UPDATE - 15

4/00

SBLOCA COMPARISON OF RCS PRESSURE  
(0.07 - 0.10 FT<sup>2</sup> BREAKS)

CAD FILE: 6588R15.DWG

FIG 14.2-26



**AmerGen**

TMI - UNIT 1

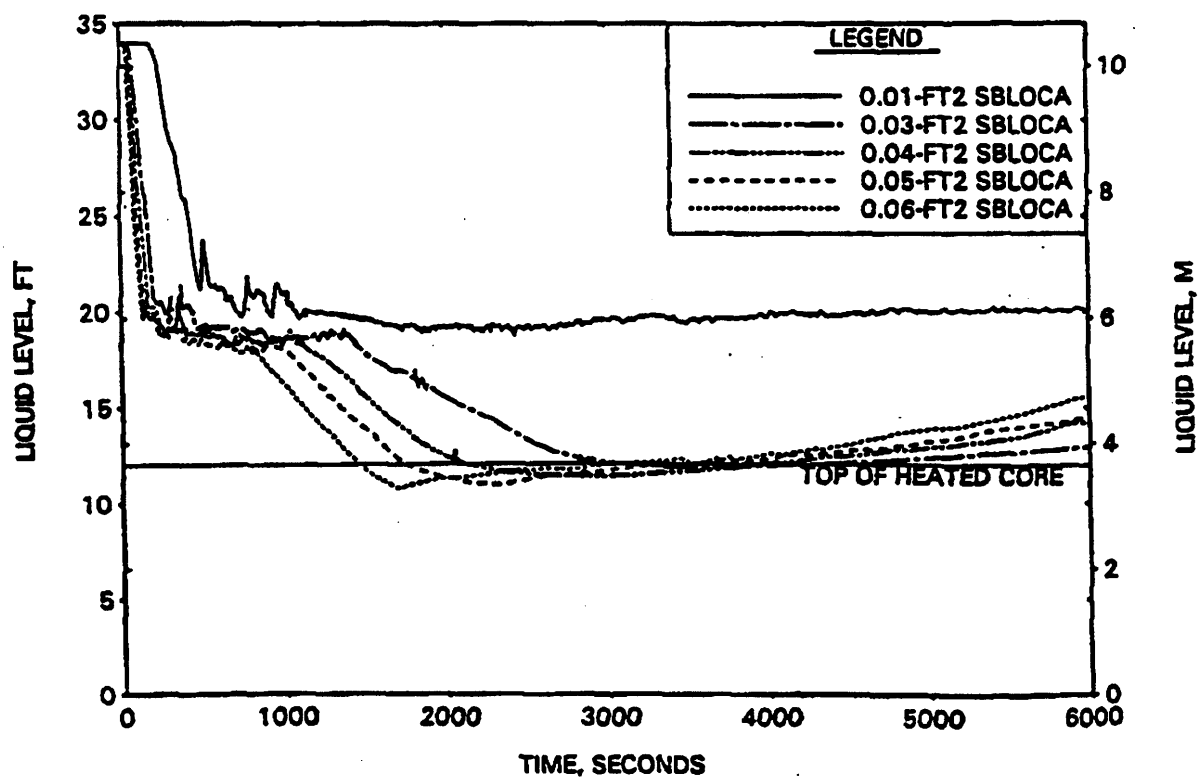
UPDATE - 15

4/00

SBLOCA COMPARISON OF RCS PRESSURE  
(0.15 - 0.75 FT<sup>2</sup> BREAKS)

CAD FILE: 6589R15.DWG

FIG 14.2-27



**AmerGen**

TMI - UNIT 1

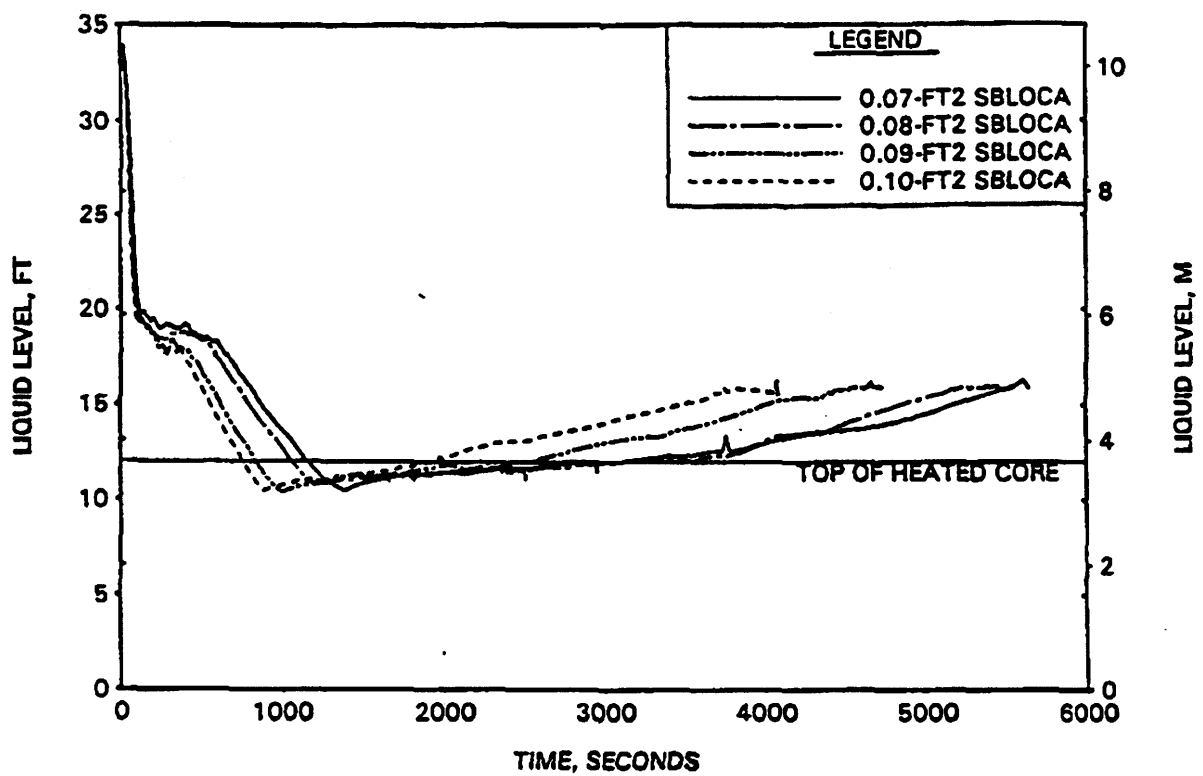
UPDATE - 15

4/00

SBLOCA COMPARISON OF REACTOR VESSEL  
COLLAPSED LIQUID LEVEL (0.01 - 0.06 FT<sup>2</sup>  
BREAKS)

CAD FILE: 6605R15.DWG

FIG 14.2-28



**AmerGen**

TMI - UNIT 1

UPDATE - 15

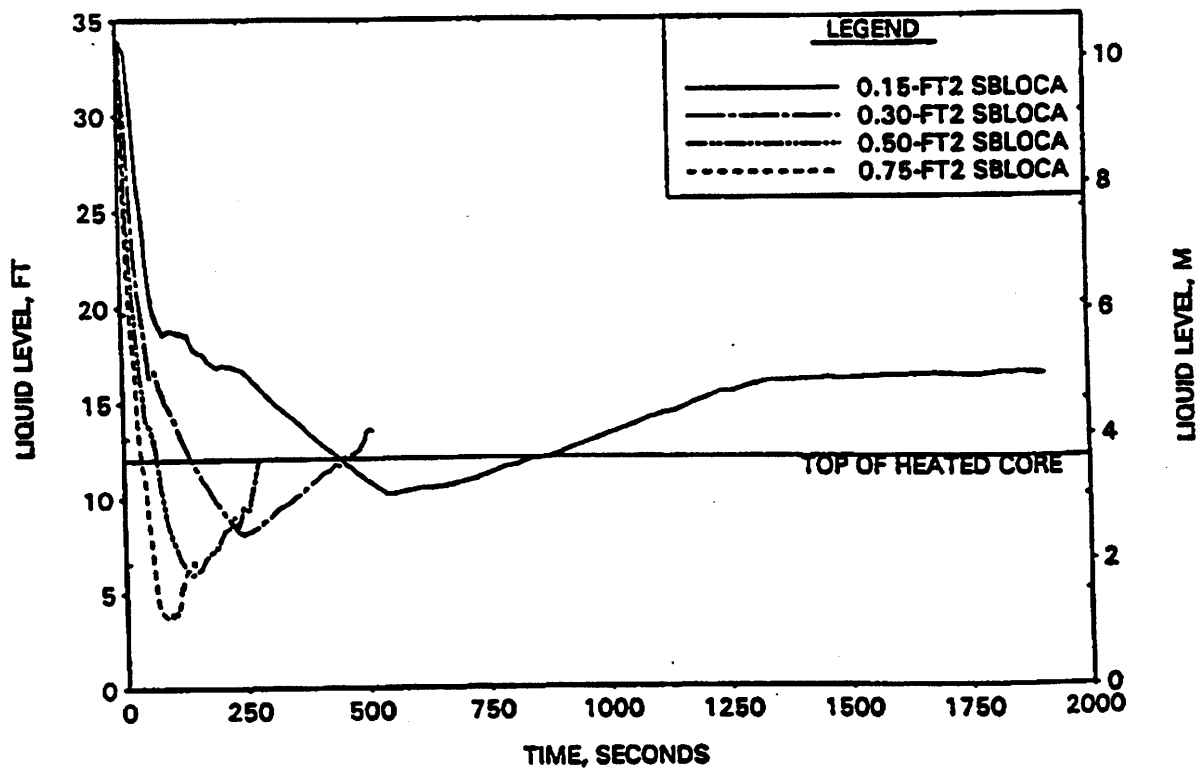
4/00

SBLOCA COMPARISON OF REACTOR VESSEL  
COLLAPSED LIQUID LEVEL (0.07 - 0.10 FT<sup>2</sup>  
BREAKS)

CAD FILE: 6606R15.DWG

FIG 14.2-29





**AmerGen**

TMI - UNIT 1

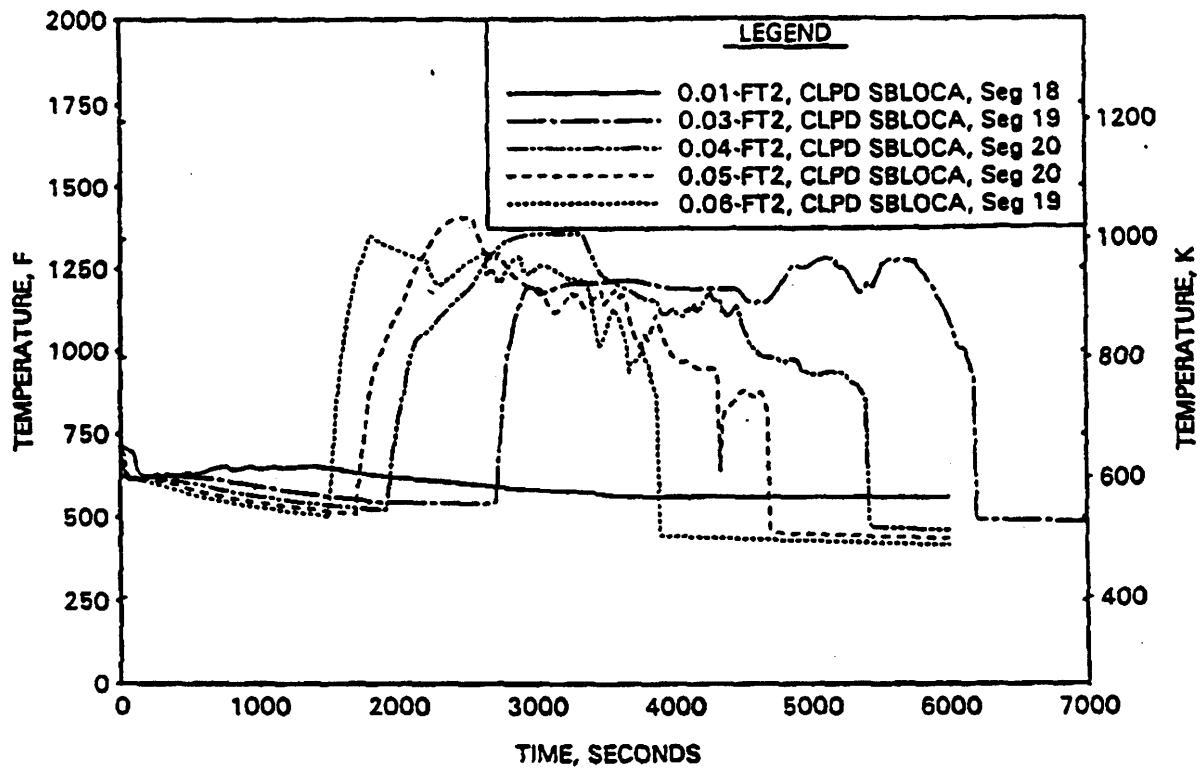
UPDATE - 15

4/00

SBLOCA COMPARISON OF REACTOR VESSEL  
COLLAPSED LIQUID LEVEL (0.15 - 0.75 FT<sup>2</sup>  
BREAKS)

CAD FILE: 6607R15.DWG

FIG 14.2-30



**AmerGen**

TMI - UNIT 1

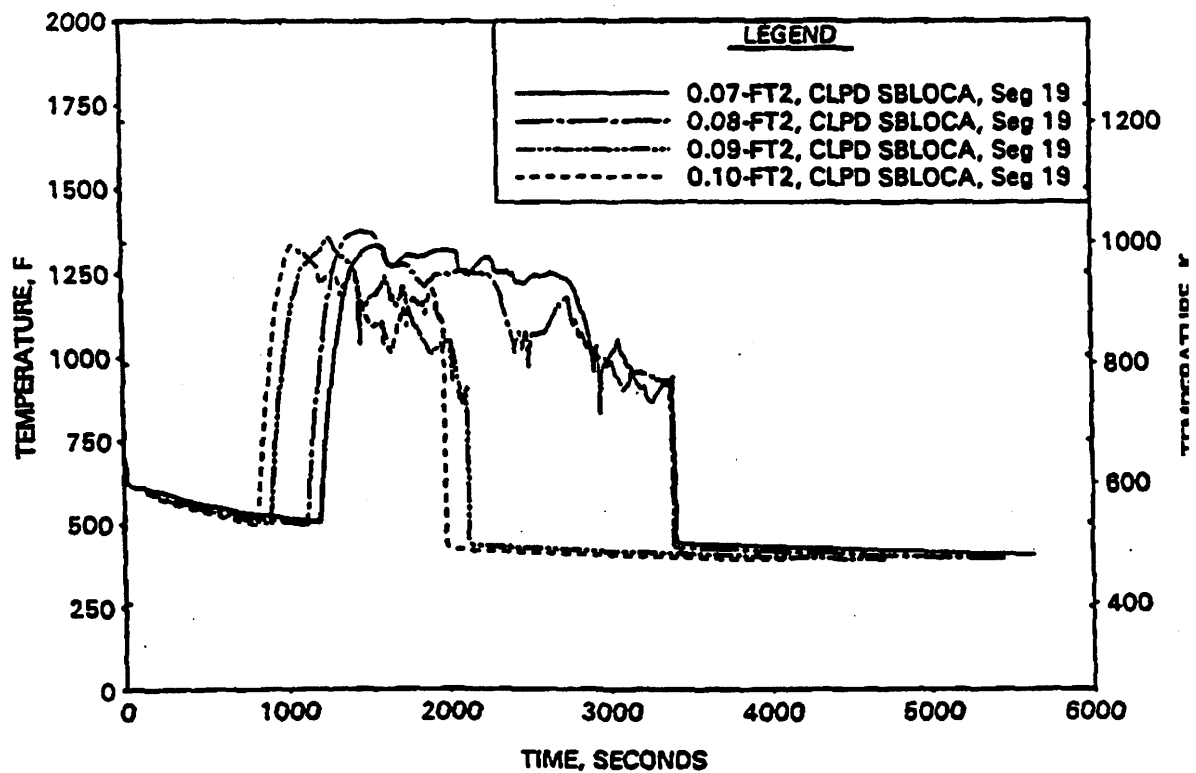
UPDATE - 15

4/00

SBLOCA COMPARISON OF PEAK CLAD  
TEMPERATURES (0.01 - 0.06 FT<sup>2</sup> BREAKS)

CAD FILE: 6608R15.DWG

FIG 14.2-31



**AmerGen**

TMI - UNIT 1

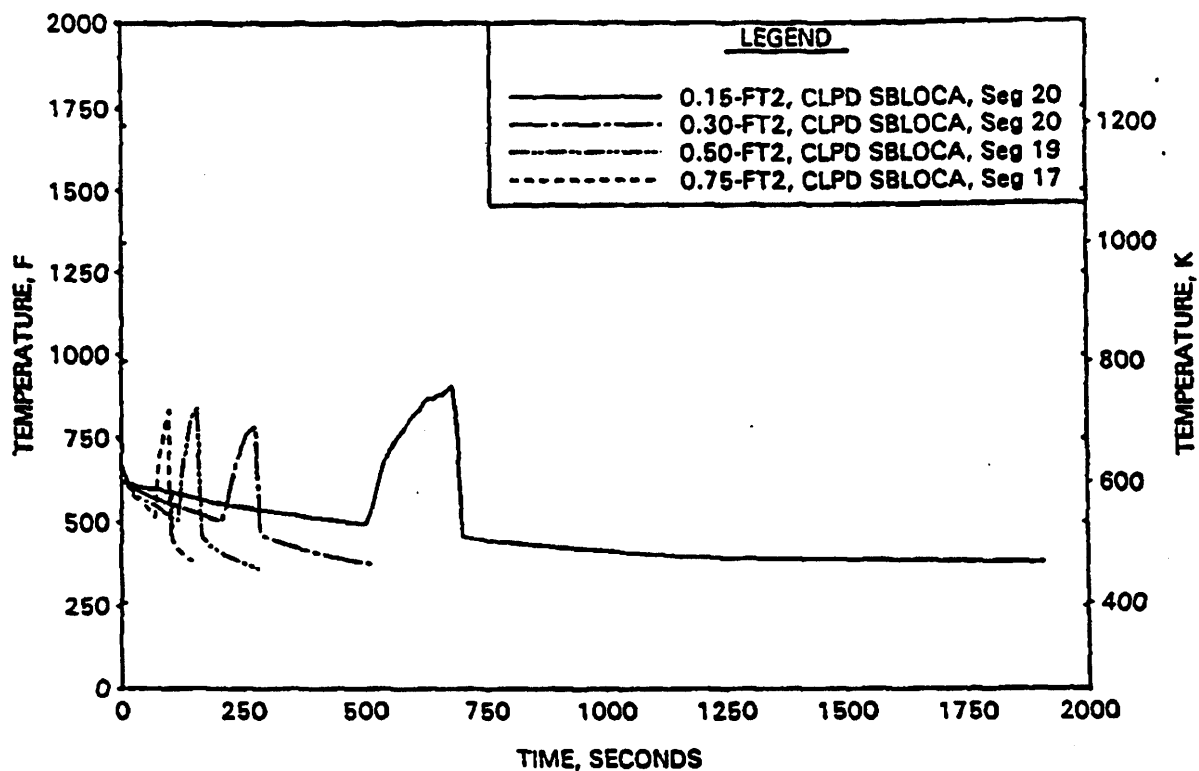
UPDATE - 15

4/00

SBLOCA COMPARISON OF PEAK CLAD  
TEMPERATURES (0.07 - 0.10 FT<sup>2</sup> BREAKS)

CAD FILE: 6590R15.DWG

FIG 14.2-32



**AmerGen**

TMI - UNIT 1

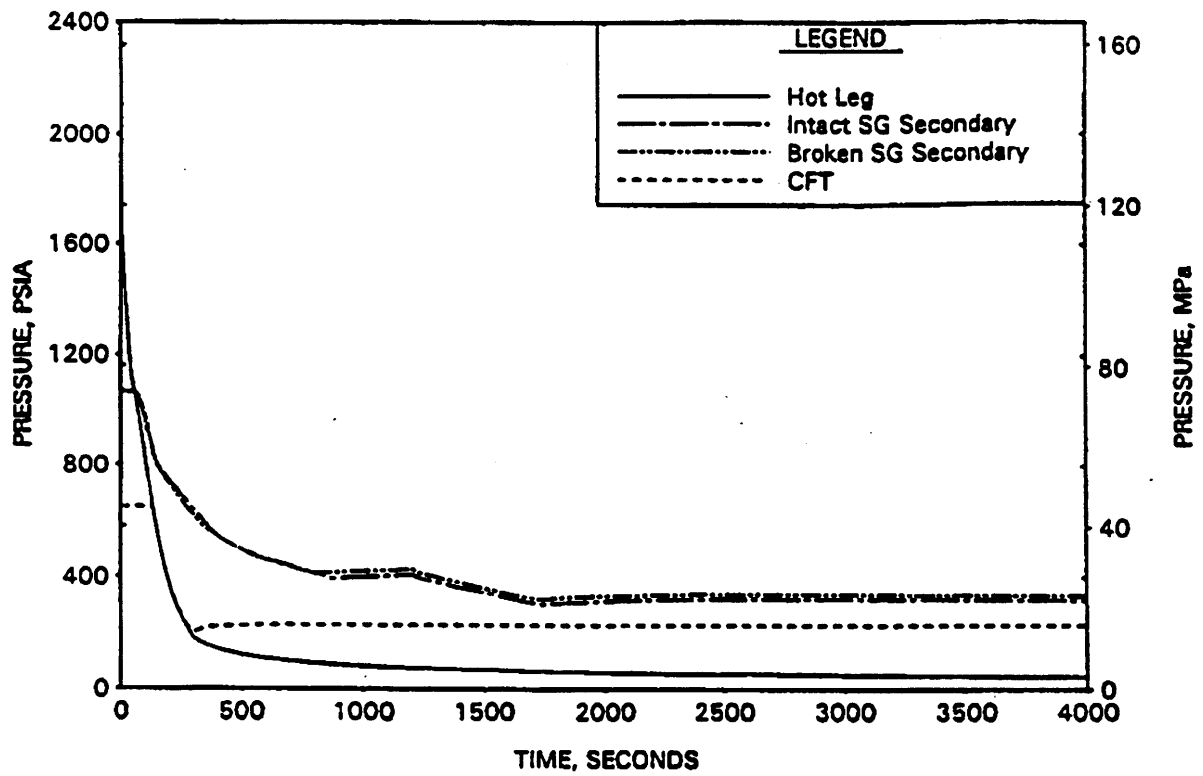
UPDATE - 15

4/00

SBLOCA COMPARISON OF PEAK CLAD  
TEMPERATURES (0.15 - 0.75 FT<sup>2</sup> BREAKS)

CAD FILE: 6591R15.DWG

FIG 14.2-33



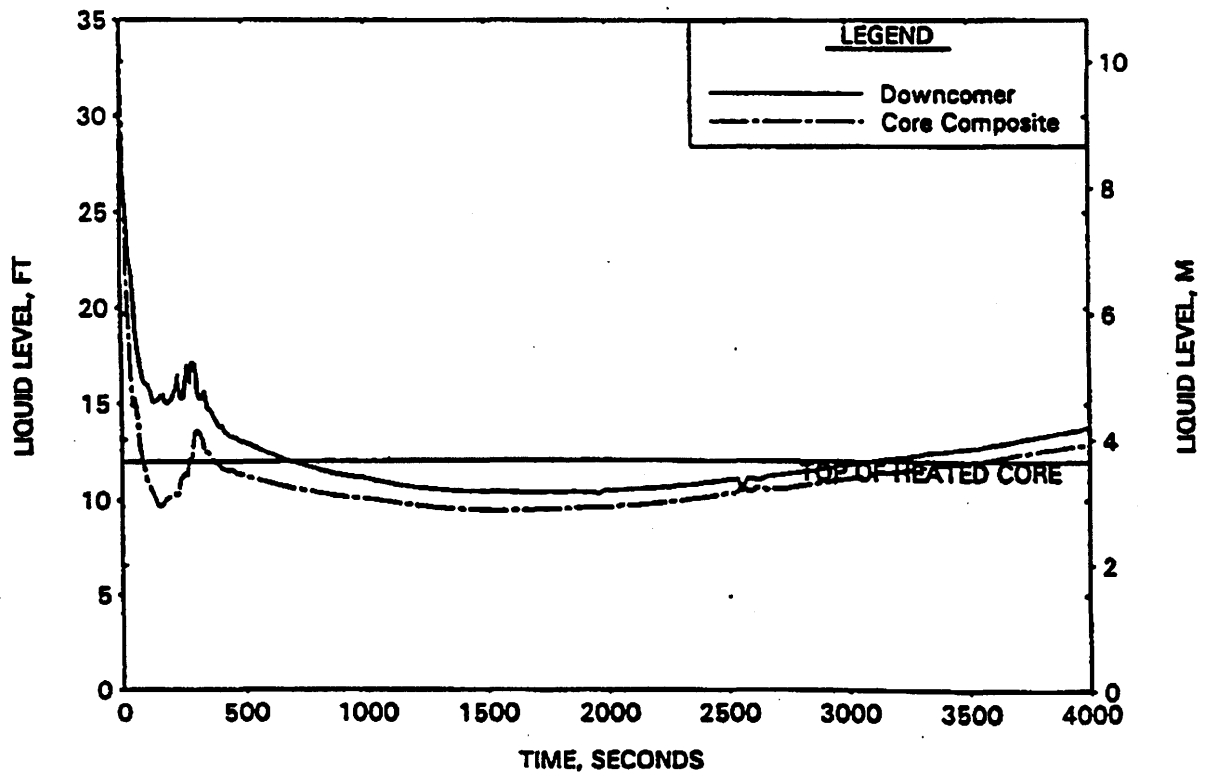
**AmerGen**  
TMI - UNIT 1

UPDATE - 15  
4/00

CFT LINE BREAK SYSTEM PRESSURE

CAD FILE: 6592R15.DWG

FIG 14.2-34



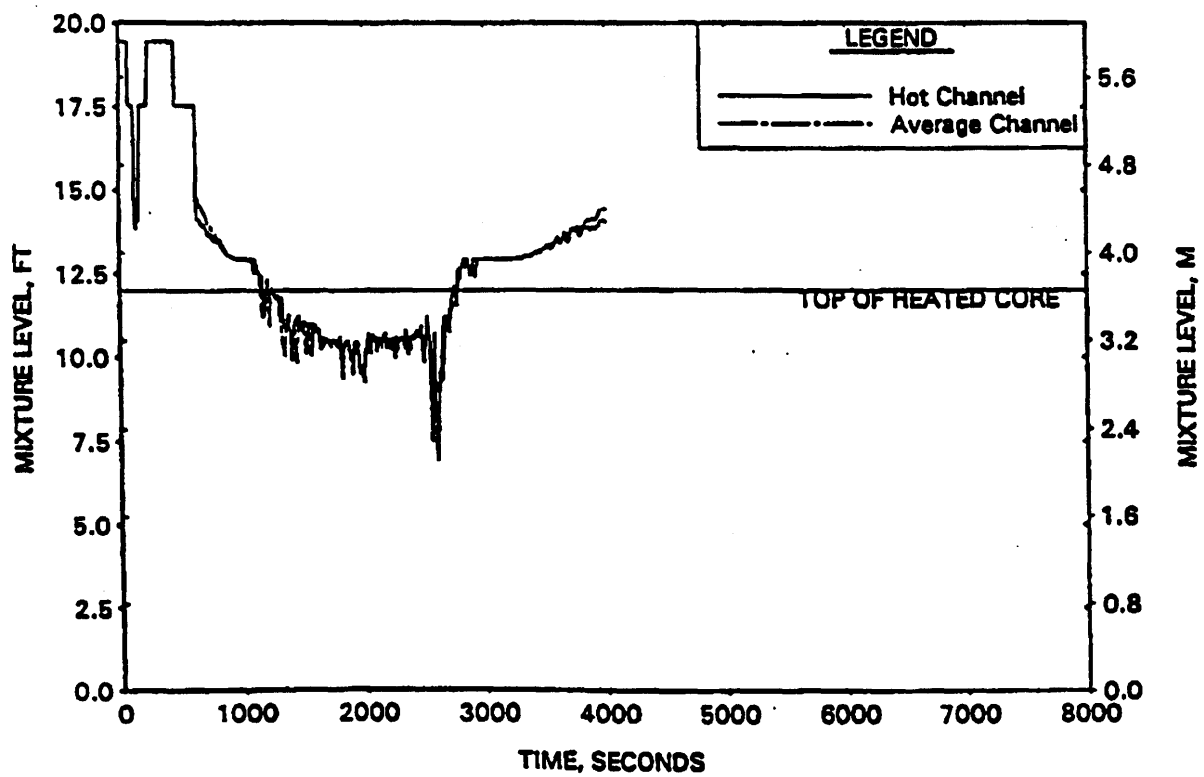
**AmerGen**  
TMI - UNIT 1

UPDATE - 15  
4/00

CFT LINE BREAK COLLAPSED LIQUID LEVELS

CAD FILE: 6593R15.DWG

FIG 14.2-35



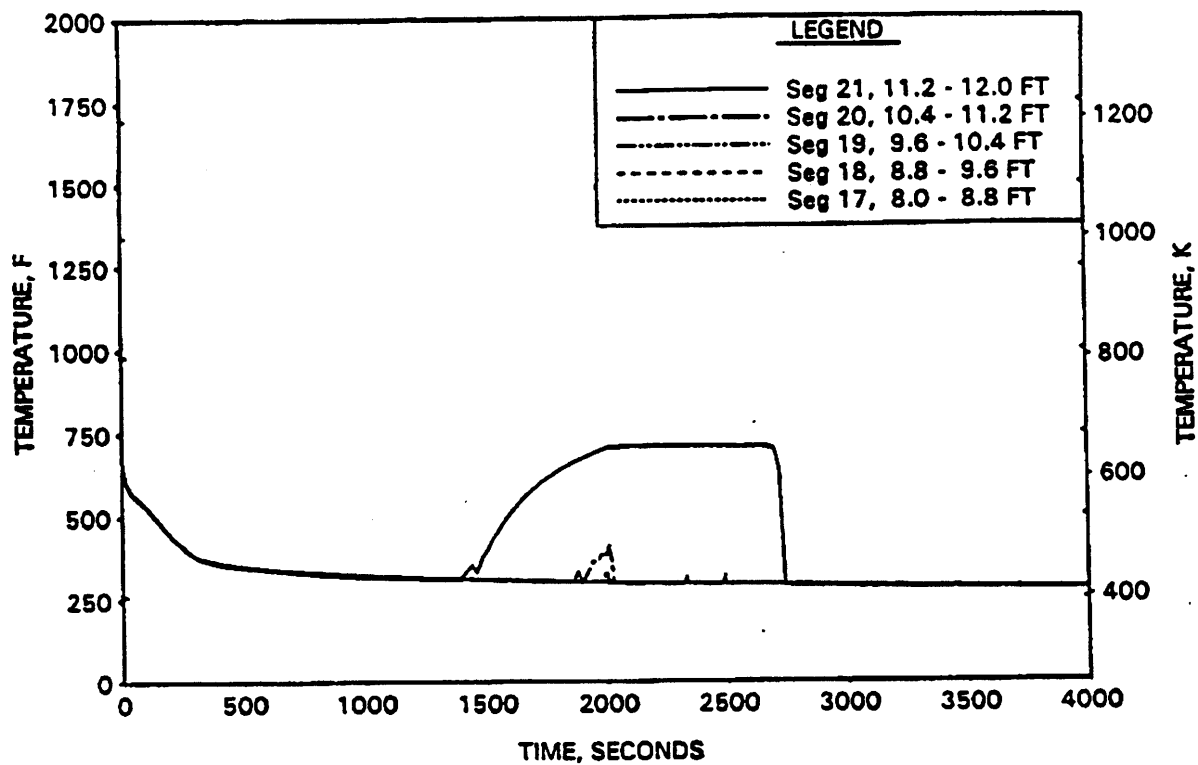
**AmerGen**  
TMI - UNIT 1

UPDATE - 15  
4/00

CFT LINE BREAK MIXTURE LEVELS

CAD FILE: 6594R15.DWG

FIG 14.2-36



**AmerGen**

TMI - UNIT 1

UPDATE - 15

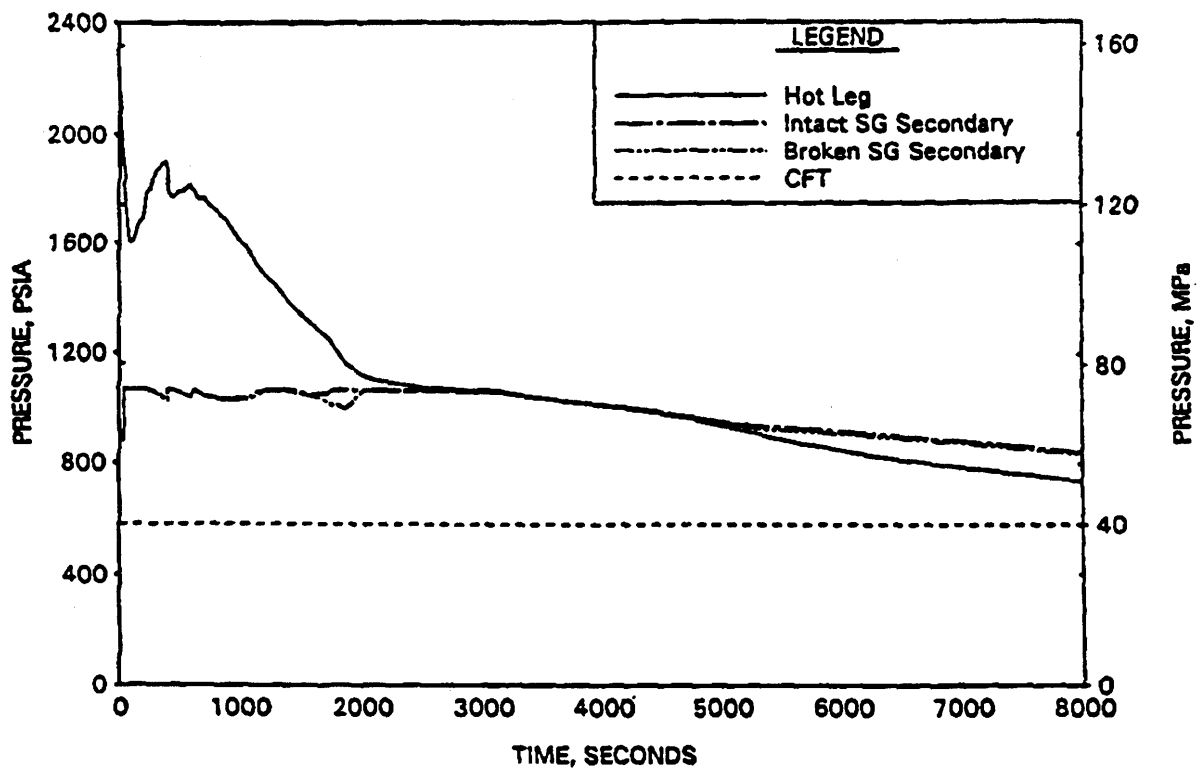
4/00

CFT LINE BREAK HOT CHANNEL CLAD  
TEMPERATURES

CAD FILE: 6595R15.DWG

FIG 14.2-37





**AmerGen**

TMI - UNIT 1

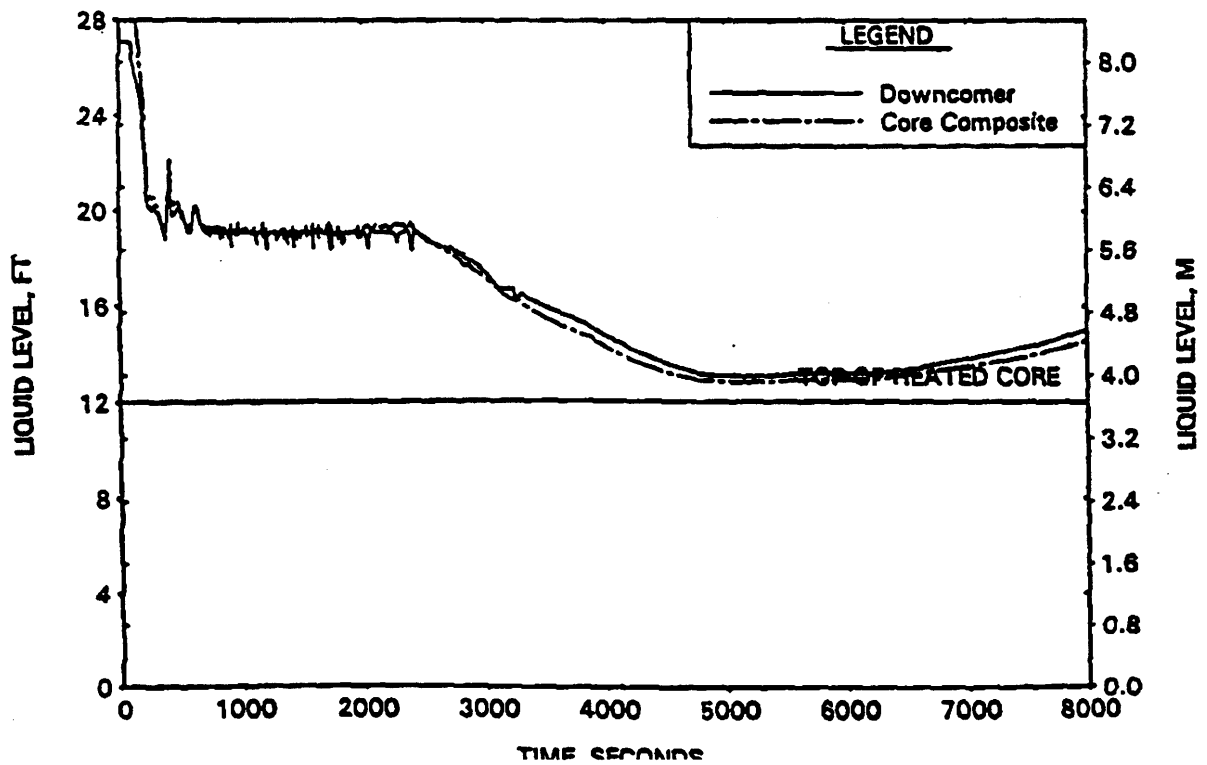
UPDATE - 15

4/00

HPI LINE BREAK SYSTEM PRESSURE  
(OPERATOR ACTION)

CAD FILE: 6596R15.DWG

FIG 14.2-38



**AmerGen**

TMI - UNIT 1

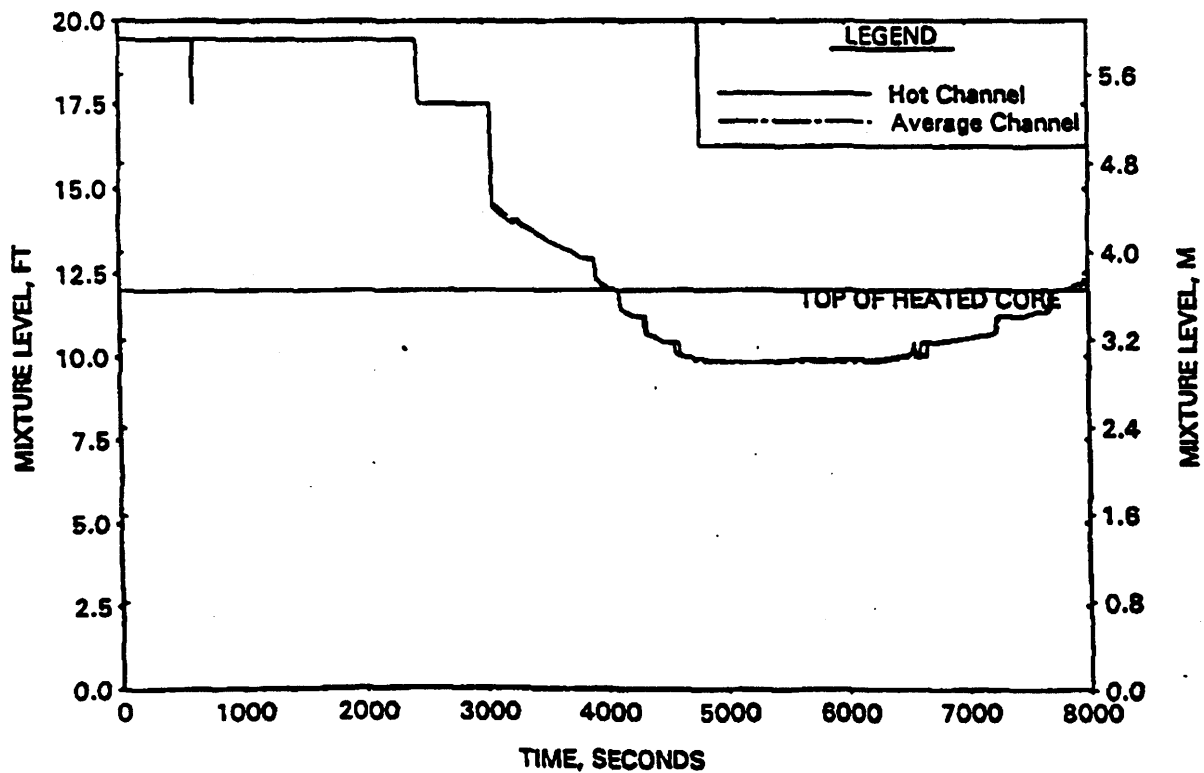
UPDATE - 15

4/00

HPI LINE BREAK COLLAPSED LIQUID LEVELS  
(OPERATOR ACTION)

CAD FILE: 6597R15.DWG

FIG 14.2-39



**AmerGen**

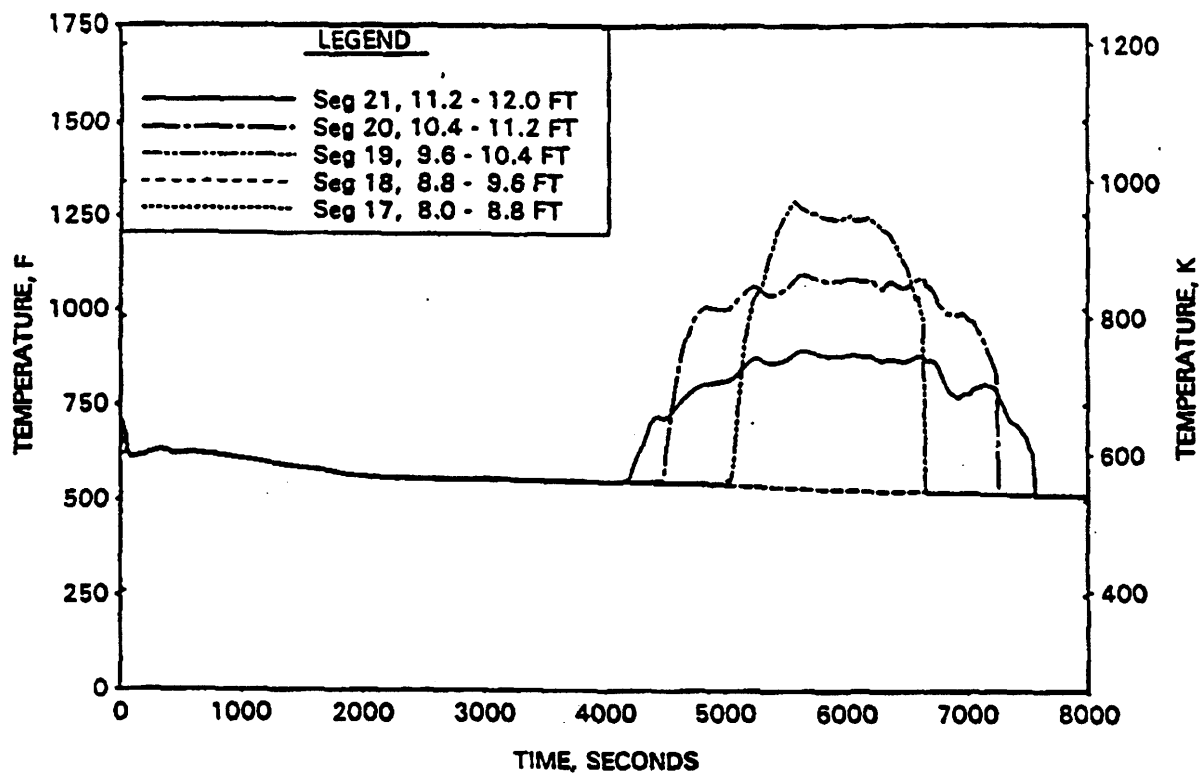
TMI - UNIT 1

UPDATE - 15  
4/00

HPI LINE BREAK MIXTURE LEVELS  
(OPERATOR ACTION)

CAD FILE: 6598R15.DWG

FIG 14.2-40



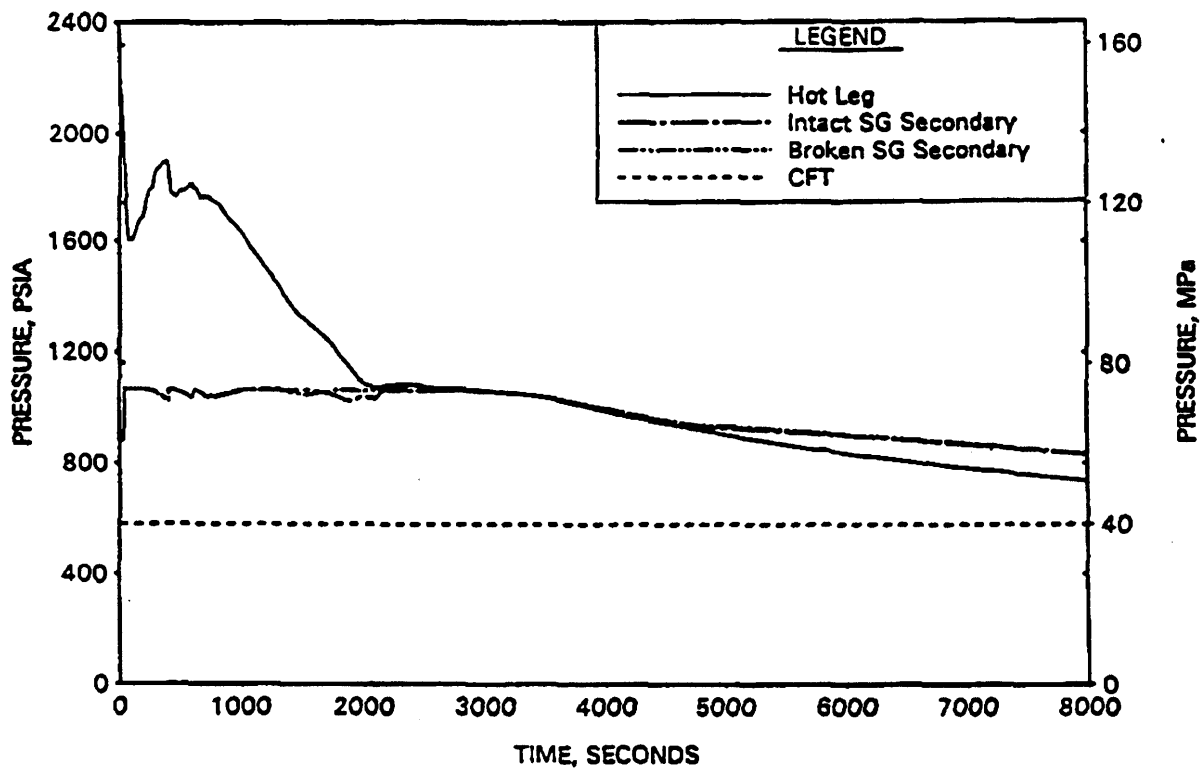
**AmerGen**  
TMI - UNIT 1

UPDATE - 15  
4/00

HPI LINE BREAK HOT CHANNEL CLAD  
TEMPERATURES (OPERATOR ACTION)

CAD FILE: 6599R15.DWG

FIG 14.2-41



**AmerGen**

TMI - UNIT 1

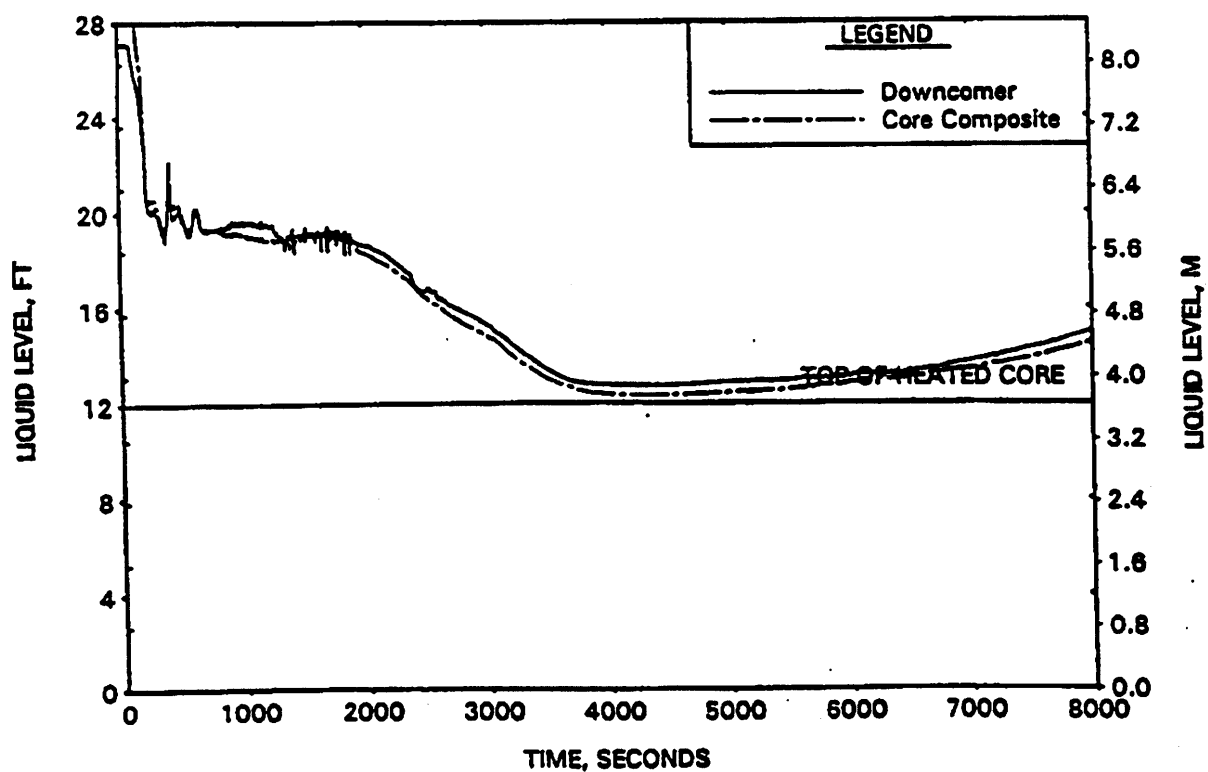
UPDATE - 15

4/00

HPI LINE BREAK SYSTEM PRESSURE (NO  
OPERATOR ACTION)

CAD FILE: 6600R15.DWG

FIG 14.2-42



**AmerGen**

TMI - UNIT 1

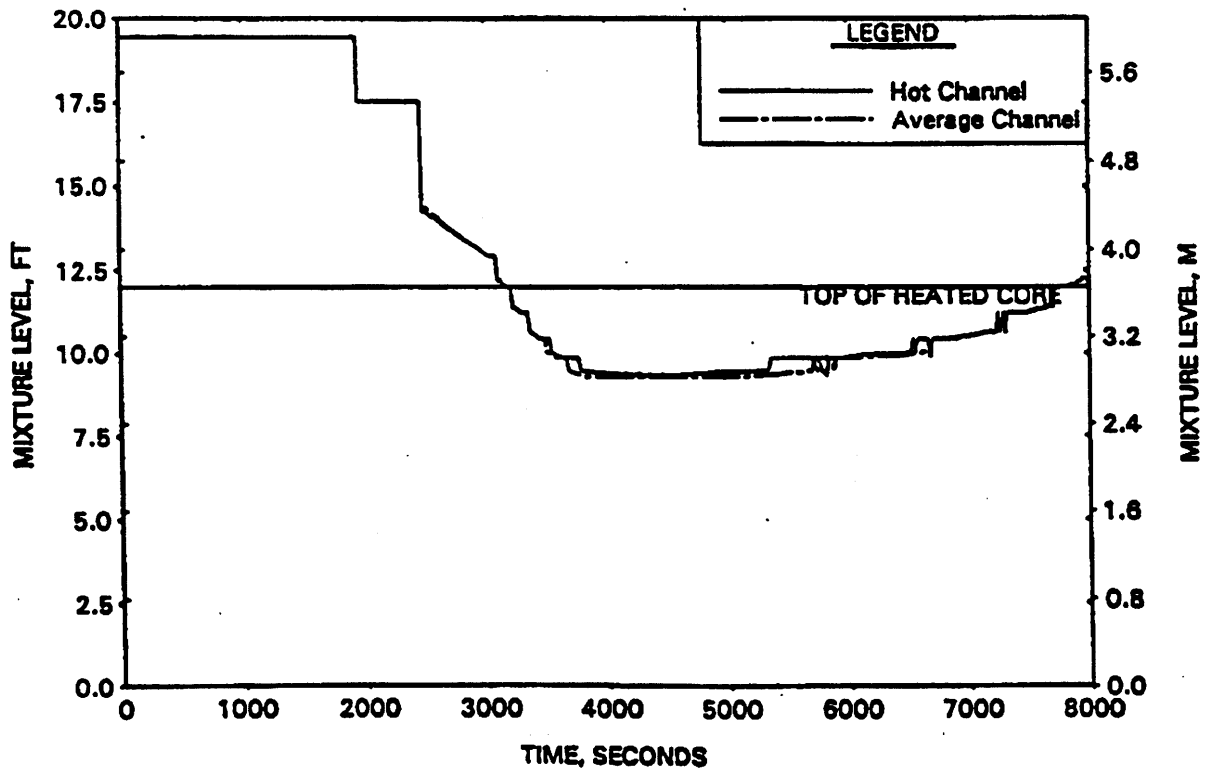
UPDATE - 15

4/00

HPI LINE BREAK COLLAPSED LIQUID LEVELS  
(NO OPERATOR ACTION)

CAD FILE: 6609R15.DWG

FIG 14.2-43



**AmerGen**

TMI - UNIT 1

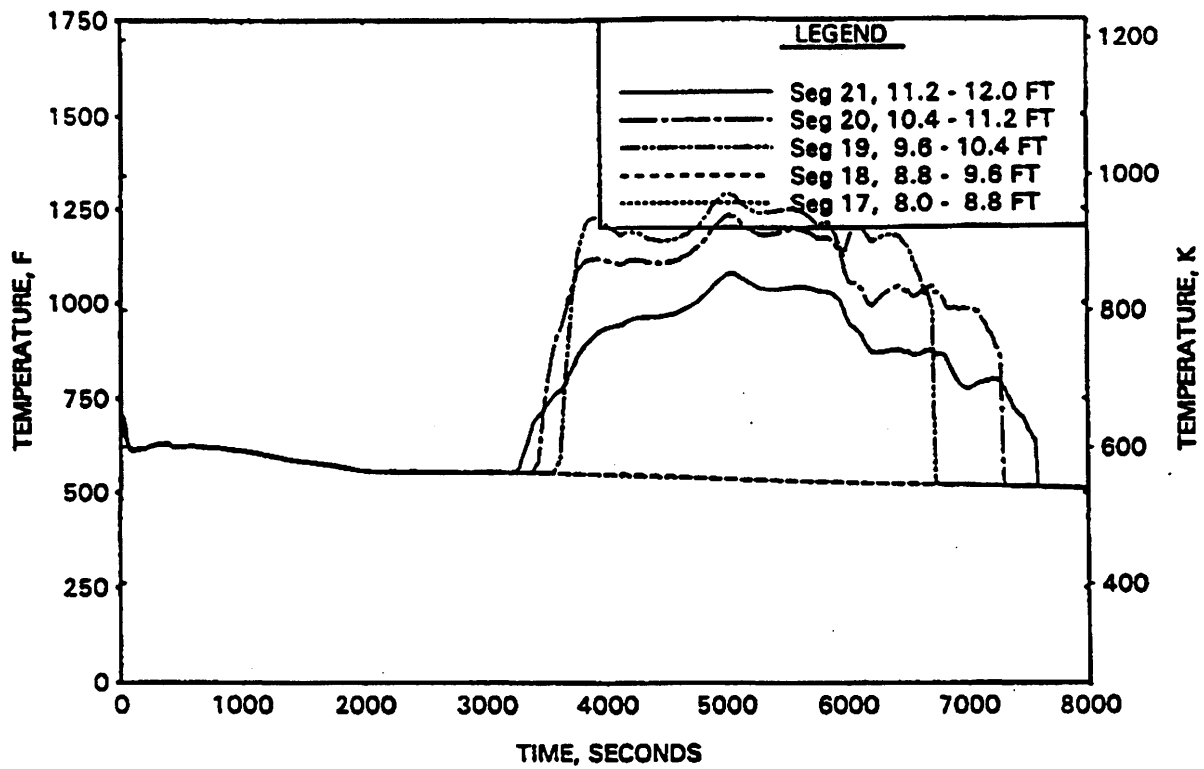
UPDATE - 15

4/00

HPI LINE BREAK MIXTURE LEVELS (NO  
OPERATOR ACTION)

CAD FILE: 6610R15.DWG

FIG 14.2-44



**AmerGen**

TMI - UNIT 1

UPDATE - 15

4/00

HPI LINE BREAK HOT CHANNEL CLAD  
TEMPERATURES (NO OPERATOR ACTION)

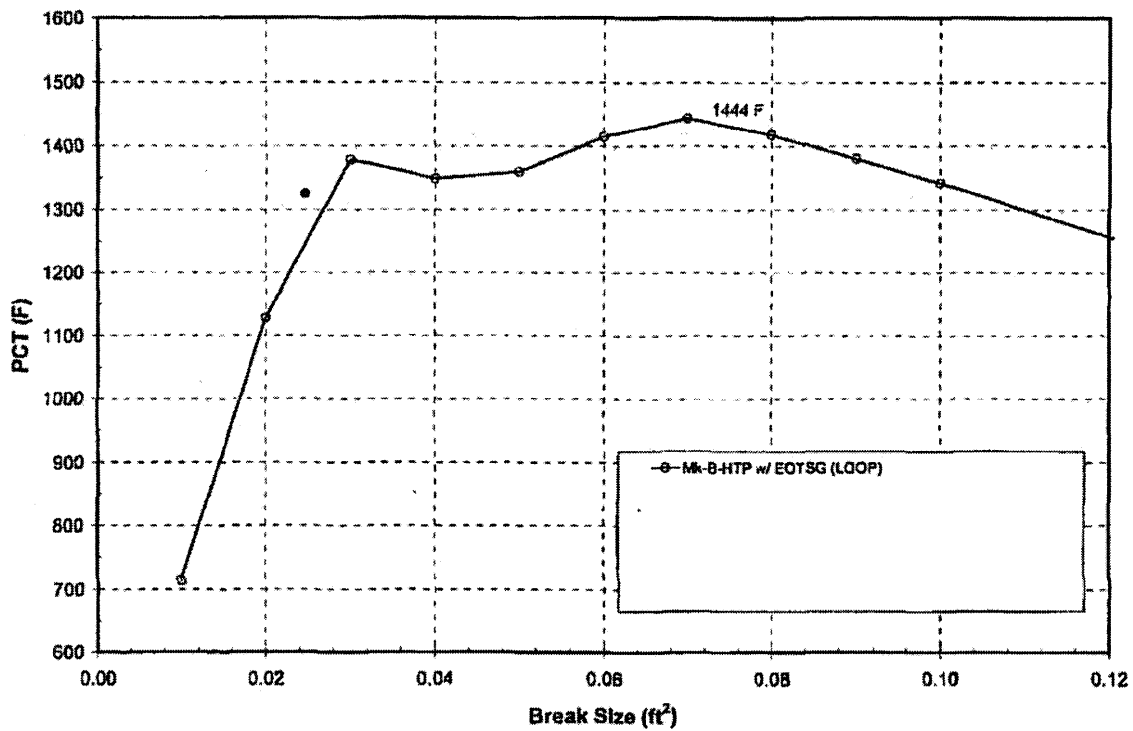
CAD FILE: 6611R15.DWG

FIG 14.2-45



# TMI-1 UFSAR

## Summary of SBLOCA PCT Spectrum – Limiting Break Range

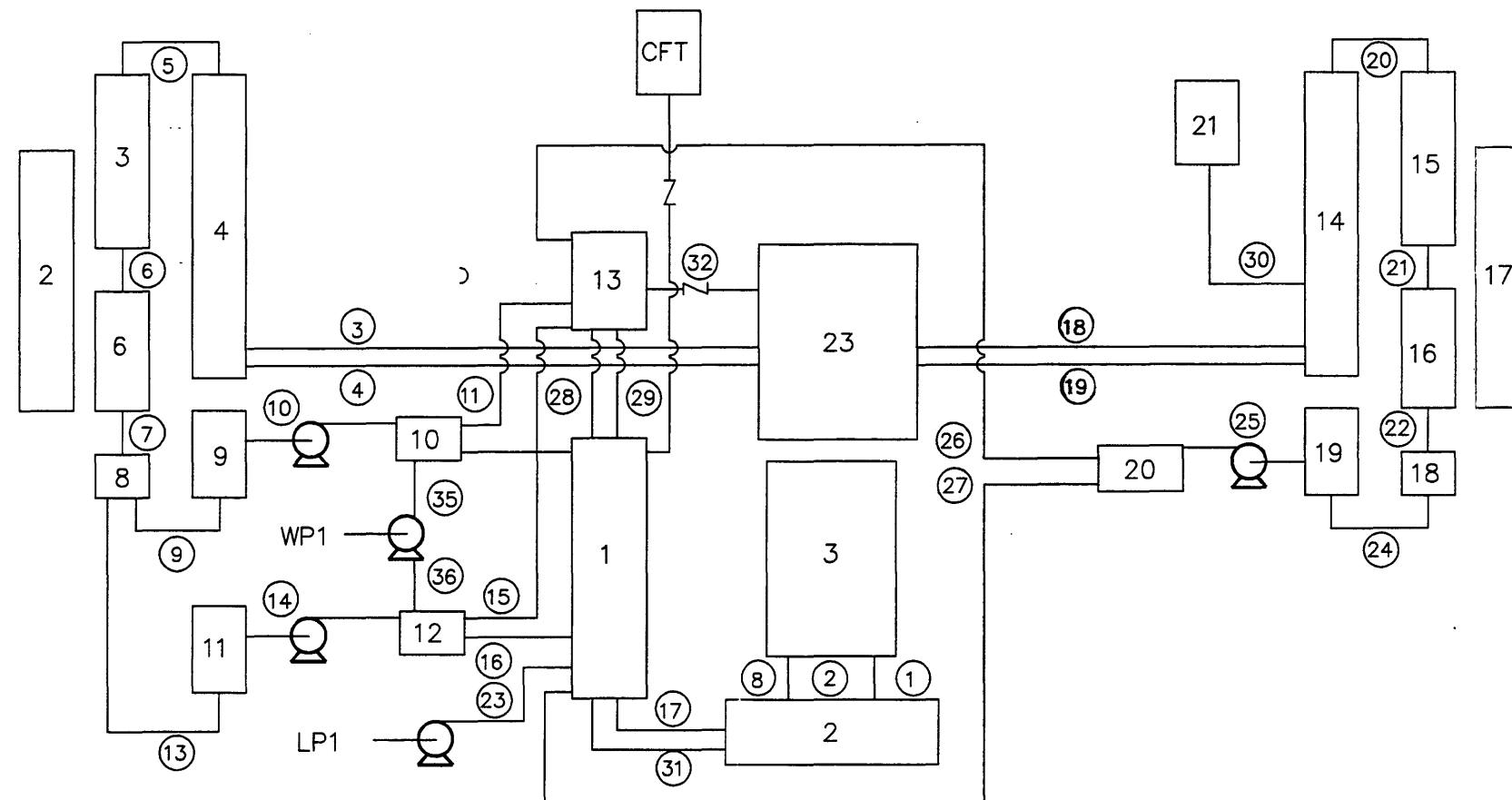


p. 14.FIG-68  
p. 14.FIG-68

**TMI-1**  
**Updated Final Safety Analysis Report**  
**SBLOCA SPECTRUM - PCT VERSUS**  
**BREAK SIZE**

Figure 14.2-68

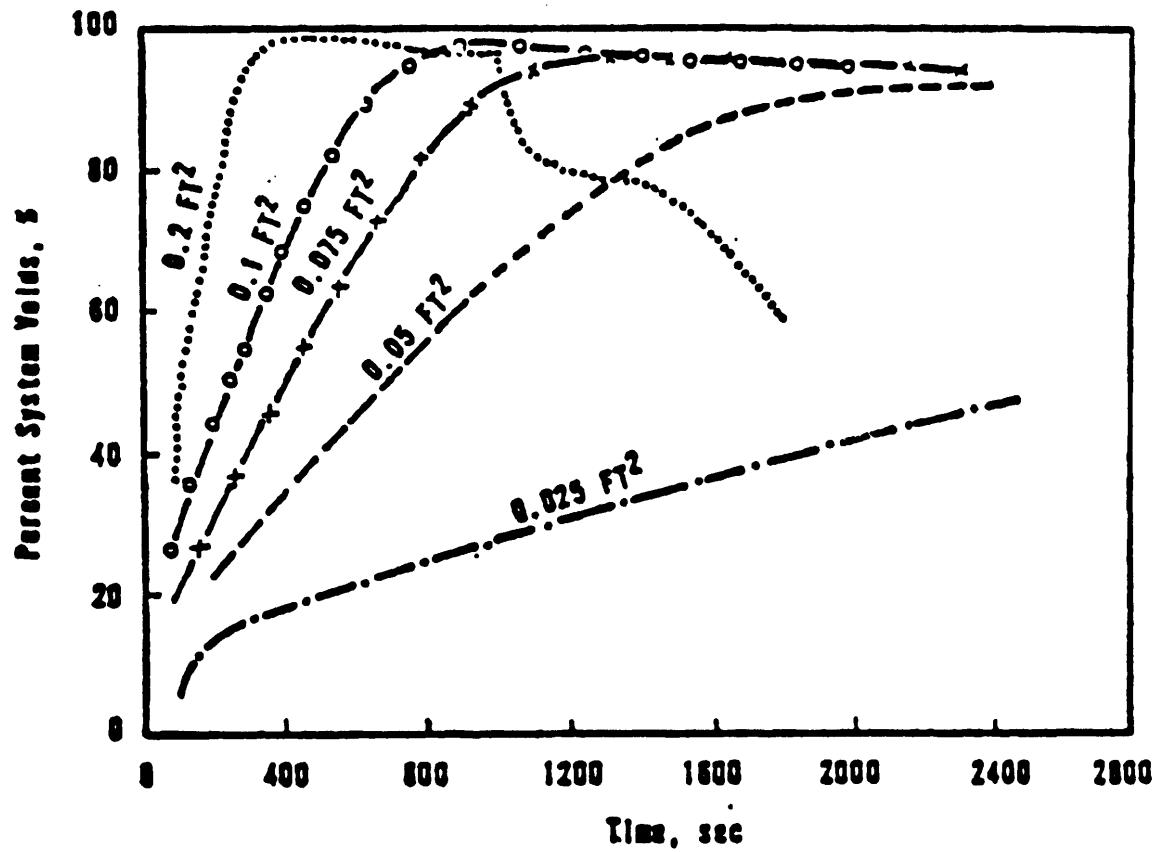
Rev. 20, 04/10

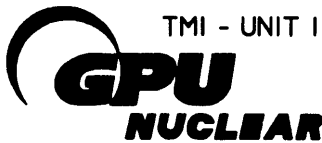


Node No.	Identification	Path No.	Identification
1	Downcomer	1,2	Core
2	Lower Plenum	3,4,18,19	Hot Leg Piping
3	Core	5,20	Hot Leg,Upper
4,14	Hot Leg Piping	6,21	SG Tubes
5,15	SG & Upper Head	7,22	SG Lower Head
6,16	Steam Generator Tubes	8	Core Bypass
7,17	Secondary,SG	9,13,24	Cold Leg Piping
8,18	SG Lower Head	10,14,25	Pumps
9,11,19	Cold Leg Piping	11,12,15,16,26,27	Cold Leg Piping
10,12,20	Cold Leg Piping	17,31	Downcomer
13	Upper Downcomer	23	LPI
21	Pressurizer	28,29	Upper Downcomer
22	Containment	30	Pressurizer
23	Upper Plenum	32	Vent Valve
		33,34	Leak & Return Path
		35,36	HPI
		37	Containment Sprays

Note: Additional Data Not Shown on Diagram are:

Node 22 Is Containment Node  
 Path 33 Is Leak Path From Break Node to Containment  
 Path 34 Is Return Leak Path From Containment to Break Node  
 Path 37 Represents Containment Spray System





**GPU  
NUCLEAR**

TMI - UNIT I

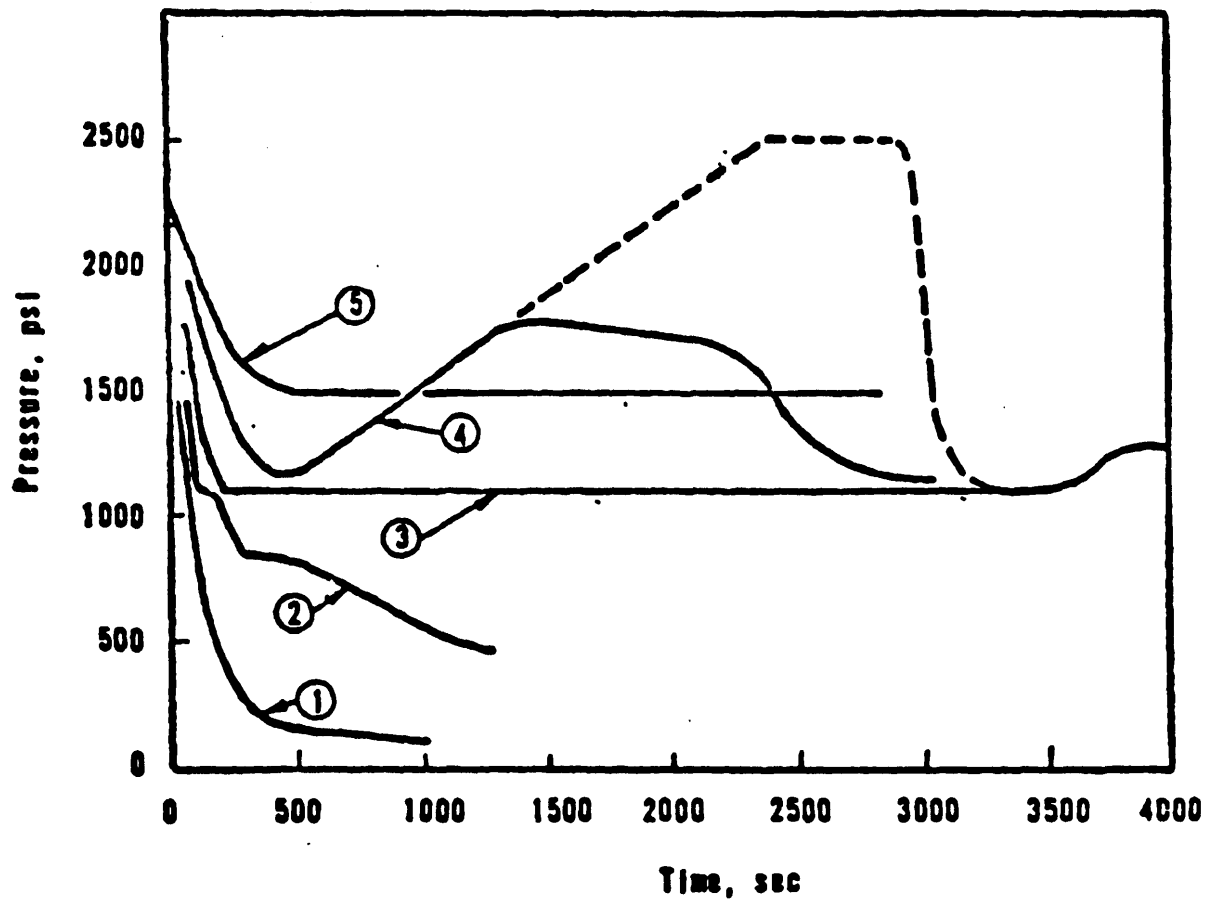
UPDATE - 14  
4/98


BREAK SPECTRUM - AVERAGE SYSTEM  
VOID FRACTION WITH THE RC PUMPS  
OPERATIVE AND 2 HPI PUMPS

CAD FILE: 6602RI4.DWG

FIG 14.2-48

# PRESSURE VS TIME - SMALL BREAKS WITH AUXILIARY FEEDWATER





TMI - UNIT I

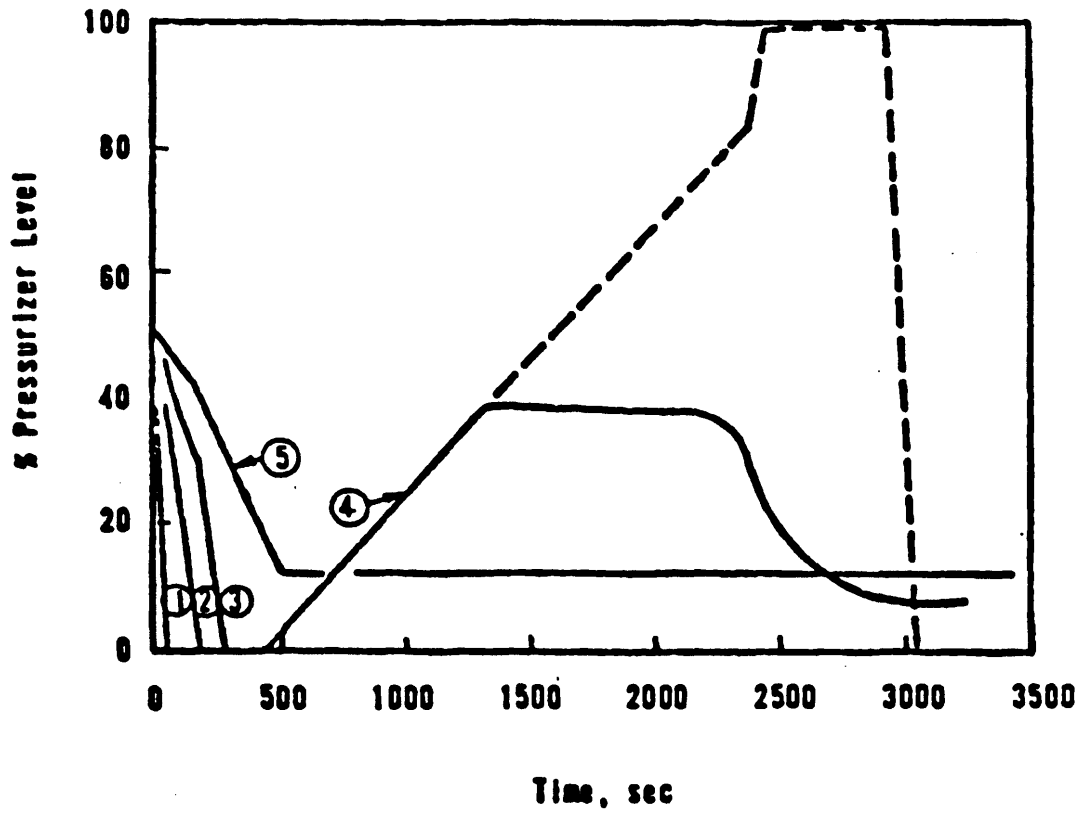
UPDATE - 14  
4/98


PRESSURE VS TIME - SMALL BREAKS  
WITH EMERGENCY FEED WATER

CAD FILE: 6603RI4.DWG

FIG 14.2-49

# PRESSURIZER LEVEL VS TIME - SMALL BREAKS WITH AUXILIARY FEEDWATER





TMI - UNIT I

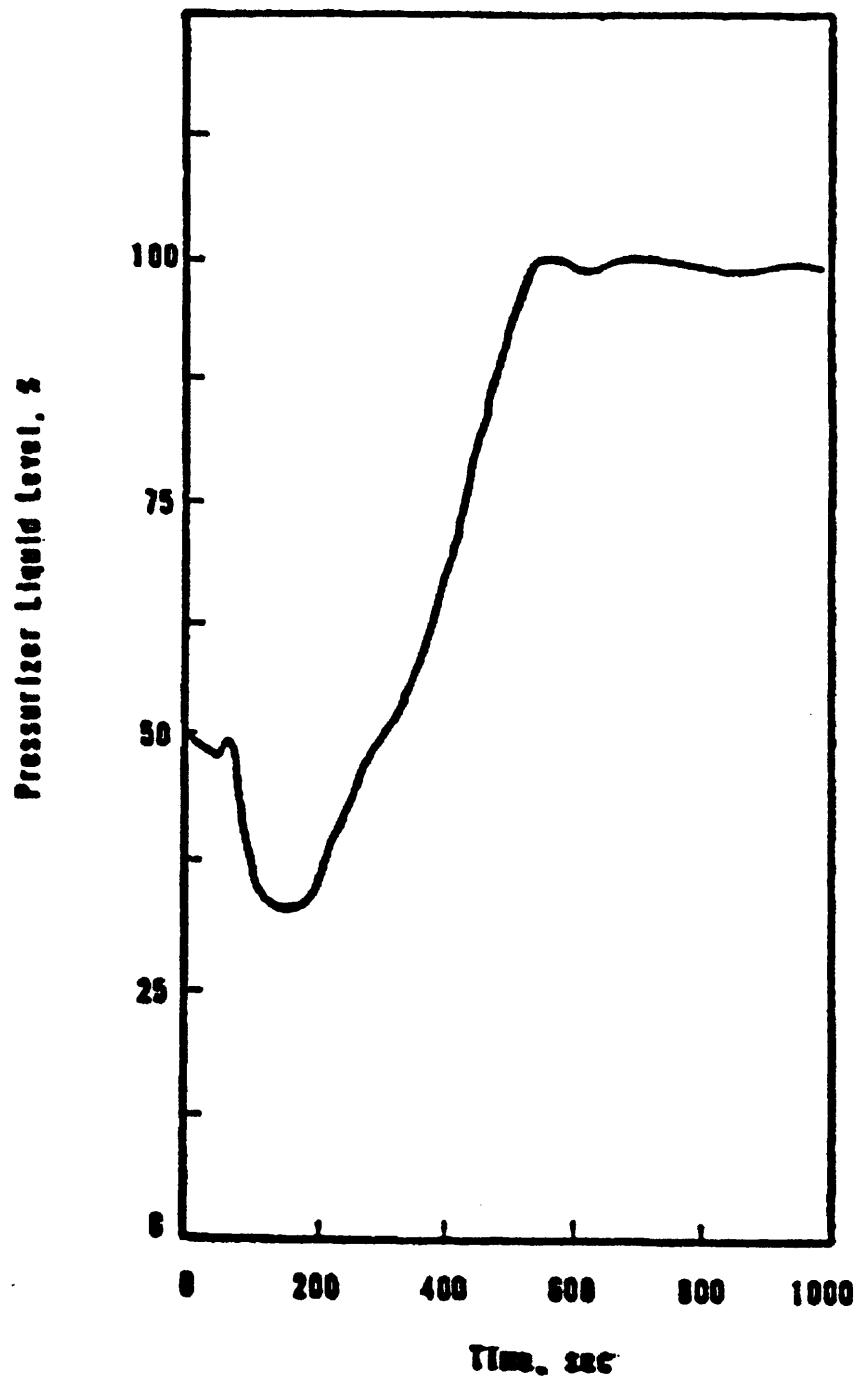
UPDATE - 14  
4/98


PRESSURE LEVEL VS TIME - SMALL BREAKS  
WITH EMERGENCY FEED WATER

CAD FILE: 6604RI4.DWG

FIG 14.2-50

PRESSURIZER LEVEL VS TIME FOR SMALL BREAK IN PRESSURIZER





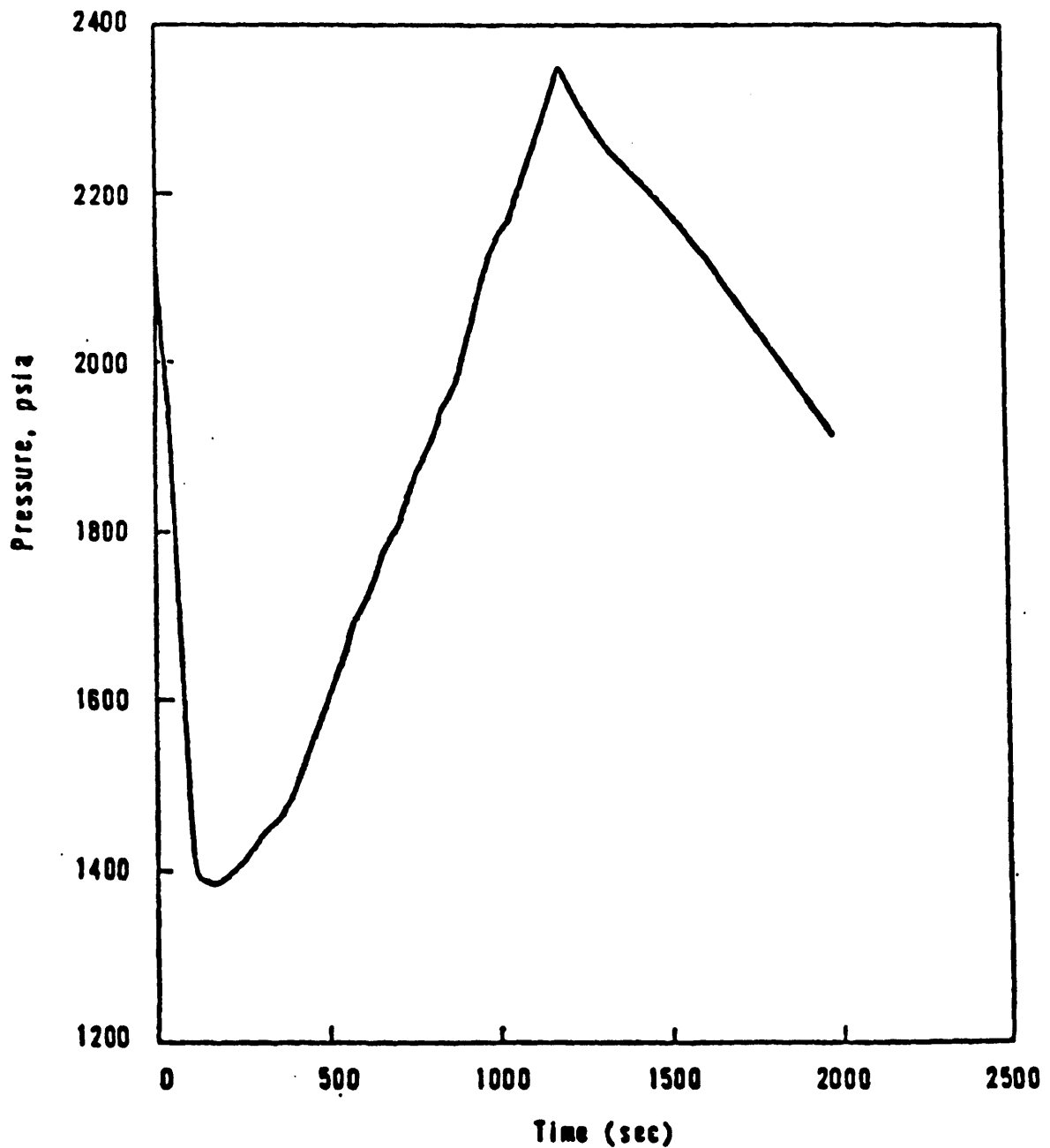
TMI - UNIT I

UPDATE - 14  
4/98

PRESSURIZER LEVEL VS. TIME - SMALL  
BREAK IN PRESSURIZER

CAD FILE: 6626RI4.DWG

FIG 14.2-51



TMI - UNIT 1

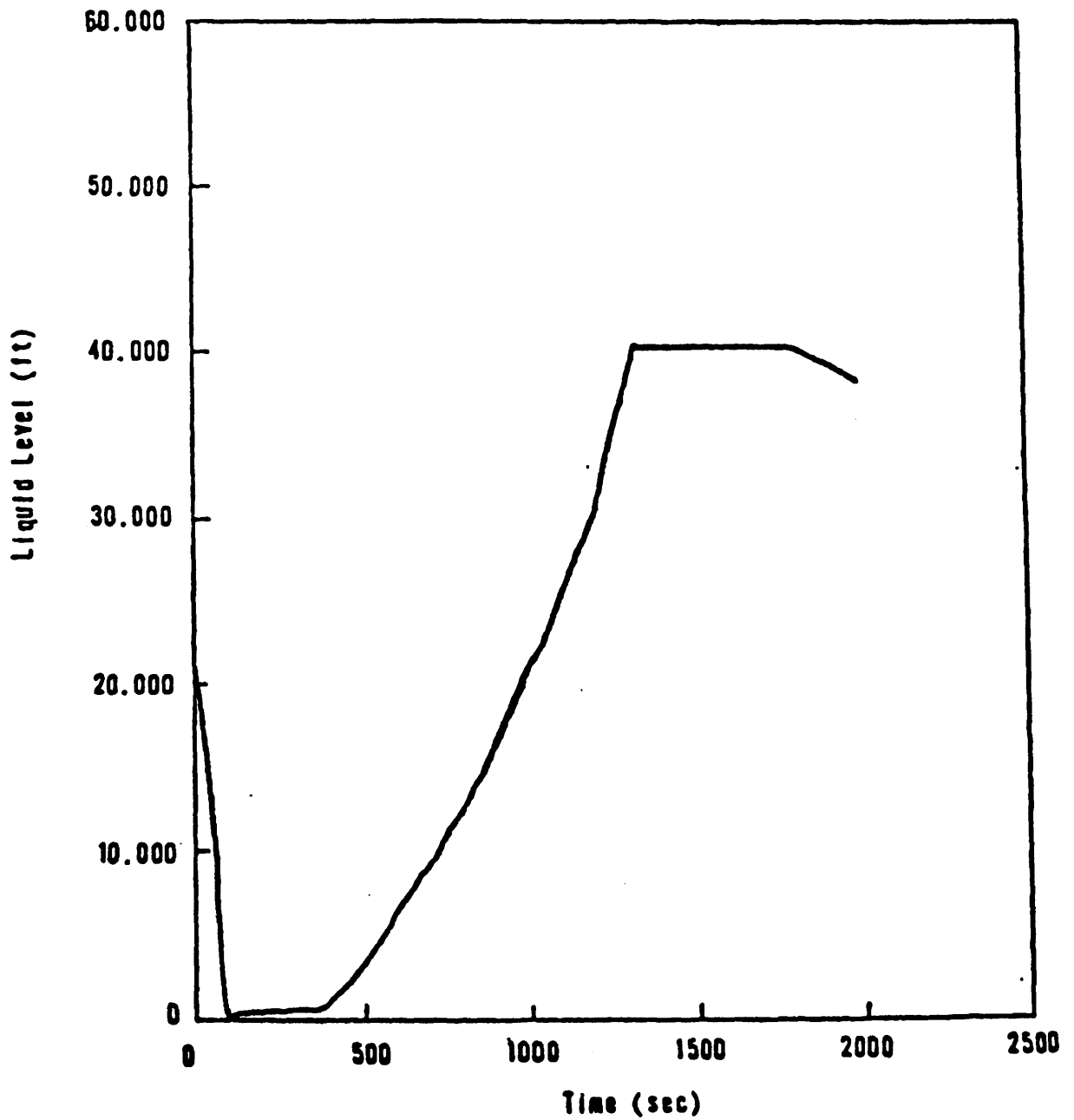
UPDATE - 14  
4/98

**GPU**  
**NUCLEAR**

0.01 FT<sup>2</sup> COLD LEG BREAK WITH NO EFW  
2 HPI'S & STUCK PORV AT 20 MIN-  
NODE 14 PRESSURE VS. TIME

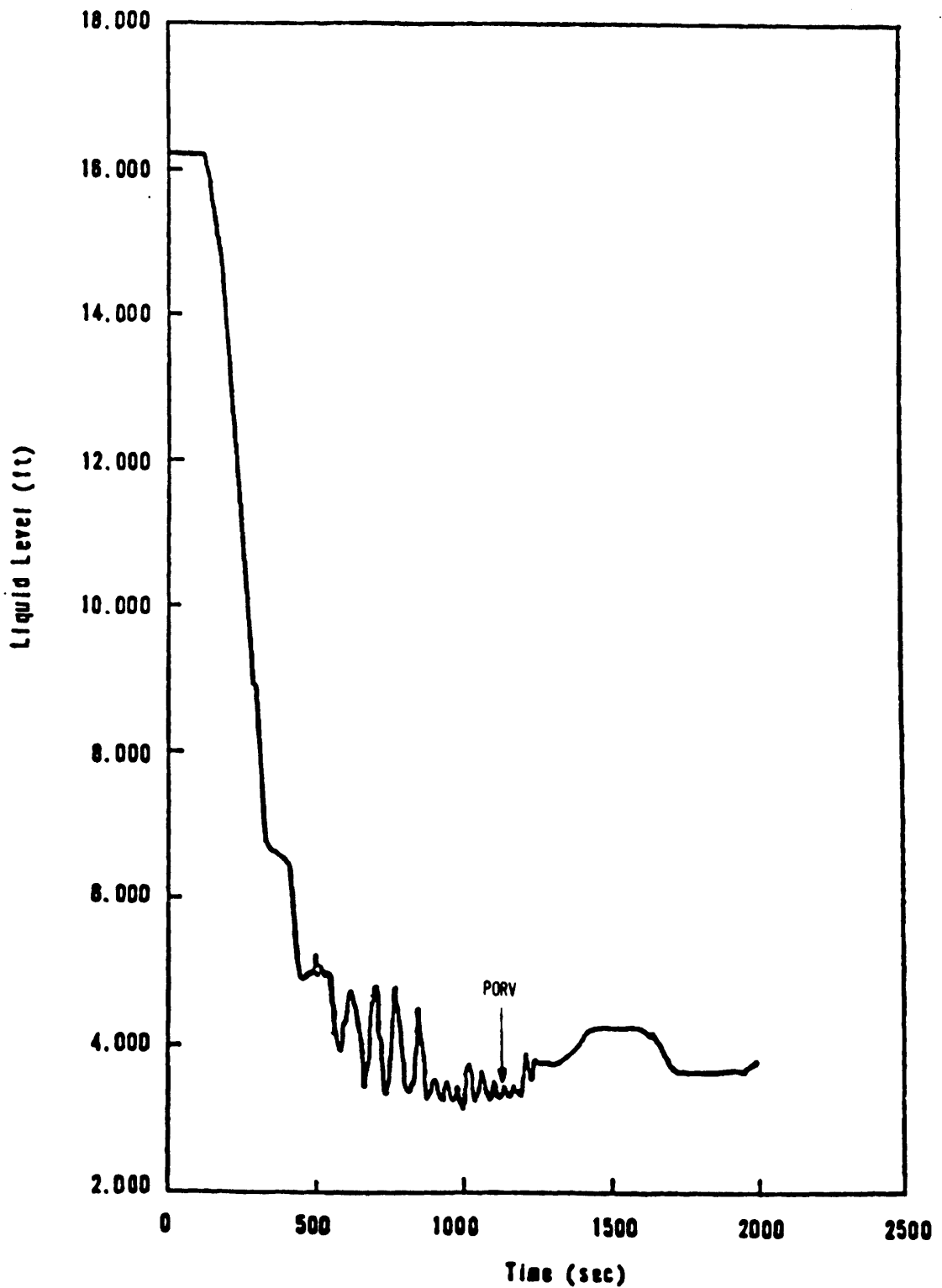
CAD FILE: 6627RI4.DWG


FIG 14.2-52



TMI - UNIT 1  
**GPU**  
**NUCLEAR**  
UPDATE - 14  
4/98  
0.01 FT<sup>2</sup> COLD LEG BREAK WITH NO EFW  
2 HPI'S & STUCK PORV AT 20 MIN-  
PRESSURIZER LIQUID LEVEL  
CAD FILE: 6628RI4.DWG  
FIG 14.2-53







**GPU  
NUCLEAR**

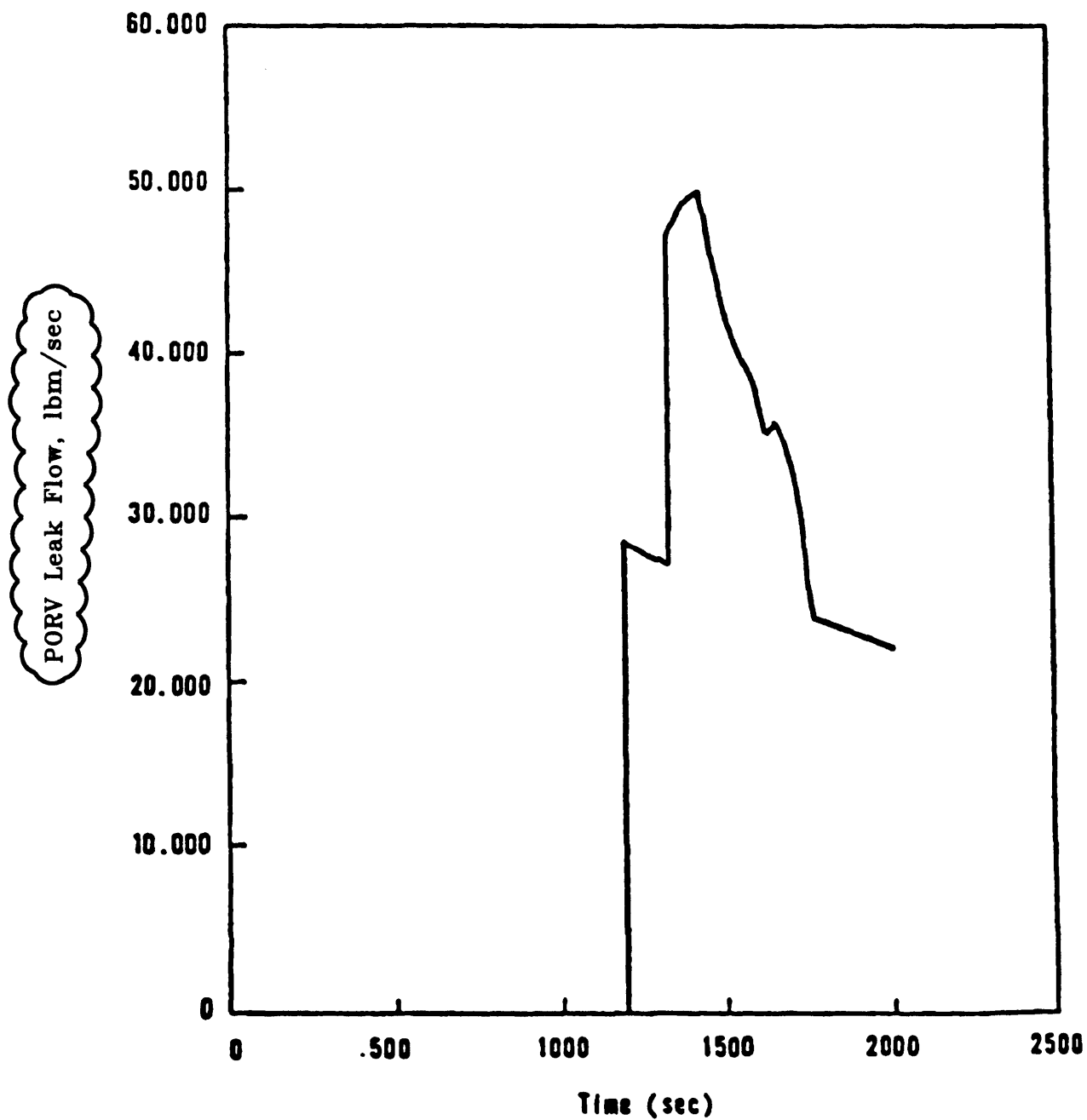
TMI - UNIT I

UPDATE - 14  
4/98

0.01 FT<sup>2</sup> COLD LEG BREAK WITH NO EFW  
2 HPI's & STUCK PORV AT 20 MIN-  
UPPER PLENUM LIQUID LEVEL

CAD FILE: 6629R14.DWG

FIG 14.2-54

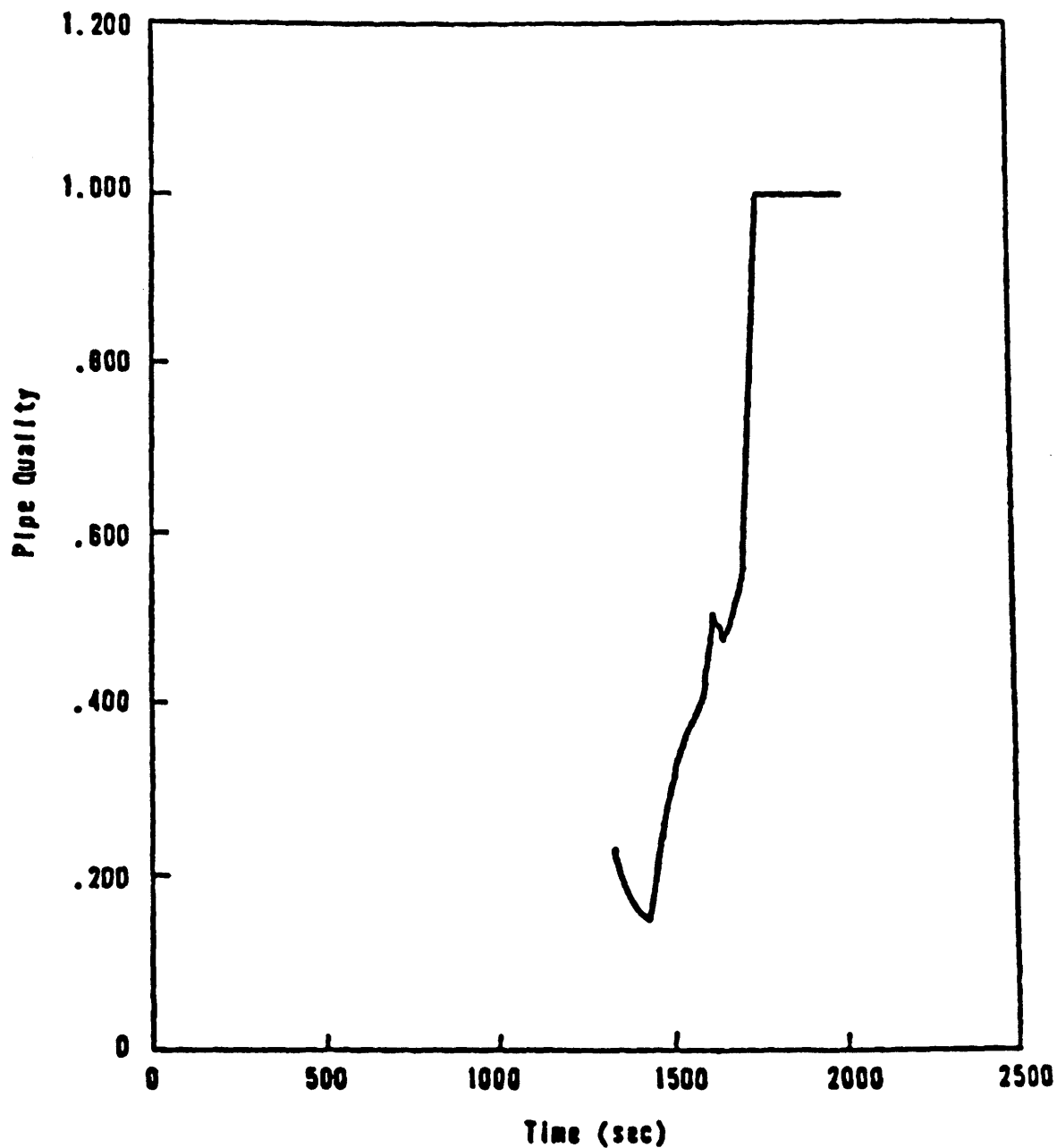



TMI - UNIT 1  
UPDATE - 14  
4/98

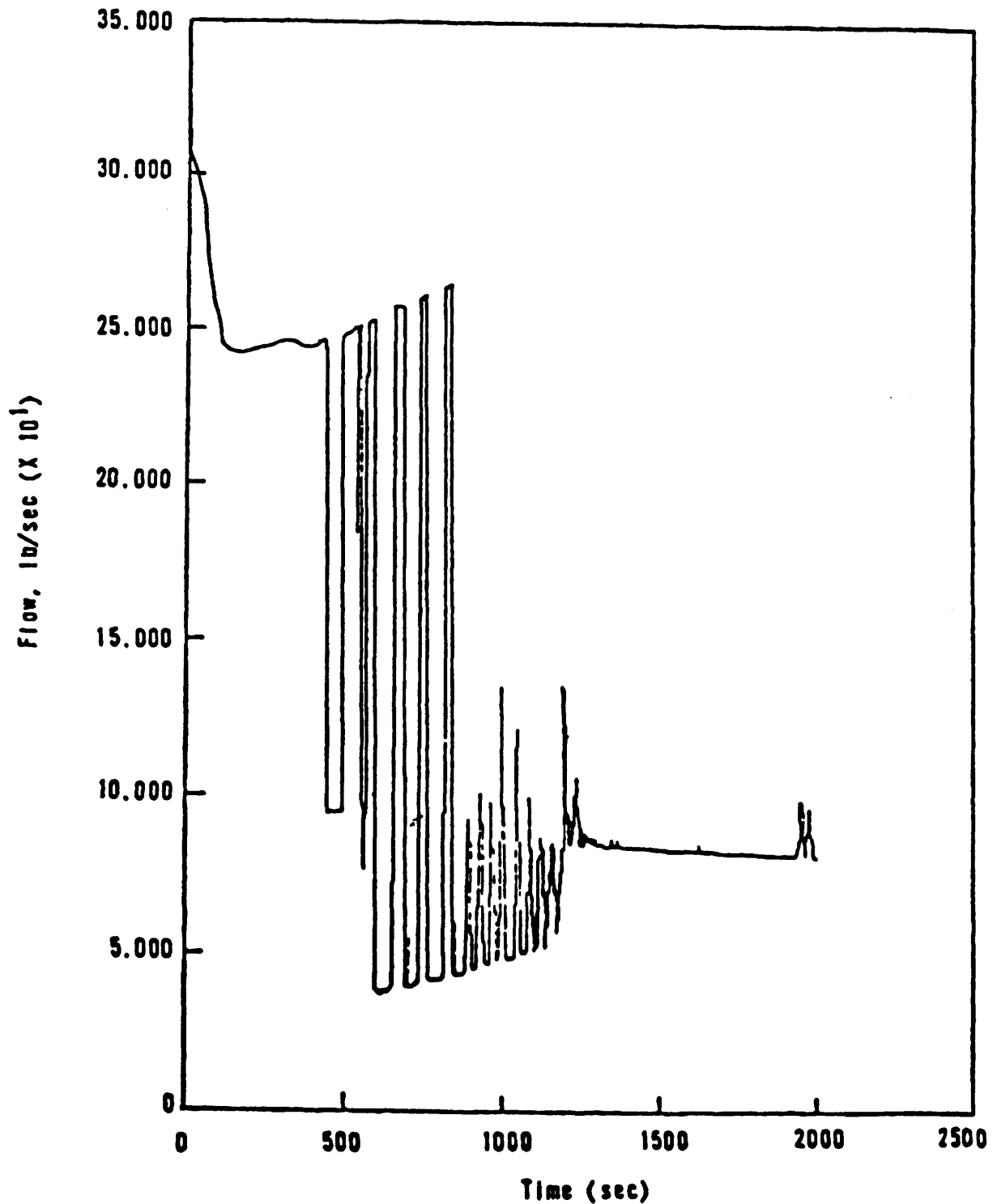
**GPU**  
**NUCLEAR**


0.01 FT<sup>2</sup> COLD LEG BREAK WITH NO EFW  
2 HPI'S & STUCK PORV AT 20 MIN-  
PORV LEAK FLOW

CAD FILE: 6613RI4.DWG  
FIG 14.2-55



	TMI - UNIT 1	UPDATE - 14
		4/98
0.01 FT <sup>2</sup> COLD LEG BREAK WITH NO EFW 2 HPI'S & STUCK PORV AT 20 MIN- PORV LEAK FLOW QUALITY		
CAD FILE: 6614RI4.DWG		FIG 14.2-56





**GPU  
NUCLEAR**

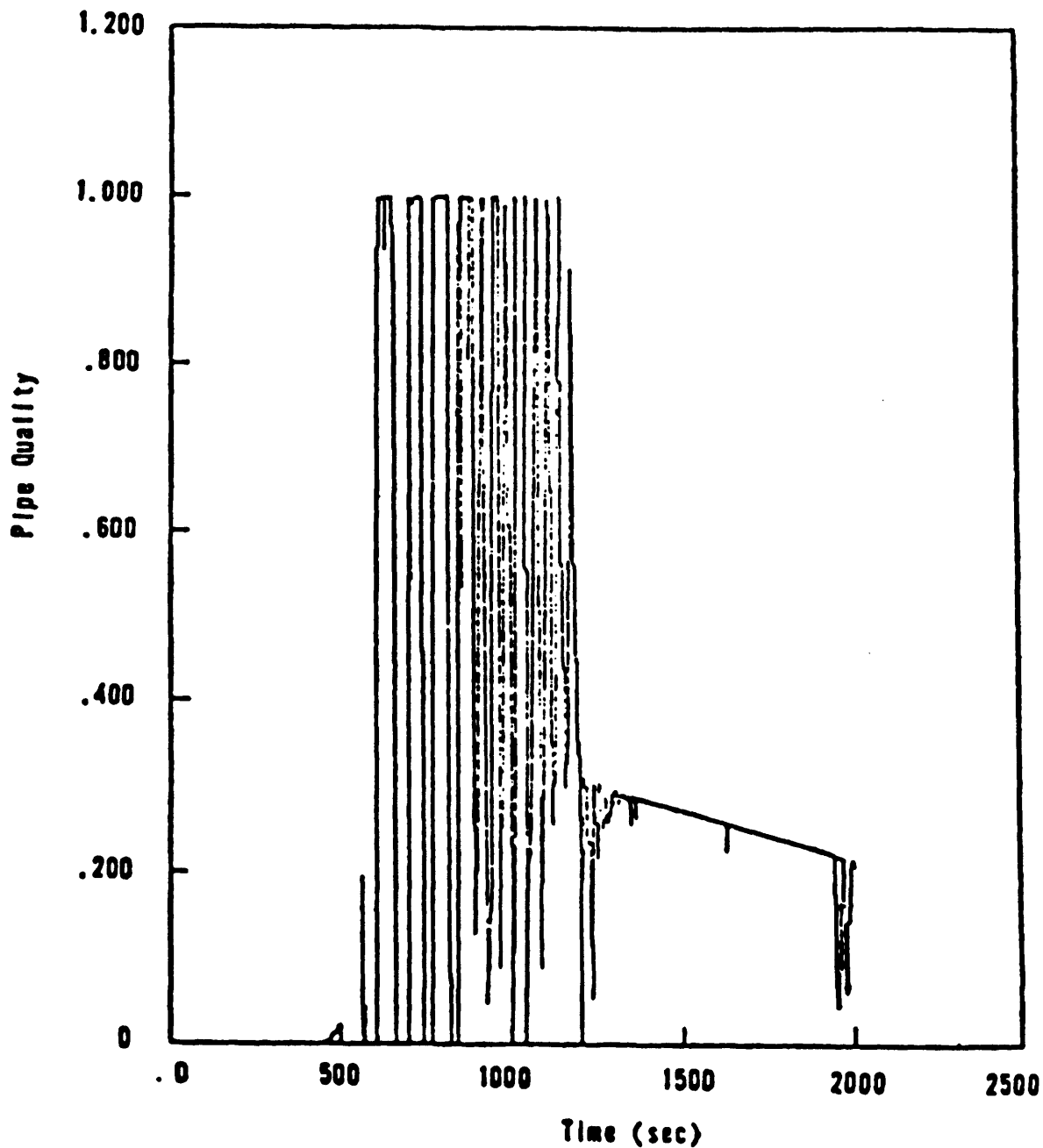
TMI - UNIT I


UPDATE - 14  
4/98

0.01 FT<sup>2</sup> COLD LEG BREAK WITH NO EFW  
2 HPI's & STUCK PORV AT 20 MIN-  
COLD LEG BREAK FLOW

CAD FILE: 6615RI4.DWG

FIG 14.2-57





**GPU  
NUCLEAR**

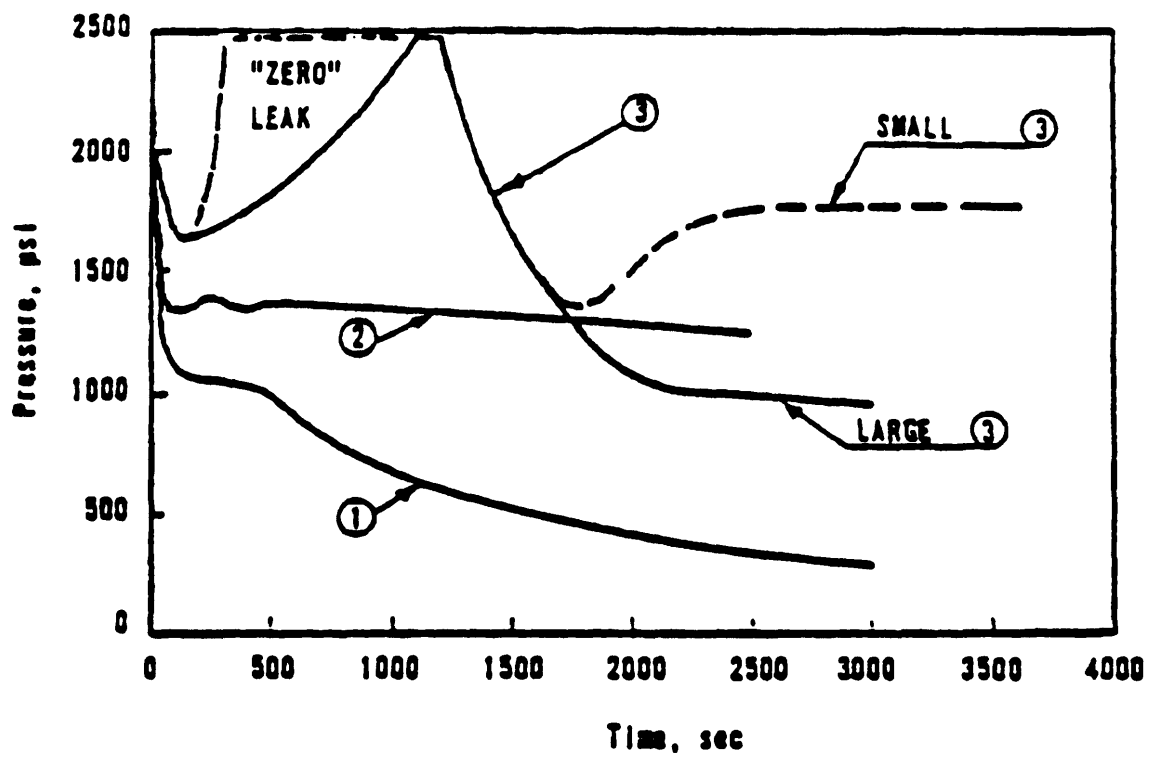
TMI - UNIT 1

UPDATE - 14  
4/98

0.01 FT<sup>2</sup> COLD LEG BREAK WITH NO EFW  
 2 HPI's & STUCK PORV AT 20 MIN-  
 COLD LEG BREAK LEAK FLOW QUALITY

CAD FILE: 6616RI4.DWG

FIG 14.2-58



TMI - UNIT I

**GPU**

**NUCLAR**

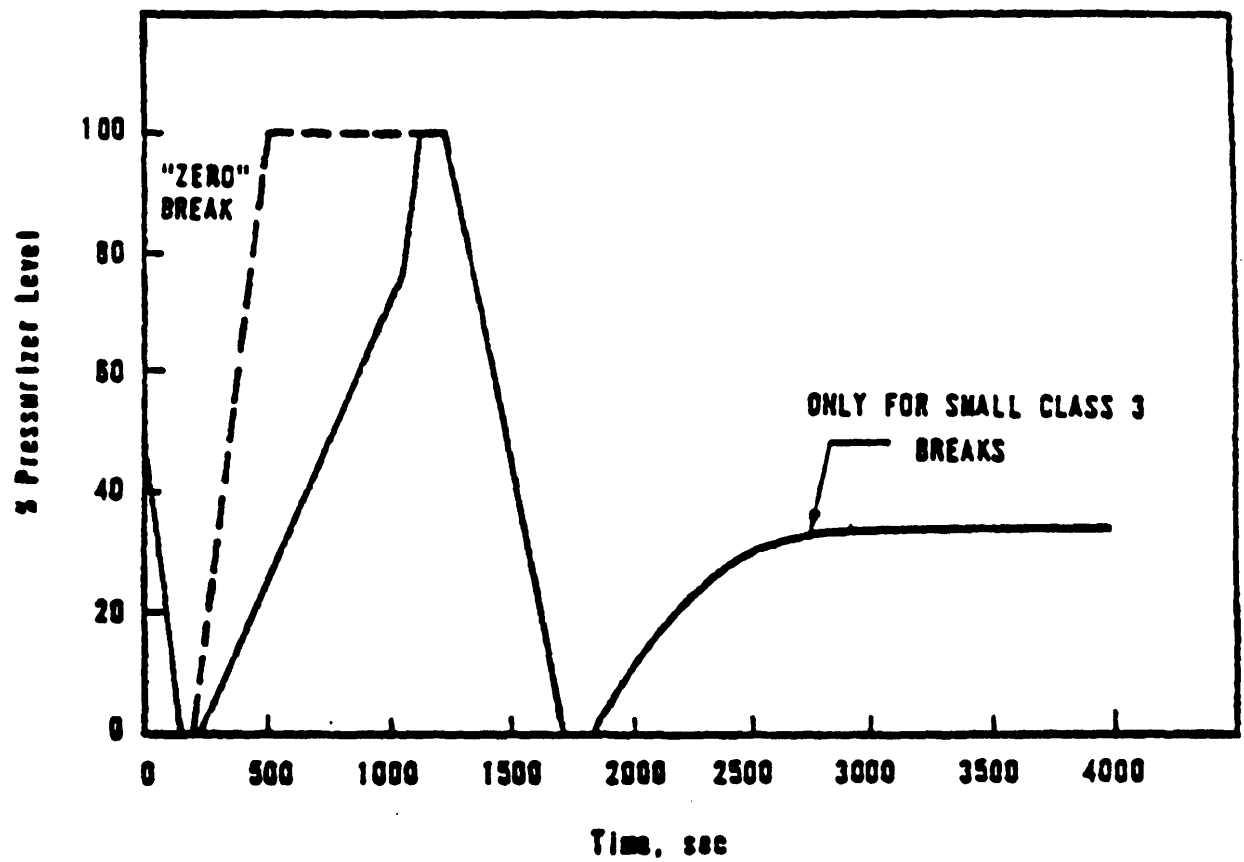
UPDATE - 14

4/98

SYSTEM PRESSURE VS. TIME - SMALL  
BREAKS W/O EMERGENCY FEEDWATER

CAD FILE: 6617RI4.DWG

FIG 14.2-59



TMI - UNIT I

**GPU**

**NUCLEAR**

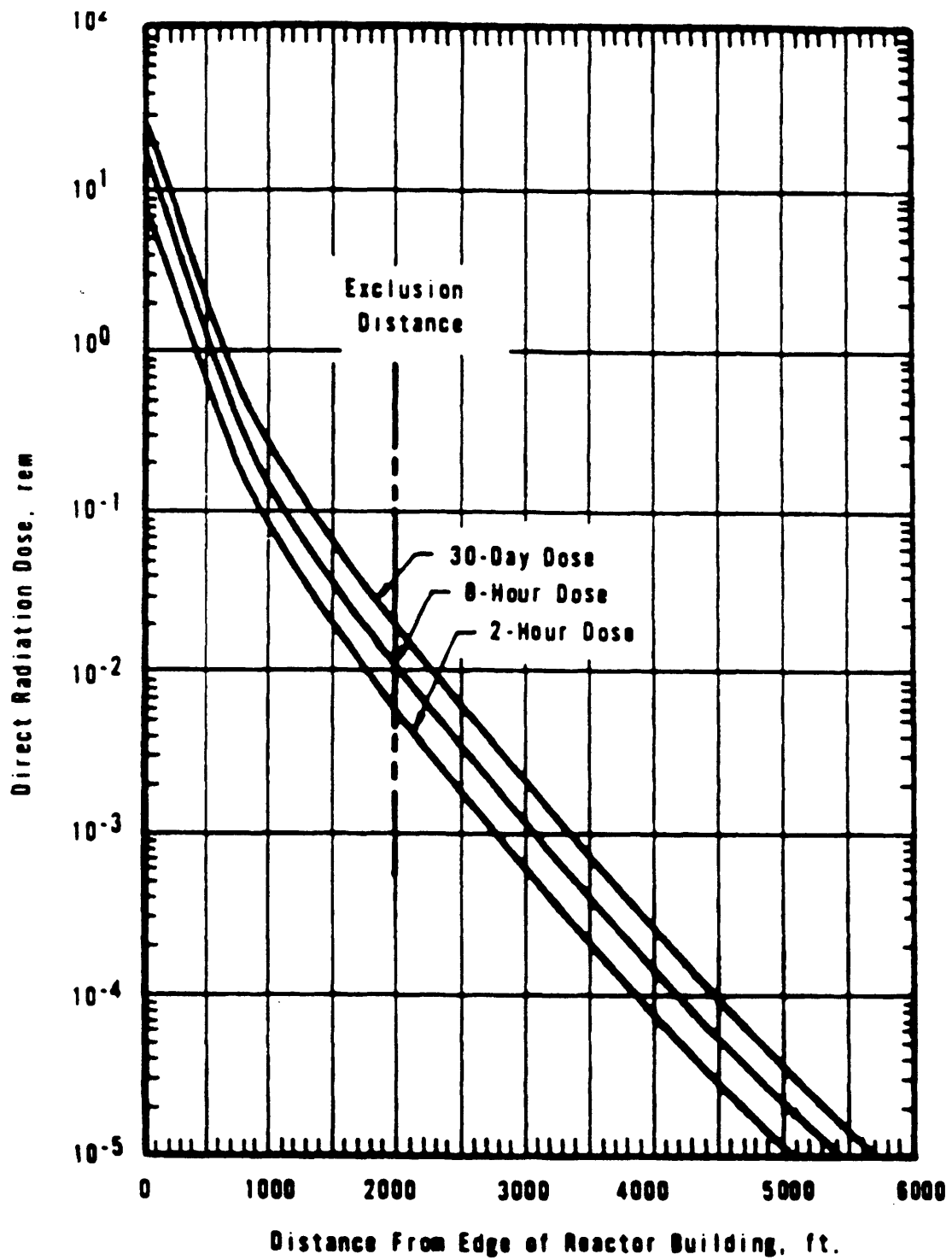
UPDATE - 14

4/98

PRESSURIZER LEVEL VS. TIME - CLASS 3 BREAKS W/O EMERGENCY FEEDWATER

CAD FILE: 6618RI4.DWG

FIG 14.2-60



TMI - UNIT 1

**GPU**

**NUCLEAR**

UPDATE - 14

4/98

INTEGRATED DIRECT DOSE FOLLOWING MHA WITH  
3-1/2 FT. REACTOR BUILDING WALL THICKNESS

CAD FILE: 6619R14.DWG

FIG 14.2-61



## TMI UFSAR

Figure 14.2-62

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