

1. Branch Technical Position RSB 5-1 "Design Requirements of the Residual Heat Removal System" attached to NUREG-800, Standard Review Plan, Section 5.4.7, RESIDUAL HEAT REMOVAL (RHR) SYSTEM, Draft Rev 4, April 1996
2. Letter from Crutchfield of NRC to Hoffman of Consumers dated October 27, 1981, "Palisades - SEP Topics V-10.B: RHR System Reliability, V-11.B, RHR Interlock Requirements, and VII-3, Systems Needed for Safe Shutdown (Safe Shutdown Systems Report)"
3. NUREG 0820, Integrated Plant Safety Assessment, Systematic Evaluation Program, Palisades Plant, Consumers Power Company, Final Report, October 1982, transmitted by NRC letter dated October 29, 1982
4. NUREG 1424, Safety Evaluation Report related to the full-term operating license for Palisades Nuclear Plant, Docket 50-255, Consumers Power Company, November 1990, transmitted by NRC letter dated November 21, 1990
5. NRC Office Instruction LIC-100, Control of Licensing Bases for Operating Reactors, Revision 00-a effective March 2, 2001
6. Letter from Crutchfield of NRC to Hoffman of Consumers dated November 3, 1981, "Palisades – SEP Topics – Design Basis Events (Evaluation of Design Basis Events: Accidents and Transients)"
7. NRC Information Notice 93-17, Revision 1, "Safety Systems Response to Loss of Coolant and Loss of Offsite Power"
8. NRC Information Report, SECY-77-439, August 17, 1977, "Single Failure Criteria"
9. Condition Report C-PAL-02-0263, "FSAR Lacks Clarity With Respect to Identification of Active and Passive Failures"
10. Winston & Strawn, et. al., "Generic Implementation Procedure (GIP) for Seismic Verification of Nuclear Plant Equipment", Prepared for the Seismic Qualification Utility Group SQUG, Revision 2A, March 1993
11. Report of SQUG Assessment at Palisades Nuclear Plant for the Resolution of USI A-46, dated May 19, 1995
12. Safety Evaluation for "Resolution of Unresolved Safety Issue (USI) A-46, Verification of Seismic Qualification of Equipment in Operating Plants (TAC No M69468)," dated September 25, 1998

13. Letter from Douglas E. Cooper to the USNRC Document Control Desk, Docket No. 50-255, License No. DPR-20 entitled, "Final Closeout of Unresolved Safety Issue A-46 Outliers," dated June 26, 2003
14. Engineering Analysis EA-MOD-2004-010-16, "Palisades Head RT-ndt Determination"
15. Letter from Crutchfield (NRC) to VandeWalle (Consumers) dated November 9, 1983, "Control of Heavy Loads (Phase I) – NUREG-0612"
16. NRC Generic Letter 85-11, "Completion of Phase II of 'Control of Heavy Loads at Nuclear Power Plants' NUREG-0612," dated June 28, 1985
17. 10 CFR 50.49, "Environmental Qualification of Electric Equipment Important to Safety for Nuclear Power Plants"
18. 10 CFR 50, APPENDIX G, "FRACTURE TOUGHNESS REQUIREMENTS"
19. 10 CFR 50, Appendix H, "Reactor Vessel Material Surveillance Program Requirements"
20. 10 CFR 50.55a, "Codes and Standards"
21. 10 CFR 50.59, "Changes, Tests and Experiments"
22. 10 CFR 50.61, "Fracture Toughness Requirements for Protection Against Pressurized Thermal Shock Events"
23. 10 CFR 50.65, "Requirements for Monitoring the Effectiveness of Maintenance at Nuclear Power Plants"
24. 10 CFR 54, "Requirements for Renewal of Operating Licenses for Nuclear Power Plants"
25. 10 CFR 54.21, "Contents of Application--Technical Information"
26. ACI 318-63
27. ANSI/ASME Standard B30.2 (1996).
28. ANSI B31.1 (1973) Power Piping Code with the Summer (1973) Addenda
29. American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code (BPV Code), Section III, Subsection 4 for Class A vessels, 1965, with addenda through Winter, 1965

30. ASME Section III, Class A, 1965
31. ASME Section III, Class A, 1965, Winter 1966 addenda
32. ASME Section III, 1977
33. ASME Section III, 1989
34. ASME BPV Code Section XI, "Rules for Inservice Inspection of Nuclear Power Plant Components"
35. ASME NOG 1, Section NOG 4140, "Load Combinations"
36. American Society for Testing Materials (ASTM) Standards D 1796, D 2276, D 2709, and D 4057
37. American Standard (ASA) Code for Pressure Piping, Section 1, "Power Piping Systems," 1955 edition
38. Combustion Engineering Owners Group Report SIR-94-080
39. Crane Manufacturers Association of America Standard CMAA-70, "Service Level A - Standby or Infrequent Service"
40. IE Bulletin 79-14, "Seismic Analyses for As-Built Safety-Related Piping Systems"
41. USAS B31.1.0 (1967) Power Piping Code
42. Power Piping Code with the Summer (1973) Addenda
43. Combustion Engineering Report NPSD-993
44. EPRI NP-5769, "Degradation and Failure of Bolting in Nuclear Power Plants"
45. EPRI TR-102134, "PWR Secondary Water Chemistry Guidelines"
46. EPRI TR-104213, "Bolted Joint Maintenance & Applications Guide"
47. EPRI TR-105714, "PWR Primary Water Chemistry Guidelines"
48. EPRI TR-107396, "Closed Cooling Water Chemistry Guideline"
49. Electric Overhead Crane Institute Specification 61
50. FSAR Section 4.3.7

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53. FSAR Section 5.10.1.1
54. Generic Safety Issue (GSI) 190, "Fatigue Evaluation of Metal Components for 60-Year Plant Life"
55. NEI 96-03, "Guideline for Monitoring the Condition of Structures at Nuclear Power Plants"
56. Nuclear Energy Institute (NEI) 97-06, "Steam Generator Program Guidelines"
57. NMC to NRC, Palisades Nuclear Plant Application for Renewed Operating Licenses, dated March 22, 2005.
58. NRC Bulletin 88-08, "Thermal Stresses in Piping Connected to Reactor Cooling Systems"
59. Revised NRC Bulletin 88-11, "Pressurizer Surge Line Stratification"
60. NRC Generic Letter 88-05, "Boric Acid Corrosion of Carbon Steel Reactor Pressure Boundary Components in PWR Plants"
61. NRC Generic Letter 89-13, "Service Water System Problems Affecting Safety-Related Equipment"
62. NRC Letter to Paul A. Harden, Nuclear Management Company, LLC, "Issuance of Renewed Facility Operating License No. DPR-20 for Palisades Nuclear Plant," dated January 17, 2007
63. NSAC-202L-R3, "Recommendations for an Effective Flow-Accelerated Corrosion Program"
64. NUMARC 93-01, "Industry Guideline for Monitoring the Effectiveness of Maintenance at Nuclear Power Plants"
65. NUREG-0612, "Control of Heavy Loads at Nuclear Power Plants"
66. NUREG-1339, "Resolution of Generic Safety Issue 29: Bolting Degradation or Failure in Nuclear Power Plants"
67. NUREG-1800, "Standard Review Plan for Review of License Renewal Applications for Nuclear Power Plants"

68. NUREG-1801, "Generic Aging Lessons Learned (GALL) Report"
69. NUREG-1871, "Safety Evaluation Report Related to the License Renewal of the Palisades Nuclear Plant"
70. NUREG/CR-5704, "Effects of LWR Coolant Environments on Fatigue Design Curves of Austenitic Stainless Steels"
71. NUREG/CR-5999, "Interim Fatigue Design Curves for Carbon, Low-Alloy, and Austenitic Stainless Steels in LWR Environments"
72. NUREG/CR-6260, "Application of NUREG/CR-5999 Interim Fatigue Curves to Selected Nuclear Power Plant Components"
73. NUREG/CR-6583, "Effects of LWR Coolant Environments on Fatigue Design Curves of Carbon and Low-Alloy Steels"
74. Regulatory Guide 1.14, "Reactor Coolant Pump Flywheel Integrity"
75. Regulatory Guide 1.99, "Radiation Embrittlement of Reactor Vessel Materials"
76. Regulatory Guide 1.160, "Monitoring the Effectiveness of Maintenance at Nuclear Power Plants"
77. RIS 2003-09, "Environmental Qualification of Low-Voltage Instrumentation and Control Cables"
78. Palisades Technical Specification 5.5.5
79. USAS B31.1.0 (1967) Power Piping Code
80. WCAP-15338-A, "A Review of Cracking Associated with Weld Deposited Cladding in Operating Pressurized Water Reactor (PWR) Plants"
81. WCAP-15973-P, "Low Alloy Steel Component Corrosion Analysis Supporting Alloy 600/690 Nozzle Repair Program"
82. NMC to NRC, Supplementary Information for the Palisades Application for Renewed Operating License Resulting from Aging Management Programs Audit, dated August 25, 2005.
83. NMC to NRC, Response to NRC Request for Additional Information Related to License Renewal dated June 3, 2005, dated July 1, 2005.

84. NMC to NRC, NMC Responses to NRC Requests for Additional Information Relating to License Renewal dated August 23, August 26, and August 31, dated September 16, 2005.
85. NMC to NRC, NMC Responses to NRC Requests for Additional Information Dated June 22, 2005 Relating to License Renewal for the Palisades Nuclear Plant, dated July 25, 2005.
86. NMC to NRC, Response to NRC Request for Additional Information Related to License Renewal dated October 24, 2005, dated November 18, 2005.
87. NMC to NRC, NMC Responses to NRC Requests for Additional Information Relating to License Renewal dated March 8, 2006, dated March 30, 2006.
88. NMC Responses to NRC Follow Up Questions Relating to License Renewal, dated April 26, 2006.
89. Letter from DP Hoffman (CPCo) to DM Crutchfield (NRC), "Control of Heavy Loads (Generic Letter 81-07)," dated July 6, 1981.
90. Letter from BD Johnson (CPCo) to DM Crutchfield (NRC), "Control of Heavy Loads (Generic Letter 81-07)," dated September 23, 1981.
91. NRC Bulletin 96-02: Movement of Heavy Loads Over Spent Fuel, Over Fuel in the Reactor Core, or Over Safety-Related Equipment, dated April 11, 1996.
92. Letter from RW Smedley (CPCo) to NRC, Response to NRC Bulletin 96-02: "Movement of Heavy Loads Over Spent Fuel, Over Fuel in the Reactor Core, or Over Safety-Related Equipment," dated May 13, 1996.
93. Letter from RG Schaaf (NRC) to NL Haskell (CPCo), Palisades Plant - Completion of Action for NRC Bulletin 96-02, Movement of Heavy Loads Over Spent Fuel, Over Fuel in the Reactor Core, or Over Safety-Related Equipment, dated April 21, 1998.
94. Letter from CJ Schwarz (ENO) to NRC, Revision to License Renewal Commitments, dated September 1, 2010.
95. Palisades License Amendment 236, Spent Fuel Pool Region I Storage Requirements, dated February 6, 2009.
96. LR-TR-025-Fatigue, Stress and Fatigue Evaluations for the License Renewal Period.

97. Engineering Analysis EA-EC27959-01, "Palisades Pressure-Temperature Limit Curves and Upper-Shelf Energy Evaluation," February 2012.
98. Palisades License Amendment No. 245, Primary Coolant System Pressure-Temperature Limits (TAC No. ME5806), dated January 19, 2012.
99. Engineering Analysis EA-EC26115-01, Revision 0, "Pressurized Thermal Shock PTS Evaluation for the Palisades Reactor Pressure Vessel," February 2012.
100. Letter from MChawla (NRC) to Vice President Operations Entergy-Palisades, "Updated Reactor Pressure Vessel Pressurized Thermal Shock Evaluation for Palisades Nuclear Plant (TAC No. ME5263)," December 7, 2011 (contains the Safety Evaluation for the updated PTS evaluation).
101. Letter from CJ Schwarz (ENO) to NRC, License Renewal Commitment to Submit Alloy 600 Program, dated March 13, 2008.
102. Letter from HM Jones (NRC) to Anthony Vitale (ENO), Palisades Nuclear Plant Safety Evaluation of Commitment Submittal for License Renewal Regarding the Alloy 600 Program (TAC No. ME7125), dated August 15, 2012.
103. NRC letter, "Palisades Nuclear Plant – Issuance of Amendment Regarding Replacement of Spent Fuel Pool Region I Storage Racks," dated February 28, 2012.
104. Letter from Entergy Nuclear Operations, Inc. to NRC, "License Amendment Request for Approval of Palisades Nuclear Plant 10 CFR 50 Appendix G Equivalent Margins Analysis," dated November 12, 2014.
105. NRC Amendment No. 258 to Facility Operating License No. DPR-20 for the Palisades Plant, Subject: "Palisades Nuclear Plant - Issuance of Amendment Re: Request for Approval of 10 CFR Part 50 Appendix G Equivalent Margins Analysis (CAC No. MF5163)," dated November 23, 2015.
106. Letter from Entergy Nuclear Operations, Inc. to NRC, "License Amendment Request for Primary Coolant System Pressure-Temperature Limits," dated March 7, 2011.
107. NRC letter, "Palisades Nuclear Plant - Staff Assessments Re: Revised Program Plan for Aging Management of Reactor Vessel Internals (TAC No. ME9569)," dated December 11, 2014 (ADAMS Accession No. ML14336A397).
108. 10 CFR 50.61a, "Alternate Fracture Toughness Requirements for Protection Against Pressurized Thermal Shock Events"

109. Entergy Nuclear Operations, Inc. letter PNP 2014-049, "License Amendment Request to implement 10 CFR 50.61a, 'Alternate Fracture Toughness Requirements for Protection Against Pressurized Thermal Shock Events,' dated July 29, 2014 (ADAMS Accession No. ML14211A520).
110. WCAP-17628-NP, "Alternate Pressurized Thermal Shock (PTS) Rule Evaluation for Palisades," Revision 1, June 2014 (Palisades Engineering Report PLP-RPT-16-00005).
111. NRC letter, "Palisades Nuclear Plant - Issuance of Amendment Re: License Amendment Request to Implement 10 CFR 50.61a, 'Alternate Fracture Toughness Requirements for Protection Against Pressurized Thermal Shock Events,' dated November 23, 2015 (ADAMS Accession No. ML15209A791).