

NRR-PMDAPEm Resource

From: Purnell, Blake
Sent: Wednesday, March 16, 2016 10:47 AM
To: Loomis, Thomas R:(GenCo-Nuc) (thomas.loomis@exeloncorp.com)
Cc: Poole, Justin; James.Barstow@exeloncorp.com
Subject: Calvert Cliffs, Nine Mile Point, and R. E. Ginna - Request to Use ASME Code Case N-789 as an Alternative to ASME Code Requirements (CAC Nos. MF7018-MF7022)
Attachments: MF7018 Exelon LC Code Case N-789 RAI.docx

Mr. Loomis:

By application dated October 28, 2015 (ADAMS Accession No. ML15301A596), Exelon Generation Company, LLC (the licensee) submitted a request in accordance with Paragraph 50.55a(z)(2) of Title 10 of the *Code of Federal Regulations* (10 CFR) for a proposed alternative to the requirements of Section XI, "Rules for Inservice Inspection of Nuclear Power Plant Components," of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code (ASME Code) for Calvert Cliffs Nuclear Power Plant, Units 1 and 2; Nine Mile Point Nuclear Station, Units 1 and 2; and R. E. Ginna Nuclear Power Plant. The proposed alternative would allow the licensee to use ASME Code Case N-789, "Alternative Requirements for Pad Reinforcement of Class 2 and 3 Moderate-Energy Carbon Steel Piping for Raw Water Service, Section XI, Division 1," in lieu of specified ASME Code requirements.

The U.S. Nuclear Regulatory Commission staff has reviewed the application and determined that it needs additional information to completed its review. A response to the attached request for additional information shall be provided within 30 days from the date of this email.

If you have any questions please contact me at 301-415-1380 or by email.

Sincerely,

Blake Purnell, Project Manager
Plant Licensing Branch III-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission

Docket Nos. 50-317, 50-318, 50-220, 50-410, and 50-244

Hearing Identifier: NRR_PMDA
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Subject: Calvert Cliffs, Nine Mile Point, and R. E. Ginna - Request to Use ASME Code Case N-789 as an Alternative to ASME Code Requirements (CAC Nos. MF7018-MF7022)

Sent Date: 3/16/2016 10:46:32 AM

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From: Purnell, Blake

Created By: Blake.Purnell@nrc.gov

Recipients:

"Poole, Justin" <Justin.Poole@nrc.gov>

Tracking Status: None

"James.Barstow@exeloncorp.com" <James.Barstow@exeloncorp.com>

Tracking Status: None

"Loomis, Thomas R:(GenCo-Nuc) (thomas.loomis@exeloncorp.com)" <thomas.loomis@exeloncorp.com>

Tracking Status: None

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MESSAGE	1508	3/16/2016 10:46:00 AM
MF7018 Exelon LC Code Case N-789 RAI.docx		27827

Options

Priority: Standard

Return Notification: No

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REQUEST FOR ADDITIONAL INFORMATION

PROPOSED ALTERNATIVE TO USE CODE CASE N-789

CALVERT CLIFFS NUCLEAR POWER PLANT, UNITS 1 AND 2

NINE MILE POINT NUCLEAR STATION, UNITS 1 AND 2

R.E. GINNA NUCLEAR POWER PLANT

DOCKET NOS. 50-317, 50-318, 50-220, 50-410, AND 50-244

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The U.S. Nuclear Regulatory Commission (NRC) staff has reviewed the application and determined that the following information is needed for the staff to complete its review.

RAI-1

Section 3.1(a)(1) of Code Case N-789 states: "Pressure pads are designed to retain pressure, and may be used only where the piping is predicted to retain full structural integrity until the next refueling outage assuming a corrosion rate of either 2 times the actual measured corrosion rate in that location, or 4 times the estimated maximum corrosion rate for the system."

For the pressure pad design, confirm that the higher of two times the actual measured corrosion rate or four times the estimated maximum corrosion rate will be used. Confirm that if the actual measured corrosion rate in the degraded location is unavailable, the estimated maximum corrosion rate for the system assumed in the design is calculated based on the same degradation mechanism as the degraded location. If this is not the case, describe how the corrosion rate will be determined and provide justification for not using the worst corrosion rate for the pressure pad design.

ENCLOSURE