

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 1**  
(1 point)

---

Given the following:

- The plant is at 100% power
- The “A” Reactor Trip Bypass Breaker has been placed in service for testing
- The “A” Reactor Trip Breaker has been removed from service
- A transient condition exists requiring a reactor trip
- The OAC depresses the manual reactor trip pushbutton prior to an RPS setpoint being reached

Which ONE of the following describes the plant response?

The “A” Reactor Trip Bypass Breaker opens due to actuation of its \_\_\_\_\_ (1) \_\_\_\_\_ and the “B” Reactor Trip Breaker opens due to actuation of its \_\_\_\_\_ (2) \_\_\_\_\_.

- A. (1) UV trip, ONLY  
(2) UV trip, ONLY
  - B. (1) UV trip, ONLY  
(2) UV and shunt trip
  - C. (1) UV and shunt trip  
(2) UV trip, ONLY
  - D. (1) UV and shunt trip  
(2) UV and shunt trip
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 2**  
(1 point)

---

Given the following:

- The plant is operating at 100% power
- A transient results in both Pressurizer PORVs opening and sticking OPEN after pressure lowers below the OPEN setpoint
- When the operator attempts to close the Block Valves for each PORV, the OATC observes that the status lights for Block Valves RC-535 and RC-536 are all EXTINGUISHED

Subsequently, the CRS directs that the power to each Block Valve be restored.

RC-535, Block Valve to Pressurizer PORV PCV-456  
RC-536, Block Valve to Pressurizer PORV PCV-455C

Which ONE of the following identifies the location(s) to which the operator must be dispatched to close the breakers for the Block Valves?

The operator must proceed to.....

- A. MCC-5 for BOTH RC-535 and RC-536
  - B. MCC-5 for RC-535; and  
MCC-6 for RC-536
  - C. MCC-5 for RC-536; and  
MCC-6 for RC-535
  - D. MCC-6 for BOTH RC-535 and RC-536
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 3**  
(1 point)

---

Given the following:

- The plant is at 100% power
- "A" CV Spray pump is out of service for maintenance

Subsequently:

- A SBLOCA occurred at 0930 coincident with a Loss of Offsite Power
- "B" EDG tripped immediately after starting
- At 1000, the crew is performing the actions of EOP-E-1, LOSS OF REACTOR OR SECONDARY COOLANT with the following current plant conditions:
  - CV pressure: 11 psig and rising slowly
  - CV Sump level: 178 inches and rising
  - CV radiation level: 0.5 R/hr
  - "A" Loop Cold Leg temperature: 338 °F
  - "B" Loop Cold Leg temperature: 425 °F
  - "C" Loop Cold Leg temperature: 427 °F

Which ONE of the following correctly completes the statements below?

CSF-4 Integrity critical safety function status tree color is \_\_\_\_ (1) \_\_\_\_.

CSF-5 Containment critical safety function status tree color is \_\_\_\_ (2) \_\_\_\_.

- A. (1) Yellow  
(2) Yellow
  - B. (1) Orange  
(2) Yellow
  - C. (1) Yellow  
(2) Orange
  - D. (1) Orange  
(2) Orange
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 4**  
(1 point)

---

Given the following:

- With the plant at 100% power a Large Break LOCA occurs

Which ONE of the following correctly completes the statements below?

IAW EOP-ES-1.3, TRANSFER TO COLD LEG RECIRCULATION entry conditions, the HIGHEST allowable RWST level before transfer to Cold Leg Recirculation is required is (1).

The reason that the above switchover criteria is established is to ensure that core flow can be maintained using one (2) pump taking suction on the RWST during realignment to the CV sump.

- A. (1) 19%  
(2) SI
  - B. (1) 19%  
(2) RHR
  - C. (1) 27%  
(2) SI
  - D. (1) 27%  
(2) RHR
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 5**  
(1 point)

---

Given the following:

- The plant is at 100% power
- The “A” Charging pump is operating in AUTO
- The following annunciators alarm in the control room, all within 30 seconds:
  - APP-001-B4, RCP SEAL INJECTION HI/LO FLOW
  - APP-001-B6, LP LTDN LN HI TEMP
  - APP-001-D6, LP LTDN HI PRESS
  - APP-001-E6, LP LTDN RELIEF HI TEMP

Which ONE of the following identifies the cause of these alarms?

- A. Seal Injection Filter clogged
  - B. HCV-121, Charging Flow Valve failed closed
  - C. TCV-144, NRHX Temperature Control Valve failed closed
  - D. CVC-204A, Letdown Line Isolation Stop Valve failed closed
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 6**  
(1 point)

---

Given the following:

- A plant cooldown is in progress
- The “A” RHR Train is in service
- At 0700 RCS temperature was 130°F

Subsequently, cooling water flow is interrupted to the “A” RHR Heat Exchanger and the following is observed:

<u>Time</u>	<u>RCS Temp</u>
0715	146
0730	162
0745	178

Which ONE of the following correctly completes the statements below?

If the current heat up rate trend continues as it has for the last 45 minutes, by 0800 Mode 4 \_\_\_\_ (1) \_\_\_\_ be entered.

If the current heat up rate trend continues as it has for the last 45 minutes, by 0800 the Technical Specification Heat up limit \_\_\_\_ (2) \_\_\_\_ be exceeded.

### **(REFERENCES PROVIDED)**

- A. (1) will  
(2) will
  - B. (1) will NOT  
(2) will
  - C. (1) will  
(2) will NOT
  - D. (1) will NOT  
(2) will NOT
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 7**  
(1 point)

---

Given the following:

- The plant is at 100% power
- A Pressurizer pressure instrument has failed resulting in the following:
  - All Pressurizer heaters are energized
  - RCS pressure is 2270 psig and rising

Which ONE of the following correctly completes the statement below?

Pressurizer Pressure channel \_\_\_\_ (1) \_\_\_\_ has failed LOW.

Pressurizer PORV \_\_\_\_ (2) \_\_\_\_ will cycle open and closed to limit the RCS pressure rise.

- A. (1) PT-444  
(2) PCV-455C
  - B. (1) PT-444  
(2) PCV-456
  - C. (1) PT-445  
(2) PCV-456
  - D. (1) PT-445  
(2) PCV-455C
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 8**  
(1 point)

---

Given the following:

- An ATWS has occurred
- Immediate Actions of FRP-S.1, RESPONSE TO NUCLEAR POWER GENERATION/ATWS, are in progress

Which ONE of the following correctly completes the statements below?

Opening a MINIMUM of \_\_\_\_ (1) \_\_\_\_ breaker(s) in the Rod Drive MG Set Room will result in a reactor trip.

**OR**

Opening a MINIMUM of \_\_\_\_ (2) \_\_\_\_ breaker(s) in the 4 KV Room will result in a reactor trip.

- A. (1) one  
(2) one
  - B. (1) one  
(2) two
  - C. (1) two  
(2) one
  - D. (1) two  
(2) two
-



# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 9**  
(1 point)

---

Given the following:

- A SGTR has occurred
- The crew has entered EOP-E-3, STEAM GENERATOR TUBE RUPTURE
- The Subcooled Margin Monitor is OOS
- The crew has initiated an RCS cooldown using the Steam Dumps to a target CET temperature

Which ONE of the following correctly completes the statements below?

The operator will open the steam dumps to cooldown at \_\_\_\_ (1) \_\_\_\_.

Subsequently:

- The RCS cooldown is completed
- CET Temperature is being maintained at 510°F
- Ruptured S/G pressure is 1000 psig
- RCS pressure has stabilized at 1400 psig
- The CRS directs you to report RCS Subcooling

Based on current plant conditions RCS Subcooling is approximately \_\_\_\_ (2) \_\_\_\_ °F.

- A. (1) a maximum of 100°F/hour  
(2) 30-40
  - B. (1) the maximum achievable rate  
(2) 30-40
  - C. (1) a maximum of 100°F/hour  
(2) 70-80
  - D. (1) the maximum achievable rate  
(2) 70-80
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 10**  
(1 point)

---

Given the following:

- The plant tripped from 100% power
- The crew is implementing EOP-ECA-2.1, UNCONTROLLED DEPRESSURIZATION OF ALL STEAM GENERATORS
- RCS Boration has been initiated
- RCS temperature is 348°F and lowering
- The current time in Core Life is 20,000 MWD/MTU
- The current boron concentration is 100 ppm

Which ONE of the following correctly completes the statements below?

IAW EOP-ECA-2.1, the operator must raise the RCS boron concentration to a MINIMUM of approximately \_\_\_\_ (1) \_\_\_\_ to establish the Cold Shutdown boron concentration.

The reactivity effects due to this event are more severe while operating at the \_\_\_\_ (2) \_\_\_\_-Of-Life.

### **(REFERENCES PROVIDED)**

- A. (1) 680-690 ppm  
(2) Beginning
  - B. (1) 680-690 ppm  
(2) End
  - C. (1) 760-770 ppm  
(2) End
  - D. (1) 760-770 ppm  
(2) Beginning
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 11**  
(1 point)

---

Given the following:

- The plant was at 100% power when a loss of both MFW pumps occurred
- All Service Water pumps have lost power and cannot be restored
- CST level is 12% and rapidly lowering
- The SDAFW pump is out of service

IAW OP-402, AUXILIARY FEEDWATER SYSTEM, which ONE of the following is required to supply the OIL COOLER for operation of the "B" MDAFW pump under these conditions?

- A. Fire Water
  - B. Potable Water
  - C. Deepwell Water
  - D. Fukushima AFW Suction Tank water
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 12**  
(1 point)

---

Given the following:

- The crew is implementing EOP-ECA-0.0, LOSS OF ALL AC POWER, mitigating an Extended Loss of AC Power (ELAP)
- A depressurization of all S/G's was initiated at 10:00
- The following timeline of events is observed:

<u>Time</u>	<u>S/G Pressures</u>	<u>Pzr Level</u>
10:15	500 psig	13%
10:45	190 psig	0%

Which ONE of the following correctly completes the statements below?

The EARLIEST time at which the depressurization is required to be stopped IAW EOP-ECA-0.0, is (1).

The reason the depressurization is stopped is to (2).

- A. (1) 1015  
(2) prevent injection of accumulator nitrogen into the RCS
  - B. (1) 1045  
(2) ensure that no reactor head voiding will occur
  - C. (1) 1015  
(2) ensure that no reactor head voiding will occur
  - D. (1) 1045  
(2) prevent injection of accumulator nitrogen into the RCS
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 13**  
(1 point)

---

Given the following:

- The plant is at 100% power when a loss of off-site power occurs
- Fifteen seconds later an SI actuation occurred due to a Large Break LOCA

Which ONE of the following correctly completes the statements below?

The BOP will expect to see the SW Booster Pumps started on the \_\_\_\_ (1) \_\_\_\_.

SW Booster Pump suction pressure \_\_\_\_ (2) \_\_\_\_ have to be at least 30 psig for the SW Booster Pump to start.

- A. (1) Blackout Sequencer  
(2) does
  - B. (1) Blackout Sequencer  
(2) does NOT
  - C. (1) SI Sequencer  
(2) does
  - D. (1) SI Sequencer  
(2) does NOT
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 14**  
(1 point)

---

Given the following:

- The plant is operating at 100% power
- A Startup Transformer lockout occurs

Which ONE of the following describes how Instrument Buses 1 and 4 will respond to this event?

- A. Both Instrument Buses 1 and 4 will de-energize
  - B. Instrument Bus 1 will remain energized;  
Instrument Bus 4 will momentarily de-energize until the Emergency Diesel Generator re-powers the associated Emergency Bus
  - C. Instrument Bus 4 will remain energized;  
Instrument Bus 1 will momentarily de-energize until the Emergency Diesel Generator re-powers the associated Emergency Bus
  - D. Both Instrument Buses 1 and 4 will remain energized
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 15**  
(1 point)

---

Given the following:

- The plant is at 100% power
- APP-036-D1, BATT CHARGER A/A-1 TROUBLE, annunciated.
- The OAO reports the following indications from Battery Charger "A-1":
  - +40 Volts on the Ground Detection Voltmeter.
  - 135 Volts DC on Charger Voltage.

Which ONE of the following correctly completes the statements below?

IAW OMM-035, GROUND ISOLATION, the FIRST action the operator will take to attempt to identify the cause of the ground is \_\_\_\_ (1) \_\_\_\_.

Subsequently, MCC-A spuriously de-energizes.

Instrument Bus 2 and 7 will \_\_\_\_ (2) \_\_\_\_.

- A. (1) swap Battery Chargers to place Battery Charger "A" in-service  
(2) de-energize
  - B. (1) swap Battery Chargers to place Battery Charger "A" in-service  
(2) remain energized
  - C. (1) cycle selected breakers on Distribution Panel A  
(2) de-energize
  - D. (1) cycle selected breakers on Distribution Panel A  
(2) remain energized
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 16**  
(1 point)

---

Given the following:

- The plant was at 100% power when a reactor trip and SI occurred
- The crew has entered EOP-E-1, LOSS OF REACTOR OR SECONDARY COOLANT
- APP-036-D7, AREA MONITOR HI RAD, has just alarmed
- APP-036-H1, WDBRP TROUBLE, has just alarmed

Which ONE of the following correctly completes the statements below?

IAW EOP-E-1, radiation monitor \_\_\_\_\_(1)\_\_\_\_\_ would be used to diagnose a LOCA outside containment.

The WDBRP TROUBLE alarm \_\_\_\_\_(2)\_\_\_\_\_ consistent with a LOCA outside Containment in the Auxiliary Building.

- A. (1) R-3, PASS Panel Area  
(2) is
  - B. (1) R-3, PASS Panel Area  
(2) is NOT
  - C. (1) R-8, Drumming Room  
(2) is
  - D. (1) R-8, Drumming Room  
(2) is NOT
-



# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 17**  
(1 point)

---

Given the following:

- The reactor was tripped from 100% power due to a loss of MFW
- A spurious SI has actuated
- RCS Pressure - 2300 psig
- All S/G pressures - 1050 psig
- S/G WIDE RANGE levels are as follows:
  - WIDE RANGE: A - 10%,
  - WIDE RANGE: B - 15%,
  - WIDE RANGE: C - 9%
- AFW flow is indicating 0 gpm
- EOP-E-0, REACTOR TRIP OR SAFETY INJECTION, Immediate Actions are complete and have just been verified

Which ONE (1) of the following correctly completes the statement below?

The crew should \_\_\_\_ (1) \_\_\_\_ because \_\_\_\_ (2) \_\_\_\_.

- A. (1) remain in EOP-E-0 until directed to transition out  
(2) the "B" S/G is still a viable Heat Sink
  - B. (1) remain in EOP-E-0 until directed to transition out  
(2) rules of usage require continuing in EOP-E-0
  - C. (1) immediately go to FRP-H.1, RESPONSE TO A LOSS OF SECONDARY HEAT SINK  
(2) RCS Bleed and Feed is required
  - D. (1) immediately go to FRP-H.1, RESPONSE TO A LOSS OF SECONDARY HEAT SINK  
(2) RCS Bleed and Feed can still be avoided
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 18**  
(1 point)

---

Given the following:

- The plant has tripped from 100% power due to a LOCA
- The crew is operating in EOP-ECA-1.1, LOSS OF EMERGENCY COOLANT RECIRCULATION
- RWST level is at 16%
- RWST Makeup has been established in accordance with Supplement P, Emergency Makeup To The RWST
- The only ECCS Pump drawing on the RWST is the "A" SI Pump and pump flow has been adjusted to 150 gpm.

Which ONE of the following correctly completes the statements below?

IAW EOP-ECA-1.1, the EARLIEST that the "A" SI Pump must be stopped is when the RWST Level lowers to \_\_\_\_ (1) \_\_\_\_.

Assuming these conditions remain the same, four hours from now the RWST level will be \_\_\_\_ (2) \_\_\_\_.

- A. (1) 9%  
(2) higher
  - B. (1) 9%  
(2) lower
  - C. (1) 13%  
(2) higher
  - D. (1) 13%  
(2) lower
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 19**  
(1 point)

---

Given the following:

- The plant is at 50% power
- During the performance of OST-011, ROD CLUSTER CONTROL EXERCISE & ROD POSITION INDICATION, ONE Control Bank "B" rod is 17 steps below the rest of the bank
- The crew is performing the steps to realign the rod IAW AOP-001, MALFUNCTION OF REACTOR CONTROL SYSTEM

Which ONE of the following correctly completes the statements below?

- 1) IAW AOP-001, OPEN the Control Bank "B" lift coil switch(es) for \_\_\_\_ (1) \_\_\_\_.
- 2) Realign the rod using the \_\_\_\_ (2) \_\_\_\_ position of the rod bank selector switch.

- A. (1) the misaligned rod ONLY  
(2) CB B
  - B. (1) the misaligned rod ONLY  
(2) M (MANUAL)
  - C. (1) all but the misaligned rod  
(2) CB B
  - D. (1) all but the misaligned rod  
(2) M (MANUAL)
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 20**  
(1 point)

---

Given the following:

- The plant is at 100% power
- The Pressurizer Level Channel Selector Switch is selected to NORMAL
- The reference leg for Pressurizer level Transmitter LT-459 develops a slow leak
- The “A” Charging Pump is running in AUTO

Which ONE of the following describes the instrument response?

	<u>LT-459</u>	<u>LT-460</u>
A.	rises	lowers
B.	lowers	remains the same
C.	rises	remains the same
D.	lowers	rises

---

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 21**  
(1 point)

---

Given the following:

- A reactor startup is being commenced
- Source Range count rate ( $CR_0$ ) prior to control bank rod withdrawal is as follows:
  - N31: 3.0E+001 cps
  - N32: 3.0E+001 cps

Subsequently:

- The crew has completed the MINIMUM number of doublings required to allow the crew to withdraw control rods as necessary to achieve criticality IAW GP-003, NORMAL PLANT STARTUP FROM HOT SHUTDOWN TO CRITICAL
- The following indications are observed:
  - N31: 2.4E+002 cps
  - N32: 1.2E+002 cps

Which ONE of the following correctly completes the statement below?

IAW GP-003, Source Range Channel     (1)     is indicating AS EXPECTED.

When the Source Range Channel with the unexpected reading is removed from service per the appropriate OWP, the reactor startup     (2)     continue.

- A.     (1) N31  
       (2) may
  - B.     (1) N31  
       (2) may not
  - C.     (1) N32  
       (2) may
  - D.     (1) N32  
       (2) may not
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 22**  
(1 point)

---

Given the following:

- A Reactor startup is in progress
- The following indications are observed:
  - N31/32 = 900 cps
  - N35/36 =  $5 \times 10^{-11}$  amps

Subsequently:

- Intermediate Range (IR) channel N36 fails LOW
- The crew places the Reactor startup on HOLD

Which ONE of the following correctly completes the following statement?

The reason that the reactor startup is placed on HOLD is because a Reactor Protection Trip Function for the \_\_\_\_\_ accident is degraded.

- A. rod ejection
  - B. boron dilution
  - C. steam line break
  - D. uncontrolled RCCA bank rod withdrawal
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 23**  
(1 point)

---

Given the following:

- An accidental gaseous radwaste release is in progress
- R-1, Control Room Area Radiation Monitor, alarms
- R-1 is indicating 8.62E-2 mR/hr and stable

Which ONE of the following correctly completes the statements below?

R-1 \_\_\_\_ (1) \_\_\_\_ operating properly.

When R-1 alarms, the CR HVAC will automatically shift to the Emergency \_\_\_\_ (2) \_\_\_\_ Mode.

- A. (1) is  
(2) Recirculation
  - B. (1) is  
(2) Pressurization
  - C. (1) is NOT  
(2) Recirculation
  - D. (1) is NOT  
(2) Pressurization
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 24**  
(1 point)

---

Given the following:

- The plant is operating at 100% power
- The following is observed on the Fire Alarm Console: **(SEE REFERENCE CONDITIONS ON NEXT PAGE)**

Which ONE of the following correctly completes the statements below?

It is expected that the automatic water suppression system \_\_\_\_ (1) \_\_\_\_ actuate for these conditions.

Based on this information, the entry conditions \_\_\_\_ (2) \_\_\_\_ met for AOP-041, RESPONSE TO FIRE EVENTS.

- A. (1) will  
(2) are
  - B. (1) will  
(2) are NOT
  - C. (1) will NOT  
(2) are
  - D. (1) will NOT  
(2) are NOT
-



Question 24 Given Conditions

**FIRE COMPUTER SYSTEM**

File   Help

Alarms 
Trouble 
Unack 
Disabled

**Tue Feb 16**  
**07:xx:00**

Most Recent Unack

☒ UPS

☒ Computer

☒ Peripherals

☒ Advisories

☒ Sys Errors

TIME	TAG	NAME	TYPE	ZONE	TRAIN	DESCRIPTION	STATE
02/16 07:xx	0344	B09	ALARM	ZN-NO	TRN-B	FDAP B1 MASTER FIRE ALARM	ALARM
02/16 07:xx	0344	B09	ALARM	ZN-NO	TRN-B	FDAP B1 MASTER FIRE ALARM	ALARM
02/16 07:xx	0345	A91	ALARM	ZN-12	TRN-B	Aux. Bldg. Hall at Air Comp.	ALARM

☐ Worldview
☐ Report
Level

ROBINSON  
FIRE
SYSTEM

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 25**  
(1 point)

---

Given the following:

- With the plant at 100%, a confirmed bomb threat has been received
- The caller specified that the bomb was located in the Control Room
- The following timeline of events is recorded:

2110 The Control Room was evacuated after all appropriate actions were taken

2145 A Control operator in the Charging Pump Room informs the SM that:

- N51 reads 3.0E5 cps and is slowly lowering
- Pressurizer pressure is 2225 psig
- Pressurizer level is 35%
- The "A" Charging Pump is operating

Which ONE of the following correctly completes the statements below?

IAW Attachment 1 of AOP-004, CONTROL ROOM INACCESSIBILITY, the operator will raise Pressurizer level to at LEAST greater than (1), and then stop the running Charging Pump.

The basis for this action is to (2).

- A. (1) 60%  
(2) prevent exceeding the Charging Pump starting duty limitations
  - B. (1) 60%  
(2) ensure that the reactor is subcritical
  - C. (1) 86%  
(2) ensure that the reactor is subcritical
  - D. (1) 86%  
(2) prevent exceeding the Charging Pump starting duty limitations
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 26**  
(1 point)

---

Given the following:

- The plant is at 100% power
- The R-9, Letdown Line Area, indication on Radiation Monitoring System Recorder RR-1 is rising

Which ONE of the following correctly completes the statements below?

If the trend continues, it is expected that \_\_\_\_ (1) \_\_\_\_.

The basis for the R-9 setpoint is to detect \_\_\_\_ (2) \_\_\_\_.

- A. (1) the RR-1 will alarm BEFORE the R-9 ratemeter  
(2) small leaks in fuel rods
  - B. (1) the RR-1 will alarm BEFORE the R-9 ratemeter  
(2) release of resin from the in-service demineralizers
  - C. (1) the R-9 ratemeter will alarm BEFORE RR-1  
(2) small leaks in fuel rods
  - D. (1) the R-9 ratemeter will alarm BEFORE RR-1  
(2) release of resin from the in-service demineralizers
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 27**  
(1 point)

---

Given the following:

- EOP-ECA-0.1. LOSS OF ALL AC POWER RECOVERY WITHOUT SI REQUIRED has been implemented
- The crew is checking for Natural Circulation with the following conditions:
  - Containment pressure is 0.5 psig
  - Steam dump using the S/G PORVs is in progress
  - RCS subcooling based on CETs are 33°F and stable
  - CETC are 555°F and stable
  - S/G pressures are stable at 900 psig
  - RCS Hot Leg temperatures are 555°F and stable
  - RCS Cold Leg temperatures are 545°F and slowly rising
  - S/G NR levels are 21% and stable

Which ONE of the following correctly completes the statements below?

Based on the indications above, Natural Circulation (NC) flow \_\_\_\_ (1) \_\_\_\_ been established.

IAW EOP-ECA-0.1, the crew will \_\_\_\_ (2) \_\_\_\_.

- A. (1) has NOT  
(2) raise S/G levels to promote the establishment of NC flow
  - B. (1) has  
(2) maintain steam dump rate and a minimum S/G NR level of 9%
  - C. (1) has  
(2) maintain steam dump rate and a minimum S/G NR level of 18%
  - D. (1) has NOT  
(2) raise the rate of dumping steam to promote the establishment of NC flow
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 28**  
(1 point)

---

Given the following:

- The plant is operating at 100% power when a transient occurs
- Over 15-20 seconds a load rejection lowers and then stabilizes power at 80%

Which ONE of the following correctly completes the statements below?

Seal water injection flow will INITIALLY       (1)      .

Subsequently,

- APP-001-B4, RCP SEAL INJ HI/LO FLOW, has alarmed
- The CRS has directed use of the expanded seal water injection flow control band

The expanded seal water injection flow control band is       (2)      .

- A.     (1) rise  
       (2) 6-20 gpm
  - B.     (1) rise  
       (2) 7-18 gpm
  - C.     (1) lower  
       (2) 6-20 gpm
  - D.     (1) lower  
       (2) 7-18 gpm
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 29**  
(1 point)

---

Given the following:

- The plant is in Mode 3
- APP-003-E2, VCT HI/LO PRESS, has alarmed
- Actual VCT pressure is 15 psig

Which ONE of the following correctly completes the statement below?

If VCT pressure continues to lower, RCP #1 Seal Leakoff flow will \_\_\_\_ (1) \_\_\_\_ and RCP #2 Seal leakoff flow will \_\_\_\_ (2) \_\_\_\_.

- A. (1) lower  
(2) rise
  - B. (1) rise  
(2) lower
  - C. (1) rise  
(2) rise
  - D. (1) lower  
(2) lower
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 30**  
(1 point)

---

Given the following:

- The plant is operating at 100% power
- Due to RTGB indications the crew has entered AOP-024, LOSS OF INSTRUMENT BUS
- An Auto Makeup to the VCT has initiated on the loss of instrument bus

Which ONE of the following correctly completes the statement below?

The Auto makeup has occurred because VCT Level transmitter \_\_\_\_ (1) \_\_\_\_ has de-energized on a loss of Instrument Bus \_\_\_\_ (2) \_\_\_\_.

- A. (1) LT-112  
(2) 7
  - B. (1) LT-112  
(2) 9
  - C. (1) LT-115  
(2) 7
  - D. (1) LT-115  
(2) 9
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 31**  
(1 point)

---

Given the following:

- A plant cooldown is in progress
- The "A" RHR Train is in service
- RCS temperature is 190°F
- RCS Pressure is 350 psig
- The RCS is in Solid Plant Operation with LTOPP in service

Subsequently, a transient occurs resulting in the following:

- RCS temperature is 195°F
- RCS Pressure is 420 psig
- PZR PORV PCV-456 indicates OPEN
- PZR PORV PCV-455C indicates CLOSED

The crew has entered AOP-019, MALFUNCTION OF RCS PRESSURE CONTROL.

Which ONE of the following correctly completes the statements below?

Based on plant conditions, both PORVs should have \_\_\_\_ (1) \_\_\_\_.

IAW AOP-019 immediate actions, the operator will **FIRST** \_\_\_\_ (2) \_\_\_\_ to control or mitigate the consequences of this event.

- A. (1) remained closed  
(2) close PCV-456
  - B. (1) opened  
(2) stop all Charging Pumps and RCPs
  - C. (1) opened  
(2) ensure the block valve for PCV-456 is OPEN
  - D. (1) remained closed  
(2) place the PCV-456 LTOPP Arming Switch to NORMAL
-



# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 32**  
(1 point)

---

Given the following:

- “A” Train RHR is in service
- Instrument Air pressure lowers to 30 psig

Which ONE of the following correctly completes the statement below?

RCS temperature will INITIALLY \_\_\_\_ (1) \_\_\_\_, and RHR System Flow will INITIALLY \_\_\_\_ (2) \_\_\_\_.

- A. (1) Rise  
(2) Rise
  - B. (1) Rise  
(2) Lower
  - C. (1) Lower  
(2) Lower
  - D. (1) Lower  
(2) Rise
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 33**  
(1 point)

---

Given the following:

- The plant is at 100% RTP
- The following timeline is observed:

<u>TIME</u>	<u>"A" SI accumulator pressure</u>	<u>"A" SI accumulator level</u>
1108	637 psig	68 percent
1111	646 psig	70 percent
1114	660 psig	75 percent

Which ONE of the following correctly completes the statement below?

The EARLIEST time at which an RTGB annunciator alarm would occur is at time \_\_\_\_\_ (1) \_\_\_\_\_, due to \_\_\_\_\_ (2) \_\_\_\_\_ reaching its alarm setpoint.

- A. (1) 1111  
(2) APP-002-A4, SI ACCUM A HI/LO LVL
  - B. (1) 1114  
(2) APP-002-B4, SI ACCUM A HI/LO PRESS
  - C. (1) 1111  
(2) APP-002-B4, SI ACCUM A HI/LO PRESS
  - D. (1) 1114  
(2) APP-002-A4, SI ACCUM A HI/LO LVL
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 34**  
(1 point)

---

Given the following:

- The plant was at 100% power when a LOCA occurs inside Containment

Which ONE of the following correctly completes the statements below?

While operating in the EOP Network, the LOWEST pressure in which “Adverse” containment values are applicable is \_\_\_\_ (1) \_\_\_\_.

Based on the increased containment temperature, level detectors with wet reference legs will indicate \_\_\_\_ (2) \_\_\_\_ than the actual level.

- A. (1) 4 psig  
(2) lower
  - B. (1) 4 psig  
(2) higher
  - C. (1) 10 psig  
(2) lower
  - D. (1) 10 psig  
(2) higher
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 35**  
(1 point)

---

Given the following:

- The plant is at 100% power
- Containment pressure is 0.2 psig
- Containment temperature is 94°F

Subsequently:

- A load rejection results in a reactor trip
- Following the trip, a Pressurizer Safety valve opens, and will NOT reseal
- The PRT rupture disks function as designed
- Containment pressure is rising at 0.1 psig every 5 minutes
- Containment temperature is rising at 2°F every 5 minutes

Assuming these conditions remain constant, which ONE of the following identifies the Containment Technical Specifications LCOs that will be affected one hour from now?

- A. Both LCO 3.6.4, Containment Pressure, and LCO 3.6.5, Containment Air Temperature, will be exceeded.
  - B. Only LCO 3.6.4, Containment Pressure, will be exceeded.
  - C. Only LCO 3.6.5, Containment Air Temperature, will be exceeded.
  - D. Neither LCO 3.6.4, Containment Pressure, nor LCO 3.6.5, Containment Air Temperature, will be exceeded.
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 36**  
(1 point)

---

Given the following:

T= 1305:10

- A loss of off-site power has occurred concurrent with a Reactor trip
- The crew enters EOP-E-0, Reactor Trip or Safety Injection

T=1307:40

- Immediate actions of EOP-E-0 are complete
- The following parameters are noted:
  - RCS pressure is 2110 psig and stable
  - SG pressures are all 910 psig and lowering slowly
  - CV pressure is 0.7 psig and rising slowly

At time 1307:40, which ONE of the following describes the CCW Pump indication available in the control room?

- A. "B" and "C" CCW Pump RED indicating lights are LIT.  
"A" CCW Pump GREEN indicating light is LIT.
  - B. All 3 CCW Pump control switch GREEN indicating lights are LIT.
  - C. "B" and "C" CCW Pump GREEN indicating lights are LIT;  
"A" CCW Pump RED indicating light is LIT.
  - D. "B" and "C" CCW Pump RED indicating lights are LIT;  
"A" CCW Pump GREEN and RED indicating lights are EXTINGUISHED.
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 37**  
(1 point)

---

The plant is at 100% power when the following occurs:

- A load rejection results in a PZR PORV actuation
- The PZR PORV will not fully seat
- The depressurization led to a Reactor Trip and Safety Injection
- PZR pressure reduces to 1400 psig and stabilizes
- The PRT rupture discs rupture at their design setpoint
- Containment pressure peaks at 15 psig

Which ONE of the following is the MAXIMUM temperature indicated by the Safety Valve tailpiece RTD during the entire event?

- A. 213°F
  - B. 250°F
  - C. 298°F
  - D. 338°F
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 38**  
(1 point)

---

Which ONE of the following identifies a condition which will cause an automatic reactor trip?

- A. RCP bus frequency reads 58.0 Hz when THREE Power Range channels read 5% RTP.
  - B. Flow in ONE Reactor Coolant Loop reads 91% when ALL Power Range channels read 35% RTP.
  - C. ONE Intermediate Range NI channel reads  $1 \times 10^{-6}$  AMPS when TWO Power Range channels read 15% RTP.
  - D. ONE Source Range NI channel reads  $5 \times 10^5$  CPS when TWO Intermediate Range NI channels read  $9 \times 10^{-11}$  AMPS.
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 39**  
(1 point)

---

Given the following:

- The plant was at 100% when a SI occurred

Which ONE of the following correctly completes the statements below regarding the Feed Reg Valves and the Feed Reg Bypass Valves?

Following the SI signal, the **Feed Reg Bypass Valves** \_\_\_\_ (1) \_\_\_\_ receive a CLOSE signal.

The **Feed Reg Valves** \_\_\_\_ (2) \_\_\_\_ be available when the SI is RESET and the Feedwater Isolation OVRD/RESET key switches are placed in OVRD/RESET.

- A. (1) will  
(2) will
  - B. (1) will NOT  
(2) will
  - C. (1) will  
(2) will NOT
  - D. (1) will NOT  
(2) will NOT
-



# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 40**

(1 point)

---

Which ONE of the following correctly completes the statements below regarding the Containment Spray Actuation logic scheme?

The MINIMUM number of Containment Pressure channels that must sense pressure greater than the Hi-Hi pressure setpoint to generate an automatic Containment Spray Actuation is \_\_\_\_ (1) \_\_\_\_.

The Containment Spray Actuation bistables are \_\_\_\_ (2) \_\_\_\_ to actuate.

- A. (1) Two  
(2) Energize
  - B. (1) Two  
(2) De-energize
  - C. (1) Four  
(2) Energize
  - D. (1) Four  
(2) De-energize
-

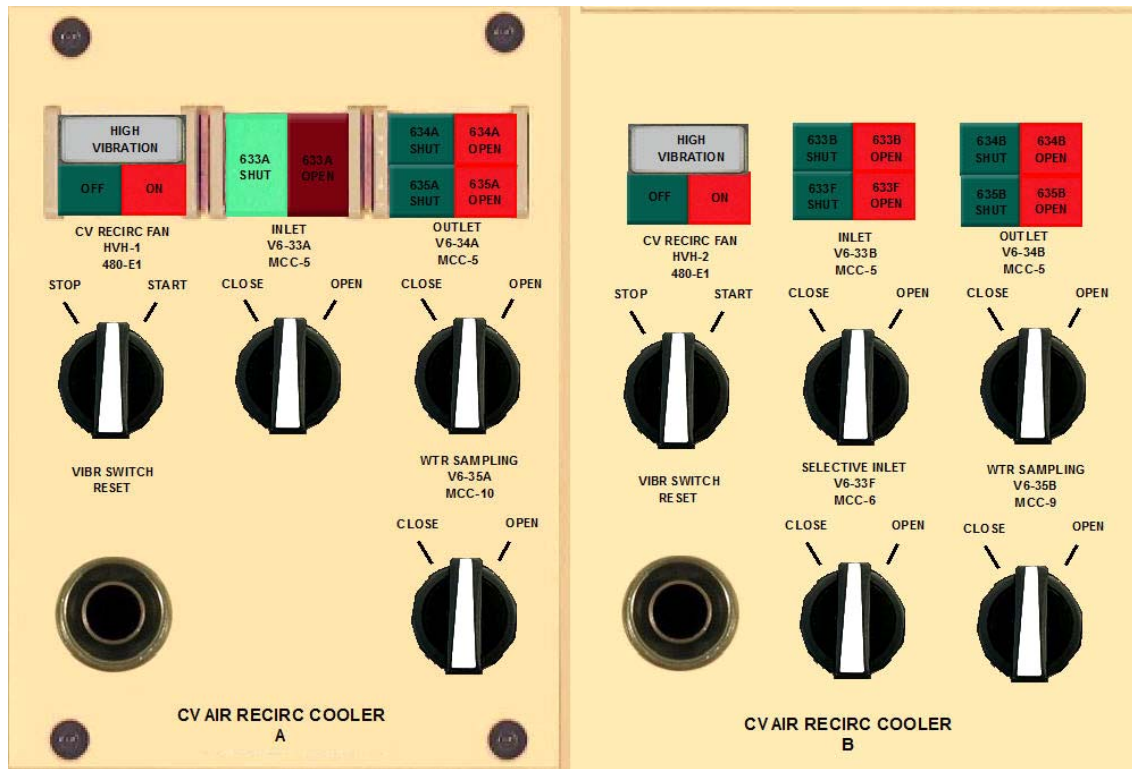
# Robinson Nuclear Plant

## ILC15 RNP SRO NRC Examination

**Question: 41**  
(1 point)

Given the following:

- The plant is operating at 100% power
- A realignment has just occurred and the operator observes the following Service Water valve alignments for HVH-1 (CV Recirc Fan A) and HVH-2 (CV Recirc Fan B):



Assuming no conditions change, which ONE of the following identifies the operational restrictions, if any, placed on HVH-1?

- A. HVH-1 may run continuously, there are no restrictions.
- B. HVH-1 may continue to run for up to 15 additional minutes.
- C. HVH-1 must be immediately secured and cannot be run under these conditions.
- D. HVH-1 must be immediately secured and can ONLY be started during emergencies.

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 42**  
(1 point)

---

Given the following:

- The plant was at 100%
- A LOCA inside containment occurs and the following timeline is observed:

<u>TIME</u>	<u>OBSERVATION</u>
1200	LOCA occurs
1201	CV Spray system actuates
1211	CV Pressure peaks at 22 PSIG
1235	CV Pressure lowers to less than 10 PSIG
1245	CV Pressure lowers to less than 4 PSIG

Which ONE of the following correctly completes the statements below?

IAW EOP-E-1, LOSS OF REACTOR OR SECONDARY COOLANT, the EARLIEST TIME that the Containment Spray Pumps may be secured is \_\_\_\_ (1) \_\_\_\_.

In order to successfully reset the Phase B Containment Isolation signal, the Containment Spray Signal \_\_\_\_ (2) \_\_\_\_ required to be reset FIRST.

- A. (1) 1235  
(2) is
  - B. (1) 1235  
(2) is NOT
  - C. (1) 1245  
(2) is
  - D. (1) 1245  
(2) is NOT
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

### Question: 43

(1 point)

---

Given the following:

- The plant is at 100% power
- R-15, Air Ejector Radiation Monitor, alarms
- The crew entered AOP-035, S/G TUBE LEAK
- The crew has initiated Attachment 5, R-15 Monitoring
- Chemistry reports the following leakrates:

<u>Time</u>	<u>"A" S/G Tube leakrate</u>
0800	31 gpd
0900	51 gpd
1000	81 gpd
1100	111 gpd

IAW AOP-035, which ONE of the following identifies the EARLIEST time that a plant shutdown is required?

- A. 0800
  - B. 0900
  - C. 1000
  - D. 1100
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 44**  
(1 point)

---

Given the following:

- The plant is operating at 100% power
- An inadvertent actuation of the Feedwater Isolation Signal (FWIS) occurs

Which ONE of the following describes how this actuation affects the Reactor and the Main Turbine?

- A. BOTH the Reactor and the Main Turbine receive trip signals directly from the FWIS.
  - B. The Reactor receives a trip signal directly from the FWIS, and causes a Main Turbine trip.
  - C. The Main Turbine receives a trip signal directly from the FWIS and the Reactor will trip because the Main Turbine tripped with power above P-8.
  - D. NEITHER the Reactor nor the Main Turbine receive trip signals directly from the FWIS. However, the Reactor will trip on plant conditions created by the FWIS actuation and cause a Main Turbine trip.
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 45**  
(1 point)

---

Given the following:

- With the plant at 100% power, an ATWS event occurred
- The crew entered FRP-S.1, RESPONSE TO NUCLEAR POWER GENERATION ATWS
- All SG Narrow Range levels are OFF-Scale LOW

Which ONE of the following correctly completes the statement below?

IAW FRP-S.1, AFW flow must be greater than a MINIMUM value of \_\_\_\_\_ to establish a Secondary Heat Sink?

- A. 300 gpm
  - B. 500 gpm
  - C. 600 gpm
  - D. 1000 gpm
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 46**

(1 point)

---

Given the following:

- The plant is at 100% power
- A reactor trip has occurred

Which ONE of the following identifies the breakers which will automatically CLOSE one minute after the reactor trip?

Breaker nomenclature:

52/10	4KV BUS 1-2 TIE BKR
52/12	START-UP TRANSFORMER TO 4KV BUS 2
52/17	START-UP TRANSFORMER TO 4KV BUS 3
52/19	4KV BUS 3-4 TIE BKR

- A. 52/10 and 52/12
- B. 52/10 and 52/17
- C. 52/12 and 52/19
- D. 52/17 and 52/19
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 47**  
(1 point)

---

Given the following:

- The plant is at 100% power
- Breaker 52/20, UNIT AUX TO 4KV BUS 4 BKR, trips on fault

Which ONE of the following correctly completes the statement below?

Circulating Water pump(s) \_\_\_\_\_ lost power.

- A. "A" & "B" have
  - B. "B" & "C" have
  - C. "A" & "C" have
  - D. "B" ONLY has
-



# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 48**

(1 point)

---

Given the following:

- The plant is in Mode 3
- The “B” Battery Charger is in service
- The “B-1” Battery Charger is in standby

Subsequently:

- MCC-6 is de-energized
- APP-036-D2, BATT CHARGER B/B-1 TROUBLE, alarms
- Two minutes later MCC-6 is re-energized

Which ONE of the following identifies when APP-036-D2 will clear?

APP-036-D2 will clear.....

- A. when the “B” Battery Charger automatically restarts.
  - B. only after the operator manually restarts the “B” Battery Charger.
  - C. only after the operator manually places the “B-1” Battery Charger back in the STANDBY mode.
  - D. only after the operator manually restarts the “B” Battery Charger and places the “B-1” Battery Charger back in the STANDBY mode.
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 49**  
(1 point)

---

Given the following:

- The plant is at 100% power
- OST-401-1, EDG "A" SLOW SPEED START testing is in progress
- Current "A" EDG load is 2500 KW

Subsequently:

- The Normal Supply Breaker to E-1, 52/18B, opens
- EDG "A" Output Breaker 52/17B, opens

Which ONE of the following correctly completes the statements below?

IAW OP-604 CAUTION statement, the consequence of opening the EDG "A" Output Breaker under load is \_\_\_\_ (1) \_\_\_\_.

IAW OP-604 when attempting to recover, the local Generator Parallel/Isolate Switch must be placed in the \_\_\_\_ (2) \_\_\_\_ position.

OP-604, DIESEL GENERATORS "A" AND "B"

- A. (1) the diesel engine may trip due to overspeed  
(2) ISOL
  - B. (1) excessive carbon buildup in the engine could occur  
(2) PARALLEL
  - C. (1) the diesel engine may trip due to overspeed  
(2) PARALLEL
  - D. (1) excessive carbon buildup in the engine could occur  
(2) ISOL
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 50**  
(1 point)

---

Which ONE of the following supplies power to the “A” and “B” EDG Fuel Oil Transfer Pumps?

- A. MCC-10 and MCC-9 respectively
  - B. MCC-9 and MCC-10 respectively
  - C. MCC-6 and MCC-5 respectively
  - D. MCC-5 and MCC-6 respectively
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 51**  
(1 point)

---

Given the following:

- The reactor is at 100%
- The crew is performing a containment vacuum relief IAW OP-921, CONTAINMENT AIR HANDLING
- R-12, CV AIR OR PLANT STACK, NOBLE GAS, alarms

V12-12, CV VAC RELIEF

V12-13, CV VAC RELIEF

APP-036-D7, AREA MONITOR HI RAD

APP-036-D8, PROCESS MONITOR HI RAD

Which ONE of the following correctly completes the statements below?

The crew would expect to see     (1)     annunciator flashing.

Due to the conditions above,     (2)     will automatically close.

- A.     (1) APP-036-D7  
       (2) V12-12 & V12-13 ONLY
- B.     (1) APP-036-D8  
       (2) V12-12 & V12-13 ONLY
- C.     (1) APP-036-D7  
       (2) V12-12, V12-13 & the CV Intake Damper
- D.     (1) APP-036-D8  
       (2) V12-12, V12-13 & the CV Intake Damper
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 52**  
(1 point)

---

Given the following:

- The plant is operating at 100% power
- R-19A, STM GEN BLOW DN SG-A, enters the alarm condition

Which ONE of the following correctly completes the statements below?

In regards to the Blowdown Isolation Valves and the Blowdown Sample Isolation Valves, an alarm condition on R-19A will automatically close the \_\_\_\_ (1) \_\_\_\_.

IAW AOP-005, RADIATION MONITORING SYSTEM, if the automatic actions fail to occur, the crew \_\_\_\_ (2) \_\_\_\_ de-energize R-19A in an attempt to cause the automatic actions to occur.

- A. (1) Blowdown Isolation Valves (FCV-1930A/B), ONLY  
(2) will
  - B. (1) Blowdown Isolation Valves (FCV-1930A/B), ONLY  
(2) will NOT
  - C. (1) Blowdown Isolation Valves (FCV-1930A/B) and Blowdown Sample Isolation Valves (FCV-1933A/B)  
(2) will
  - D. (1) Blowdown Isolation Valves (FCV-1930A/B) and Blowdown Sample Isolation Valves (FCV-1933A/B)  
(2) will NOT
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

### Question: 53

(1 point)

---

Which ONE of the following correctly completes the statements below regarding components normally cooled by Service Water?

The Circulating Water Pumps \_\_\_\_ (1) \_\_\_\_ be cooled by the Fire Water System during an emergency.

The Control Room HVAC Cooling Units \_\_\_\_ (2) \_\_\_\_ be cooled by the Fire Water System during an emergency.

- A. (1) can  
(2) can
  - B. (1) can  
(2) can NOT
  - C. (1) can NOT  
(2) can NOT
  - D. (1) can NOT  
(2) can
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 54**  
(1 point)

---

Given the following:

- The plant is at 100% power
- “D” IA Compressor is running
- Station Air Compressor is running

Subsequently:

- IA Header pressure is 79 PSIG and lowering
- The crew is implementing AOP-017, LOSS OF INSTRUMENT AIR

SA-5, STATION AIR TO INST AIR CROSS CONNECT

SA-220, SA TO IA CROSS CONNECT BYPASS FILTER ISOLATION

SA-221, SA TO IA CROSS CONNECT BYPASS FILTER ISOLATION

Which ONE of the following correctly completes the statements below?

IAW AOP-017, the PREFERRED method to cross connect IA from Station Air is by opening  
\_\_\_\_(1)\_\_\_\_.

IA will no longer be used to supply Breathing Air to \_\_\_\_ (2) \_\_\_\_.

- A. (1) SA-5  
(2) minimize IA loads ONLY
  - B. (1) SA-5  
(2) minimize IA loads and prevent harm to users of Breathing Air
  - C. (1) SA-220 and SA-221  
(2) minimize IA loads ONLY
  - D. (1) SA-220 and SA-221  
(2) minimize IA loads and prevent harm to users of Breathing Air
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 55**  
(1 point)

---

Given the following:

- A LOCA occurred 45 minutes ago
- The crew is implementing the Emergency Operating Procedures
- The following conditions exist:
  - CET temperature is 340°F
  - RCS pressure is 60 psig and lowering
  - Full Range RVLIS is 40%
  - RCS Cold Leg temperature is 320°F and lowering
  - S/G NR levels are off-scale LOW
  - S/G pressures are 550 psig and trending down
  - AFW flow is 450 gpm
  - All RCPs are OFF
  - CV pressure is 43 psig and rising
- SPDS has been reset

Which ONE of the following identifies the MOST challenged Critical Safety Function?

- A. Integrity
  - B. Heat Sink
  - C. Containment
  - D. Core Cooling
-



# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 56**  
(1 point)

---

Given the following:

- The plant is operating at 100% power

Subsequently, all IRPI indication is lost except a green LED is illuminated on each rod indicator

Which ONE of the following correctly completes the statements below?

The cause of this condition is the \_\_\_\_ (1) \_\_\_\_ has tripped.

For this condition entry into AOP-001, MALFUNCTION OF REACTOR CONTROL SYSTEM, \_\_\_\_ (2) \_\_\_\_ required.

- A. (1) Instrument Bus 7A, Circuit 10 breaker, (RTGB receptacles),  
(2) is
  - B. (1) PP-61, Circuit 1 breaker, ROD POSITION INDICATION SYSTEM,  
(2) is NOT
  - C. (1) PP-61, Circuit 1 breaker, ROD POSITION INDICATION SYSTEM,  
(2) is
  - D. (1) Instrument Bus 7A, Circuit 10 breaker, (RTGB receptacles),  
(2) is NOT
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 57**  
(1 point)

---

Given the following:

- GP-008, DRAINING THE REACTOR COOLANT SYSTEM, is in progress
- The crew has positioned HIC-142, PURIFICATION FLOW, to establish required drain rate

Which ONE of the following correctly completes the statements below?

LCV-115A, VCT/HLDP TK DIV, will begin to divert at \_\_\_\_ (1) \_\_\_\_ in the VCT.

IAW GP-008, \_\_\_\_ (2) \_\_\_\_ level change can be used to determine the volume drained from the RCS.

- A. (1) 30"  
(2) WHUT
  - B. (1) 30"  
(2) CVCS HUT
  - C. (1) 51.6"  
(2) CVCS HUT
  - D. (1) 51.6"  
(2) WHUT
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 58**  
(1 point)

---

Given the following:

- The plant is at 35% power and stable
- The “A” Charging Pump is running in AUTO
- The “B” Charging Pump is running in MANUAL and adjusted to 50% output on Charging Pump Speed Controller SC-151
- Letdown flow is 105 gpm

Subsequently:

- Letdown flow is lowered 45 gpm
- Reactor power is lowered to 30% and stabilized
- Pressurizer Level lowers to, and has stabilized at 31.5%

Which ONE of the following correctly completes the statements below?

The Pressurizer Level Control System \_\_\_\_ (1) \_\_\_\_ operating as expected for these plant conditions.

While Letdown was being lowered, APP-003-F3, CHG PMP LO SPEED, alarmed.

According to OP-301, CHEMICAL AND VOLUME CONTROL SYSTEM, the operator will be required to \_\_\_\_ (2) \_\_\_\_ Charging Pump to clear the alarm.

- A. (1) is  
(2) place the “A” Charging Pump in Manual and adjust the speed of the “A”
  - B. (1) is  
(2) adjust the speed of the “B”
  - C. (1) is NOT  
(2) place the “A” Charging Pump in Manual and adjust the speed of the “A”
  - D. (1) is NOT  
(2) adjust the speed of the “B”
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 59**  
(1 point)

---

Given the following:

- The Plant is in Mode 1, following a control rod manipulation
- The RTGB IRPI indication for Control Bank "D" Rod M-8 indicates 120 inches
- The RTGB IRPI indication for Control Bank "D" Rod H-4 indicates 110 inches
- Control Bank "D" step counter position is 192 steps

Which ONE of the following correctly completes the statements below?

Based on the given conditions, control rod \_\_\_\_ (1) \_\_\_\_ is stuck.

Assuming the average of the individual rod positions in the affected bank indicate 120 inches, entry into ITS LCO 3.1.4, ROD GROUP ALIGNMENT LIMITS, \_\_\_\_ (2) \_\_\_\_ required.

- A. (1) M-8  
(2) is
  - B. (1) M-8  
(2) is NOT
  - C. (1) H-4  
(2) is
  - D. (1) H-4  
(2) is NOT
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 60**  
(1 point)

---

Given the following:

- The operator is reviewing the T/C TREND pages on both the Train "A" and Train "B" ICCM displays

Which ONE of the following correctly completes the statements below?

The TOTAL number of Core Exit Thermocouples (CET) that monitor the core is \_\_\_\_\_ (1) \_\_\_\_\_.

The T/C TREND page displays the average of \_\_\_\_\_ (2) \_\_\_\_\_ CET indications over the last 30 minutes.

- A. (1) 23  
(2) the five highest
  - B. (1) 23  
(2) all
  - C. (1) 46  
(2) all
  - D. (1) 46  
(2) the five highest
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 61**  
(1 point)

---

Which ONE (1) of the following is the power supply for HVE-3, CV Air Iodine Removal Fan?

- A. MCC-5
  - B. MCC-6
  - C. MCC-9
  - D. MCC-10
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 62**  
(1 point)

---

Which ONE of the following correctly completes the statements below?

IAW GP-010, REFUELING, the Containment Purge System \_\_\_\_ (1) \_\_\_\_ required be in operation during Core Alterations with the Equipment Hatch removed.

The Containment Purge System \_\_\_\_ (2) \_\_\_\_ required be in operation if a Containment entry is being made with the plant in Mode 1.

- A. (1) is  
(2) is
  - B. (1) is  
(2) is NOT
  - C. (1) is NOT  
(2) is
  - D. (1) is NOT  
(2) is NOT
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 63**  
(1 point)

---

Given the following:

- The plant is operating at 100% power
- Spent fuel handling activities are NOT in progress

Subsequently:

- SFP level starts to rapidly lower due to a large structural and piping failure
- The crew has entered AOP-036, SFP EVENTS

R-14, Plant Vent Effluent Monitor

R-21, Fuel Handling Building Upper Level, Noble Gas

HVE-15, Fuel Handling Building Exhaust Fan

Which ONE of the following correctly completes the statements below?

IAW AOP-036, R-14 \_\_\_\_ (1) \_\_\_\_ monitored for indications of fuel damage.

If R-21 were to subsequently alarm, HVE-15 \_\_\_\_ (2) \_\_\_\_ automatically trip.

- A. (1) is  
(2) will
  - B. (1) is  
(2) will NOT
  - C. (1) is NOT  
(2) will
  - D. (1) is NOT  
(2) will NOT
-



# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 64**  
(1 point)

---

Given the following:

- The plant is operating at 100% power
- The SDAFW Pump is OOS

Subsequently:

- The crew trips the reactor and initiates safety injection due to a SGTR on the "B" Steam Generator
- V2-16A, AFW Header Discharge Valve, has failed CLOSED
- The "B" MDAFW Pump trips on auto-start
- "B" SG NR level is 6% and rising slowly
- "A" and "C" SG NR levels are off-scale low

Which ONE of the following correctly completes the statements below?

Without operator action, a Secondary Heat Sink \_\_\_\_\_ (1) \_\_\_\_\_ established.

Based on the current conditions, the operator \_\_\_\_\_ (2) \_\_\_\_\_ immediately isolate feed to the "B" S/G.

- A. (1) is  
(2) will
  - B. (1) is  
(2) will NOT
  - C. (1) is NOT  
(2) will
  - D. (1) is NOT  
(2) will NOT
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 65**  
(1 point)

---

Which ONE of the following discharges to the Reactor Coolant Drain Tank?

- A. RCV-609, CC SURGE TANK VENT
  - B. CVC-203 A/B, LETDOWN RELIEF VALVES
  - C. CVC-389, EXCESS LETDOWN DIVERSION
  - D. CVC-400 A/B/C, CHARGING PUMP "A/B/C" LEAKAGE ISOLATION
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 66**  
(1 point)

---

Which ONE of the following correctly completes the statements below?

IAW OPS-NGGC-1303, VERIFICATION PRACTICES, guidance for waiving Independent Verification (IV) requirements states that the IV may be waived if the operators were expected to \_\_\_\_ (1) \_\_\_\_ to complete the verification.

In these cases, \_\_\_\_ (2) \_\_\_\_ Verification will be used as an alternative to Independent Verification.

- A. (1) be in an area where the dose rate is greater than 100 mrem/hour  
(2) Concurrent
  - B. (1) receive a total exposure of greater than 10 mrem  
(2) Concurrent
  - C. (1) receive a total exposure of greater than 10 mrem  
(2) Functional
  - D. (1) be in an area where the dose rate is greater than 100 mrem/hour  
(2) Functional
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 67**

(1 point)

---

Given the following plant conditions:

- The plant is in MODE 6 for refueling operations
- Core alterations are in progress

Which ONE of the following correctly completes the statements below?

IAW LCO 3.9.2, NUCLEAR INSTRUMENTATION, \_\_\_\_\_ (1) \_\_\_\_\_ shall be OPERABLE.

N-51 and N-52, \_\_\_\_\_ (2) \_\_\_\_\_ capable of providing audible indication inside Containment.

N-31, N-32, SOURCE RANGE NUCLEAR INSTRUMENTS

N-51, N-52, REG. GUIDE 1.97 NEUTRON FLUX (SOURCE RANGE, WIDE RANGE)

- A. (1) either N-31 OR N-32  
(2) are
  - B. (1) both N-31 AND N-32  
(2) are
  - C. (1) either N-31 OR N-32  
(2) are NOT
  - D. (1) both N-31 AND N-32  
(2) are NOT
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 68**  
(1 point)

---

Given the following:

- A plant startup is in progress
- NIS Power Indication is 50%
- The STA is recording data IAW GP-005, POWER OPERATION, Attachment 1, REACTOR POWER ASCENSION INDICATOR LOG

Which ONE of the following correctly completes the statements below?

IAW GP-005, if Core  $\Delta T$  indicates 33°F, the crew \_\_\_\_ (1) \_\_\_\_ be required to stabilize reactor power and perform OST-010, Power Range Calorimetric During Power Operation (ERFIS).

IAW GP-005, if Calorimetric power indicates 51.5%, the crew \_\_\_\_ (2) \_\_\_\_ be required to stabilize reactor power and perform OST-010, Power Range Calorimetric During Power Operation (ERFIS).

- A. (1) will  
(2) will
  - B. (1) will  
(2) will NOT
  - C. (1) will NOT  
(2) will
  - D. (1) will NOT  
(2) will NOT
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 69**  
(1 point)

---

Given the following:

- The plant is at 100% power
- The FWUFM System has been OOS for five days

Subsequently:

- An AO reports a steam leak near the Main Steam Isolation Valves
- The following indications are noted in the Control Room:
  - Tavg is LOWERING
  - Steam flow and feed flow have RISEN
  - A Power Limit Warning alarm on ERFIS has been received
  - Reactor power is slowly RISING

Which ONE of the following identifies the MAXIMUM thermal power limit allowed by the operating license under these conditions?

- A. 2300 MWt
  - B. 2307 MWt
  - C. 2339 MWt
  - D. 2346 MWt
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 70**  
(1 point)

---

Given the following:

- The plant is at 100% power
- It has been determined that LCO 3.2.3, Axial Flux Difference is outside the target band.

Which ONE of the following correctly completes the statement below?

IAW Technical Specification LCO 3.2.3 ACTION.....

Restore AFD to within target band within   (1)  .

If AFD cannot be restored within target band within the allowable time, Reduce THERMAL POWER to < 90% RTP or 0.9 APL, whichever is less, within an additional   (2)  .

- A.     (1) 15 minutes  
       (2) 15 minutes
- B.     (1) 15 minutes  
       (2) 1 hour
- C.     (1) 30 minutes  
       (2) 15 minutes
- D.     (1) 30 Minutes  
       (2) 1 hour
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 71**  
(1 point)

---

Given the following:

- The plant is operating at 100% power
- Letdown flow is 45 gpm
- The “A” Charging Pump is operating in AUTO
- The “B” Charging Pump is operating in MANUAL

Subsequently:

- R-9, Letdown Line Area Monitor, alarms
- The alarm is determined to be valid
- The crew enters AOP-005, RADIATION MONITORING SYSTEM

Which ONE of the following correctly completes the statements below?

IAW with Attachment 9 of AOP-005, the operator       (1)       required to evacuate the Aux Building of non-essential personnel.

IAW with Attachment 9 of AOP-005, the operator       (2)       required to isolate letdown flow.

- A.     (1) is  
       (2) is
  - B.     (1) is  
       (2) is NOT
  - C.     (1) is NOT  
       (2) is
  - D.     (1) is NOT  
       (2) is NOT
-



# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 72**  
(1 point)

---

Given the following:

- An operator has been assigned work in the radiologically controlled area
- The dose rate in the area is 500 mR/hr
- The operator has a current yearly dose of 0.5 Rem TEDE

Which ONE of the following identifies the MAXIMUM time that the operator can perform work BEFORE reaching the Duke Energy Annual Administrative Dose Limit without receiving an extension?

- A. 1 hour
  - B. 3 hours
  - C. 4 hours
  - D. 7 hours
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 73**  
(1 point)

---

Given the following:

- A plant cooldown is in progress
- RCS Temperature is 430°F
- RCS pressure is 725 psig
- The SI Accumulators have been isolated

Subsequently, an RCS leak occurs and the operating crew enters AOP-016, EXCESSIVE RCS LEAKAGE.

Which ONE of the following correctly completes the statements below?

AOP-016 will ensure that a MINIMUM of \_\_\_\_ (1) \_\_\_\_ Charging Pumps are running with their speed adjusted to maximum.

With the current plant conditions, if Pressurizer level continues to lower uncontrollably with maximum charging flow, the crew would transition to \_\_\_\_ (2) \_\_\_\_.

- A. (1) two  
(2) AOP-033, SHUTDOWN LOCA
  - B. (1) two  
(2) EOP-E-0, REACTOR TRIP OR SAFETY INJECTION
  - C. (1) three  
(2) AOP-033, SHUTDOWN LOCA
  - D. (1) three  
(2) EOP-E-0, REACTOR TRIP OR SAFETY INJECTION
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 74**  
(1 point)

---

Given the following:

- The plant is at 100% power
- The Control Room receives a confirmed report of a fire in the Unit 2 Cable Spread Room (Fire Zone 19) at 0730

Which ONE of the following correctly completes the statements below?

IAW AOP-041, RESPONSE TO FIRE EVENT, the operator must \_\_\_\_ (1) \_\_\_\_.

The LATEST time that this action may be complete is \_\_\_\_ (2) \_\_\_\_.

- A. (1) close both Pressurizer PORV Block Valves (RC-535 and RC-536), ONLY  
(2) 0740
- B. (1) close both Pressurizer PORV Block Valves (RC-535 and RC-536), ONLY  
(2) 0745
- C. (1) close both Pressurizer PORV Block Valves (RC-535 and RC-536) AND place the PCV-455C and 456 Isolate Switches to the ISOLATE position  
(2) 0745
- D. (3) close both Pressurizer PORV Block Valves (RC-535 and RC-536) AND place the PCV-455C and 456 Isolate Switches to the ISOLATE position  
(4) 0740
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 75**  
(1 point)

---

Given the following:

- The plant is operating at 100% power

Subsequently, the following time line of events occur:

<u>TIME</u>	<u>EVENT</u>
1200	APP-009-C5, MAIN TRANSF TROUBLE, alarms
1203	The Outside AO reports that all Main Transformer Fans and Pumps are NOT operating
1204	The crew enters AOP-037, LARGE TRANSFORMER MALFUNCTIONS
1208	TIME NOW

IAW AOP-037, which ONE of the following identifies the EARLIEST time that the reactor must be tripped?

- A. IMMEDIATELY
  - B. 1211
  - C. 1214
  - D. 1231
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 76**  
(1 point)

---

Given the following:

- With the plant at 100% power a Feedwater Isolation Signal occurs
- Safety Injection actuates
- The crew is implementing FRP-S.1, RESPONSE TO NUCLEAR GENERATION-ATWS.

Which ONE of the following correctly completes the statement below?

IAW the FRP-S.1 basis document, the Turbine Stop Valves must be CLOSED within a MAXIMUM of \_\_\_\_ (1) \_\_\_\_ from event initiation.

This action \_\_\_\_ (2) \_\_\_\_ taken in order to maintain S/G inventory.

- A. (1) 30 seconds  
(2) is
  - B. (1) 30 seconds  
(2) is NOT
  - C. (1) 60 seconds  
(2) is
  - D. (1) 60 seconds  
(2) is NOT
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 77**  
(1 point)

---

Given the following:

- The crew is in EOP-E-3, STEAM GENERATOR TUBE RUPTURE, has completed tube rupture mitigation in the "C" S/G and is now selecting a recovery procedure
- Containment pressure is 4.5 psig due to a previously faulted "B" S/G
- A Loss of Off Site Power has occurred
- The Shift Manager has indicated that the recovery procedure should minimize radiation release

The following indications are observed:

- S/G "C" level is 86% and rising
- PZR level is 67% and stable

Which ONE of the following correctly completes the statement below?

Based on the current plant conditions RCS depressurization \_\_\_\_ (1) \_\_\_\_ required.

The CRS will select \_\_\_\_ (2) \_\_\_\_ as the recovery procedure.

### **(REFERENCES PROVIDED)**

- A. (1) is  
(2) EOP-ES-3.2, POST-SGTR COOLDOWN USING BLOWDOWN
  - B. (1) is  
(2) EOP-ES-3.3, POST-SGTR COOLDOWN USING STEAM DUMP
  - C. (1) is NOT  
(2) EOP-ES-3.2, POST-SGTR COOLDOWN USING BLOWDOWN
  - D. (1) is NOT  
(2) EOP-ES-3.3, POST-SGTR COOLDOWN USING STEAM DUMP
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 78**

(1 point)

---

Given the following:

- The plant is at 5% power
- One PZR PORV is CLOSED, has been isolated and cannot be manually cycled

Subsequently:

- A transient occurs causing a reactor trip and SI actuation
- The crew enters FRP-H.1, RESPONSE TO LOSS OF SECONDARY HEAT SINK
- Bleed and Feed criteria has been met
- Safety Injection has been manually actuated
- The OPERABLE PZR PORV has been OPENED

RC-567, HEAD VENT

RC-568, HEAD VENT

RC-569, PZR VENT

RC-570, PZR VENT

RC-571, PRT ISO

RC-572, CV ATMOS

IAW FRP-H.1, which ONE of the following actions, if any, is required to establish an adequate bleed path?

- A. No action is required, an adequate bleed path exists.
  - B. Open the Reactor Head Vent Valves, RC-571 and RC-572, ONLY.
  - C. Open the Pressurizer Vent Valves, RC-571 and RC-572, ONLY.
  - D. Open the Reactor Head Vent Valves, Pressurizer Vent Valves, RC-571 and RC-572.
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 79**  
(1 point)

---

Given the following:

- A Station Blackout has occurred from 100% power
- The crew is performing actions of ECA-0.0, LOSS OF ALL AC POWER
- Immediate actions are complete
- Offsite power is available to re-energize the Start-up Transformer (SUT)
- APP-009-B5, MAIN TRANSF FAULT TRIP, alarm is ILLUMINATED.

Subsequently:

- E-1 is restored with EDG "A"
- Transmission Maintenance has taken action such that APP-009-B5 is now extinguished.

86P, Generator Lockout Relay, Primary  
86BU, Generator Lockout Relay, Backup  
OP-603, ELECTRICAL DISTRIBUTION  
OP-603-3, RESETTING HIGH IMPEDANCE FAULTS

Which ONE of the following correctly completes the statements below regarding the actions performed IAW EOP-ECA-0.0 to restore normal power?

The CRS will direct the use of \_\_\_\_ (1) \_\_\_\_ to restore power via the SUT.

Prior to restoration of power via the SUT, the selected procedure \_\_\_\_ (2) \_\_\_\_ direct the crew to ensure the Main Generator Lockout Relays 86P and 86BU are reset.

- A. (1) OP-603  
(2) will NOT
  - B. (1) OP-603  
(2) will
  - C. (1) OP-603-3  
(2) will NOT
  - D. (1) OP-603-3  
(2) will
-



# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 80**  
(1 point)

---

Given the following:

- The plant tripped from 100% power when the "B" DC Bus de-energized.
- EPP-27, LOSS OF DC BUS "B", has been entered.
- The "B" battery and the in-service battery charger are determined to be damaged.

Which ONE of the following completes the statements below?

Generator Output Breakers OCBs 52/8 AND 52/9 \_\_\_\_ (1) \_\_\_\_.

Within four hours of the failure:

- The plant is at normal operating temperature and pressure
- The standby battery charger is aligned to energize DC Bus "B" to nominal voltage, while repair efforts continue

IAW Technical Specification 3.8, Electrical Power Systems, OPERABILITY of DC Bus "B"  
\_\_\_\_ (2) \_\_\_\_ restored.

- A. (1) must be tripped locally  
(2) is
  - B. (1) must be tripped locally  
(2) is NOT
  - C. (1) will trip on Generator Lockout  
(2) is
  - D. (1) will trip on Generator Lockout  
(2) is NOT
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 81**  
(1 point)

---

Given the following:

- The plant is operating at 100% power
- The “A” Emergency Diesel Generator is paralleled to Bus E-1 for periodic surveillance

Subsequently:

- The Load Dispatcher notifies the Shift Manager of a low frequency condition on the Grid
- The crew entered AOP-026, GRID INSTABILITY, and stabilized power at 98.5%
- The following conditions exist 1 minute after the load reduction:
  - Main Generator Frequency: 59.6 Hz and Stable
  - GEN Phase “A” and “B” Volts: 22 KV and Stable
  - GEN Phase “C” amps: 21.9 KA and Stable

Which ONE of the following correctly completes the statement below?

IAW AOP-026, the Generator output is \_\_\_\_ (1) \_\_\_\_.

Based on plant conditions, the CRS will \_\_\_\_ (2) \_\_\_\_.

### **(REFERENCES PROVIDED)**

- A. (1) acceptable  
(2) shutdown the “A” Emergency Diesel Generator
  - B. (1) unacceptable  
(2) shutdown the “A” Emergency Diesel Generator
  - C. (1) acceptable  
(2) isolate the Emergency Busses so that the busses are being solely supplied from the Emergency Diesel Generators
  - D. (1) unacceptable  
(2) isolate the Emergency Busses so that the busses are being solely supplied from the Emergency Diesel Generators
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 82**

(1 point)

---

Given the following:

- The plant was at 17% power with power being raised IAW GP-005, POWER OPERATION
- The Control Bank "D" rods continued to withdraw after the RO released the rod drive switch
- At 14:40 the crew has completed immediate actions and is implementing FRP-S.1, RESPONSE TO NUCLEAR POWER GENERATION – ATWS

Which ONE of the following correctly completes the statements below?

FRP-S.1 directs alignment of the emergency boration flow path by \_\_\_\_ (1) \_\_\_\_, then starting the Boric Acid pump aligned for BLEND.

After emergency boration is commenced the following conditions exist:

- Power is in the Intermediate Range with a negative Startup Rate
- Control rods H-8, D-6 and K-14 are stuck out, all other rods indicate on the bottom
- The STA is performing a Manual Determination of Shutdown Margin Boron concentration, and is not yet complete

Subsequently:

15:00 Control Rod D-6 drops fully into the core

15:15 Chemistry reports the RCS boron concentration is 1975 ppm

The EARLIEST time that the crew can stop the emergency boration is \_\_\_\_ (2) \_\_\_\_ .

MOV-350, BA TO CHARGING PMP SUCT

FCV-113A, BA TO BLENDER

FCV-113B, BLENDED MU TO CHG SUCT

- A. (1) opening MOV-350  
(2) 15:00
  - B. (1) opening FCV-113A and FCV-113B  
(2) 15:00
  - C. (1) opening MOV-350  
(2) 15:15
  - D. (1) opening FCV-113A and FCV-113B  
(2) 15:15
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

### Question: 83

(1 point)

---

Given the following:

- The plant at 25% power
- The crew enters AOP-012, PARTIAL LOSS OF CONDENSER VACUUM OR CIRCULATING WATER PUMP TRIP, due to a trip of the "A" Circulating Water Pump
- Condenser backpressure is approaching the Restricted Region of the Condenser Backpressure Limit Curve

Which ONE of the following correctly completes the statements below?

IAW AOP-012, it \_\_\_\_ (1) \_\_\_\_ expected that an AO will be dispatched to complete Attachment 1, Local Actions, during the mitigation of this event.

IAW AOP-012, the CRS \_\_\_\_ (2) \_\_\_\_.

- A. (1) is  
(2) will direct a reactor trip and go to EOP-E-0, REACTOR TRIP OR SAFETY INJECTION
- B. (1) is  
(2) will direct a turbine trip and go to AOP-007, TURBINE TRIP BELOW P-8
- C. (1) is not  
(2) will direct a turbine trip and go to AOP-007, TURBINE TRIP BELOW P-8
- D. (1) is not  
(2) will direct a reactor trip and go to EOP-E-0, REACTOR TRIP OR SAFETY INJECTION
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 84**  
(1 point)

---

Given the following:

- Core Alterations are on-going in accordance with GP-010, REFUELING
- R-2, CV Low Range, alarms (HIGH LED is LIT)

Which ONE of the following correctly completes the statement below?

To determine the correct R-2 setpoint the operator will use \_\_\_\_ (1) \_\_\_\_.

If R-2 was subsequently determined to be inoperable, Core Alterations \_\_\_\_ (2) \_\_\_\_.

- A. (1) Section 3.1, Monitor Alarm Setpoint Determination, of the Offsite Dose Calculation Manual (ODCM)  
(2) must be suspended immediately
  - B. (1) the Setpoint Change and Log Record of OMM-014, Radiation Monitor Setpoints  
(2) must be suspended immediately
  - C. (1) the Setpoint Change and Log Record of OMM-014, Radiation Monitor Setpoints  
(2) may continue, provided that continuous HP Technician coverage and a Portable Area Radiation Monitor are provided
  - D. (1) Section 3.1, Monitor Alarm Setpoint Determination, of the Offsite Dose Calculation Manual (ODCM)  
(2) may continue, provided that continuous HP Technician coverage and a Portable Area Radiation Monitor are provided
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 85**

(1 point)

---

Given the following:

- A LOCA has occurred and NO ECCS Pumps are operating
- The crew is implementing FRP-C.2, RESPONSE TO DEGRADED CORE COOLING
- While depressurizing all intact S/G's they receive a RED condition on CSF-4, Integrity
- RCS pressure is 200 psig

Which ONE of the following identifies the required implementation of procedures for this event?

FRP-P.1, RESPONSE TO IMMINENT PRESSURIZED THERMAL SHOCK

- A. Remain in FRP-C.2 until completed, THEN transition to FRP-P.1
  - B. Transition to FRP-P.1 and perform until completed, THEN transition to FRP-C.2
  - C. Transition to FRP-P.1 and when it is determined that RCS Integrity does not exist at Step 2, return to FRP-C.2
  - D. Transition to FRP-P.1 and initiate a soak, THEN perform FRP-C.2 actions that do not cooldown or raise RCS pressure until the soak is complete
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 86**  
(1 point)

---

Given the following:

- The plant is operating at 25% power
- A plant shutdown is in progress
- The following timeline of events occur:

0800	Pressurizer Level Transmitter LT-460 starts to fluctuate rapidly characterized by abrupt spiking in the high and low direction
0800	A channel check indicates that all other Pressurizer level indications indicate on program and stable
0800	ITS LCO 3.3.1, REACTOR PROTECTION SYSTEM (RPS) INSTRUMENTATION, action statement for LT-460, PZR level, was entered
0802	The lowest spike on the LT-460 indication reaches 13%
0804	The highest spike on the LT-460 indication reaches 92%
0810	The crew completes removing the instrument from service IAW OWP-030, PRESSURIZER LEVEL TRANSMITTERS (PLT)
0830	Plant down-power is initiated
1000	Reactor Power is stabilized at 8% and REACTOR TRIP BLOCK P-7 status light is ILLUMINATED

Which ONE of the following correctly completes the statements below?

The EARLIEST time that a single Reactor Protection Bistable will illuminate due to the erratic behavior of LT-460 is \_\_\_\_ (1) \_\_\_\_.

At 1000, IAW ITS LCO 3.3.1, the action statement for LT-460, PZR level \_\_\_\_ (2) \_\_\_\_ applicable.

- A. (1) 0802  
(2) is NOT
  - B. (1) 0802  
(2) is
  - C. (1) 0804  
(2) is
  - D. (1) 0804  
(2) is NOT
-

# Robinson Nuclear Plant

## ILC15 RNP SRO NRC Examination

**Question: 87**  
(1 point)

---

Given the following initial plant conditions:

- The plant was at 100% power when a reactor trip and safety injection occurred
- R-31C, STEAM LINE RADIATION MONITOR, is in alarm and rising
- The crew is implementing EOP-E-3, STEAM GENERATOR TUBE RUPTURE with the following conditions:
  - ALL MSIVs are closed
  - Containment pressure 0.8 psig
  - RWST level is 50% and lowering due to a leak in the RWST
  - S/G Parameters:

	<u>Pressure</u>	<u>NR Level</u>
A	960 psig and lowering slowly	22% and rising
B	960 psig and lowering slowly	7% and rising
C	450 psig and lowering rapidly	7% and lowering

Which ONE (1) of the following correctly completes the statements below?

IAW EOP-E-3 the crew will continue to feed \_\_\_\_ (1) \_\_\_\_ .

When the CRS is transitioning from EOP-E-3, a transition to EOP-ECA-3.1 \_\_\_\_ (2) \_\_\_\_ required PRIOR to entering EOP-ECA-3.2.

EOP-E-2, FAULTED STEAM GENERATOR ISOLATION  
EOP-ECA-3.1, SGTR WITH LOSS OF REACTOR COOLANT: SUBCOOLED RECOVERY DESIRED  
EOP-ECA-3.2, SGTR WITH LOSS OF REACTOR COOLANT: SATURATED RECOVERY DESIRED

- A. (1) "A" AND "B" Steam Generators ONLY  
(2) is
- B. (1) ALL Steam Generators  
(2) is
- C. (1) "A" AND "B" Steam Generators ONLY  
(2) is NOT
- D. (1) ALL Steam Generators  
(2) is NOT
-



# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 88**  
(1 point)

---

Given the following:

- The plant is at 100% power
- The crew has just completed swapping Service Water Booster Pumps from “A” to “B” IAW OP-903, SERVICE WATER SYSTEM
- The following data was obtained after the pump swap:
  - HVH-1 – FI-1698A: 830 gpm
  - HVH-2 – FI-1698B: 810 gpm
  - HVH-3 – FI-1698C: 745 gpm
  - HVH-4 – FI-1689D: 805 gpm

Which ONE of the following correctly completes the statements below?

Entry into Technical Specification LCO 3.6.6, Containment Spray and Cooling Systems  
\_\_\_\_(1)\_\_\_\_ required.

OP-903 states that the electrical analysis for the Service Water Booster Pump motors may be exceeded if total HVH Unit Service Water Flow exceeds \_\_\_\_ (2) \_\_\_\_.

- A. (1) is NOT  
(2) 4000 gpm
  - B. (1) is NOT  
(2) 4220 gpm
  - C. (1) is  
(2) 4220 gpm
  - D. (1) is  
(2) 4000 gpm
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 89**  
(1 point)

---

Given the following:

- A plant cooldown is in progress
- RCS temperature is 380°F
- “B” Instrument Air Compressor is in AUTO with the “B” Instrument Air Dryer in service
- “D” Instrument Air Compressor is operating
- “A” Instrument Air Compressor and “A” Instrument Air Dryer are out of service

Subsequently:

- The “D” Instrument Air Compressor trips, followed by the “B” Instrument Air Dryer clogging
- The crew has entered AOP-017, LOSS OF INSTRUMENT AIR
- RCS temperature is slowly rising
- IA header pressure is 40 psig and rising slowly

Which ONE of the following correctly completes the statements below?

Based upon current conditions, FCV-1740, Air Dryer Hi DP Flow Control valve  
\_\_\_\_(1)\_\_\_\_ be open.

IAW AOP-017, the CRS will direct use of \_\_\_\_ (2) \_\_\_\_ as the preferred method to control  
RCS temperature.

Attachment 2, Nitrogen Alignment to Steam Line PORVs  
Attachment 3, Manual Steam Dump of S/Gs

- A. (1) will  
(2) Attachment 2
  - B. (1) will NOT  
(2) Attachment 2
  - C. (1) will  
(2) Attachment 3
  - D. (1) will NOT  
(2) Attachment 3
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 90**  
(1 point)

---

Given the following plant conditions:

- The plant is operating at 100% power

Subsequently:

- A Large Break LOCA and loss of offsite power has occurred
- The "A" EDG has failed to start
- HVH-3, CV RECIRC FAN C, failed to AUTO start on the SI actuation
- SI-880D, CV SPRAY PUMP "B" DISCHARGE, failed to automatically OPEN
- Containment pressure is 13 psig and rising

The Shift Manager is evaluating whether or not the EAL Threshold "Containment pressure  $\geq$  10 psig with  $\leq$  one full train of depressurizing equipment," is met.

Which ONE of the following assesses whether or not one full train of depressurizing equipment is operating, and if not, identifies the MINIMUM action that must be taken from the Control Room to ensure that one full train is running?

- A. One full train of depressurization equipment is operating.
  - B. One full train of depressurization equipment is NOT operating; HVH-3 must be manually started.
  - C. One full train of depressurization equipment is NOT operating; HVH-3 must be manually started OR SI-880D must be OPENED.
  - D. One full train of depressurization equipment is NOT operating; HVH-3 must be manually started AND SI-880D must be OPENED.
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 91**  
(1 point)

---

Given the following:

- The plant was at 100% when Control Rod H-12 dropped
- The crew lowered reactor power to 68% IAW AOP-001
- During this load reduction AFD stabilized at -19

Which ONE of the following correctly completes the statements below?

NI-41 and NI-43 will indicate \_\_\_\_ (1) \_\_\_\_ than NI-42 and NI-44 due to the Rod H-12 drop.

IAW AOP-001, the CRS \_\_\_\_ (2) \_\_\_\_ required to use Attachment 1 to reduce power to less than or equal to 50% within 30 minutes.

AOP-001, MALFUNCTION OF REACTOR CONTROL SYSTEM  
AOP-001, ATTACHMENT 1, TURBINE LOAD ADJUSTMENT

**(REFERENCE PROVIDED)**

- A. (1) higher  
(2) is
  - B. (1) lower  
(2) is NOT
  - C. (1) lower  
(2) is
  - D. (1) higher  
(2) is NOT
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 92**  
(1 point)

---

Given the following:

- Monitor Tank "A" is being prepared for release
- Tank pH was sampled and found to be too low

Which ONE of the following correctly completes the statements below?

IAW OP-705, WASTE LIQUID RELEASE AND RECIRCULATION Monitor Tank "A" is recirculated for a MINIMUM period of time to ensure \_\_\_\_ (1) \_\_\_\_.

IAW EMP-023, LIQUID WASTE RELEASE AND SAMPLING, pH must be adjusted to a MINIMUM of \_\_\_\_ (2) \_\_\_\_ before the tank is released.

- A. (1) representative samples are obtained  
(2) > 5.0
  - B. (1) representative samples are obtained  
(2) > 6.0
  - C. (1) radioactive hot spots do not occur during the release  
(2) > 5.0
  - D. (1) radioactive hot spots do not occur during the release  
(2) > 6.0
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 93**  
(1 point)

---

Which ONE of the following correctly completes the statements below?

The TRM limitation on the quantity of radioactivity permitted in \_\_\_\_ (1) \_\_\_\_ Waste Gas Decay Tank(s) is  $\leq 1.9 \times 10^4$  Curies noble gas.

The bases for TRM 3.21 is concerned with exposure to \_\_\_\_ (2) \_\_\_\_.

- A. (1) each  
(2) personnel in the Control Room
  - B. (1) ALL  
(2) personnel in the Control Room
  - C. (1) each  
(2) members of the public at the site boundary
  - D. (1) ALL  
(2) members of the public at the site boundary
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 94**  
(1 point)

---

Given the following:

- The plant was at 35% power when a valid alarm on R-19 occurred
- The crew has just entered AOP-035, S/G TUBE LEAK
- Chemistry is sampling S/G's to determine primary to secondary leakage rate

Which ONE of the following correctly completes the statements below?

IAW AOP-035 the crew will use \_\_\_\_ (1) \_\_\_\_ as a reliable diverse indication to confirm the S/G tube leak.

Subsequently, Chemistry reports that the "B" S/G tube leakage rate is 0.071 gpm

IAW AOP-035, the CRS is required to initiate a plant shutdown and place the unit in MODE 3 within \_\_\_\_ (2) \_\_\_\_.

R-15, CONDENSER AIR EJECTOR GAS  
R-19, A, B, C STEAM GENERATOR BLOWDOWN  
R-24, A, B, C, MAIN STEAM LINE N-16

- A. (1) R-15  
(2) 3 hours
  - B. (1) R-15  
(2) 6 hours
  - C. (1) R-24  
(2) 3 hours
  - D. (1) R-24  
(2) 6 hours
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 95**  
(1 point)

---

Given the following:

- The plant is operating at 100% power.
- The crew has tested SW-PMP-A, "SERVICE WATER PUMP A" on the previous shift per OST-302-1, SERVICE WATER PUMPS A & B INSERVICE TEST" and recorded elevated vibration level readings that are in the REQUIRED ACTION range.
- Maintenance now has a plan to repair the pump.

Subsequently, the following sequence of events occur:

- SW-PMP-A is shutdown, placed under clearance, and repairs completed.
- SW-PMP-A is re-energized and realigned in preparation for retesting.
- SW-PMP-A is retested IAW OST-302-1 and vibrations are now found in the ALERT range.

Based on the above information, Which ONE of the following correctly completes the statements below?

IAW OST-302-1, SW-PMP-A is required to be declared INOPERABLE when \_\_\_\_ (1) \_\_\_\_.

Based on the retest of SW-PMP-A, the pump is \_\_\_\_ (2) \_\_\_\_.

- A. (1) the pump is placed under clearance  
(2) INOPERABLE
  - B. (1) the pump is placed under clearance  
(2) OPERABLE
  - C. (1) when OST 302-1 was completed on the previous shift  
(2) INOPERABLE
  - D. (1) when OST 302-1 was completed on the previous shift  
(2) OPERABLE
-



# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 96**  
(1 point)

---

Given the following:

- The plant is operating at 100% power

Subsequently, the following timeline of events occurs:

0200	EDG "A" is declared inoperable due to wrong oil being added during maintenance
0400	EDG "B" is declared inoperable due to mechanical issues
0400	The crew enters Technical Specification LCO 3.0.3
0500	A downpower is initiated
0900	Mode 3 is entered

Which ONE of the following correctly completes the statements below?

IAW Technical Specifications LCO 3.8.1, the operator must verify the correct breaker alignment and indicated power availability for the offsite circuit (offsite power checks) no later than \_\_\_\_ (1) \_\_\_\_.

Based on this, the LATEST time that the plant must be placed in Mode 4 is \_\_\_\_ (2) \_\_\_\_.

- A. (1) 0300  
(2) 1500
- B. (1) 0300  
(2) 1700
- C. (1) 0500  
(2) 1500
- D. (1) 0500  
(2) 1700
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 97**  
(1 point)

---

Given the following:

- The Inside AO needs to enter a room during EOP implementation
- The TSC and OSC are NOT activated
- General dose rate levels in the room range from 25-45 mr/hr
- The survey map shows the following on contact readings:
  - Point 1 is 100 mr/hr at 30 cm
  - Point 2 is 500 mr/hr at 30 cm
  - Point 3 is 1100 mr/hr at 30 cm

Which ONE of the following correctly completes the statements below?

Based on the plant indications, the radiological posting required is \_\_\_\_ (1) \_\_\_\_.

Under the stated plant conditions, the Shift Manager \_\_\_\_ (2) \_\_\_\_ grant access to this area.

- A. (1) Very High Radiation Area  
(2) may
  - B. (1) Very High Radiation Area  
(2) may NOT
  - C. (1) Locked High Radiation Area  
(2) may NOT
  - D. (1) Locked High Radiation Area  
(2) may
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 98**  
(1 point)

---

Given the following:

- A General Emergency has been declared
- The Dose Rate in Pipe Alley is 105 R/hr
- A 10-minute evolution in Pipe Alley is necessary to conduct repair and re-entry activities
- Two two-person teams have been briefed and are prepared to enter Pipe Alley to complete the task

Which ONE of the following correctly completes the statement below?

IAW EPOSC-04, EMERGENCY WORK CONTROL, the \_\_\_\_ (1) \_\_\_\_ can authorize performance of the evolution in Pipe Alley.

The MINIMUM number of two-person teams needed to be dispatched to complete the 10-minute evolution in Pipe Alley without exceeding any emergency exposure limits is \_\_\_\_ (2) \_\_\_\_.

- A. (1) Site Emergency Coordinator  
(2) one
  - B. (1) Site Emergency Coordinator  
(2) two
  - C. (1) Radiological Control Director  
(2) one
  - D. (1) Radiological Control Director  
(2) two
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 99**  
(1 point)

---

Given the following:

- The plant is operating at 100% power.
- At 0100, LCO 3.4.13, RCS OPERATIONAL LEAKAGE, was entered due to an 15 gpm leak from a crack on the spray line penetration weld at the PZR
- The crew commenced a shutdown at 0200
- The plant reached MODE 3 at 0545 when the OATC manually tripped the Rx as part of the plant shutdown per GP-006-1, NORMAL PLANT SHUTDOWN FROM POWER OPERATION TO HOT SHUTDOWN.

Which ONE of the following identifies the LATEST time at which the NRC must be notified?

**(REFERENCES PROVIDED)**

- A. 0130
  - B. 0215
  - C. 0315
  - D. 0600
-

# Robinson Nuclear Plant

## *ILC15 RNP SRO NRC Examination*

**Question: 100**  
(1 point)

---

Given the following:

- The plant is at 100% power when the following annunciators alarm:
  - APP-001-C5, RCP Standpipe HI/LO LVL
  - APP-001-E2, RCP #1 SEAL LEAKOFF LO FLOW
- The crew has entered AOP-018, REACTOR COOLANT PUMP ABNORMAL CONDITIONS

Which ONE of the following correctly completes the statements below?

APP-001-C5, RCP Standpipe HI/LO LVL is due to \_\_\_\_ (1) \_\_\_\_.

Subsequently:

- Indicated "B" RCP #1 seal leakoff flow: 0.6 gpm
- The BOP performed Attachment 2 of AOP-018, RCP #2 SEAL LEAK RATE CALCULATION, and calculates Total "B" RCP #1 seal leakoff flow at 6.5 gpm
- "B" RCP Bearing Temperatures are 130°F and stable
- "B" RCP #1 Seal Leakoff Temperature is 125°F and stable

IAW AOP-018 the CRS will direct the crew to continue to monitor RCP parameters and place the plant in MODE 3 within \_\_\_\_ (2) \_\_\_\_ using GP-006-1, NORMAL PLANT SHUTDOWN FROM POWER OPERATION TO HOT SHUTDOWN.

- A.     (1) high level  
          (2) 3 hours
  - B.     (1) high level  
          (2) 8 hours
  - C.     (1) low level  
          (2) 3 hours
  - D.     (1) low level  
          (2) 8 hours
-

# RCS P/T Limits 3.4.3

## MATERIALS PROPERTIES BASE

CONTROLLING MATERIAL: Upper Shell Plate W10201-1

Limiting ART Values at 35 EFPY: 1/4T, 167°F

3/4T, 147°F

Curves applicable for heatup rates up to 60 °F/Hr for Service period up to 35 EFPY.

Heatup Curves include +10°F and -60 psig allowance For instrumentation error.

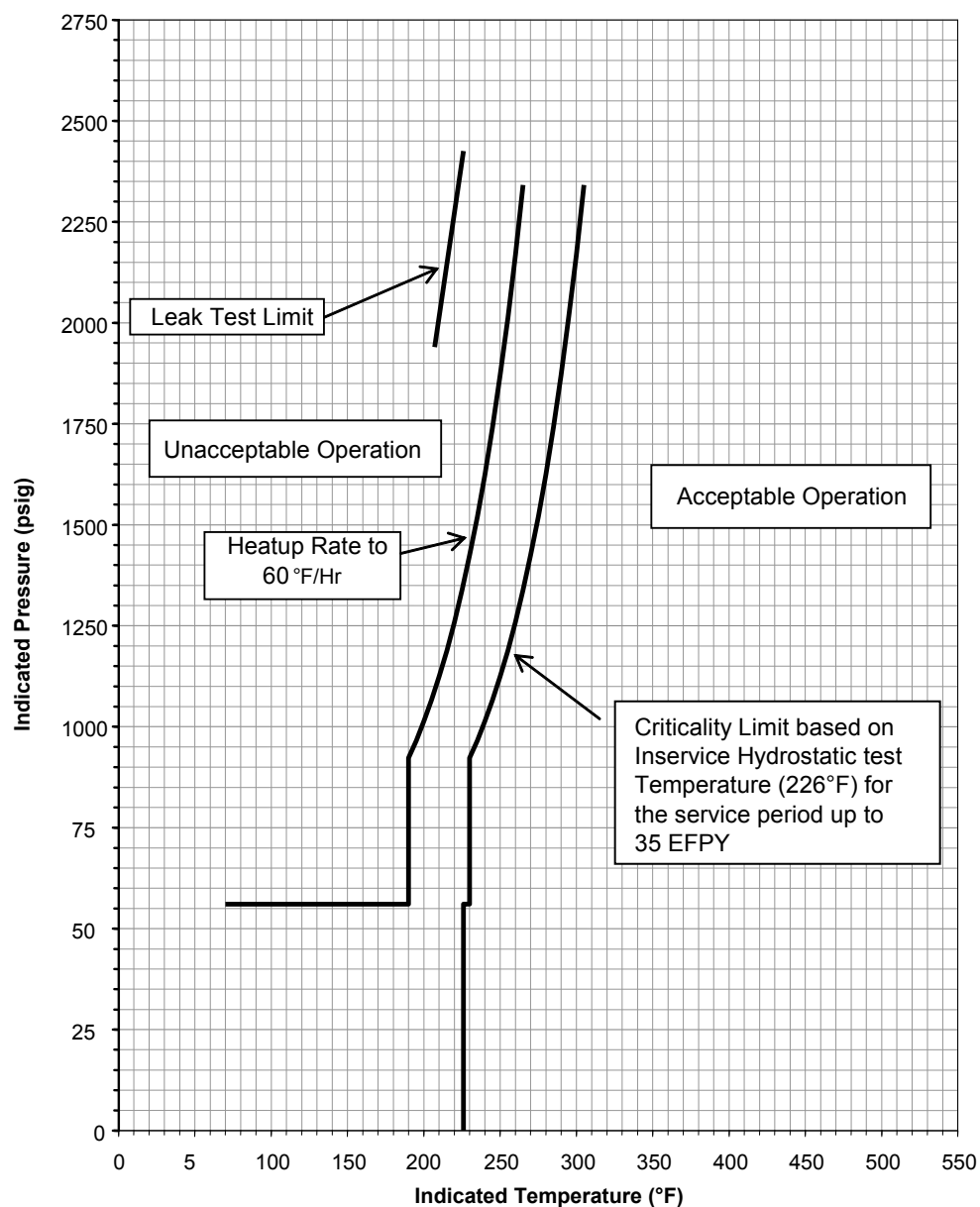


Figure 3.4.3-1  
Reactor Coolant System Heatup Limits  
Applicable Up to 35 EFPY

**MATERIALS PROPERTY BASE**

**CONTROLLING MATERIAL:** Upper Shell Plate W10201-1  
and Girth Weld 10-273

**Limiting ART Values at 35 EFY:** 1/4T, 167°F and 242°F  
3/4T, 147°F and 172°F

Curves applicable for cooldown rates up to 100°F/Hr for the service period up to 35 EFY. Curves include +10°F and -60 PSIG allowance for instrument error.

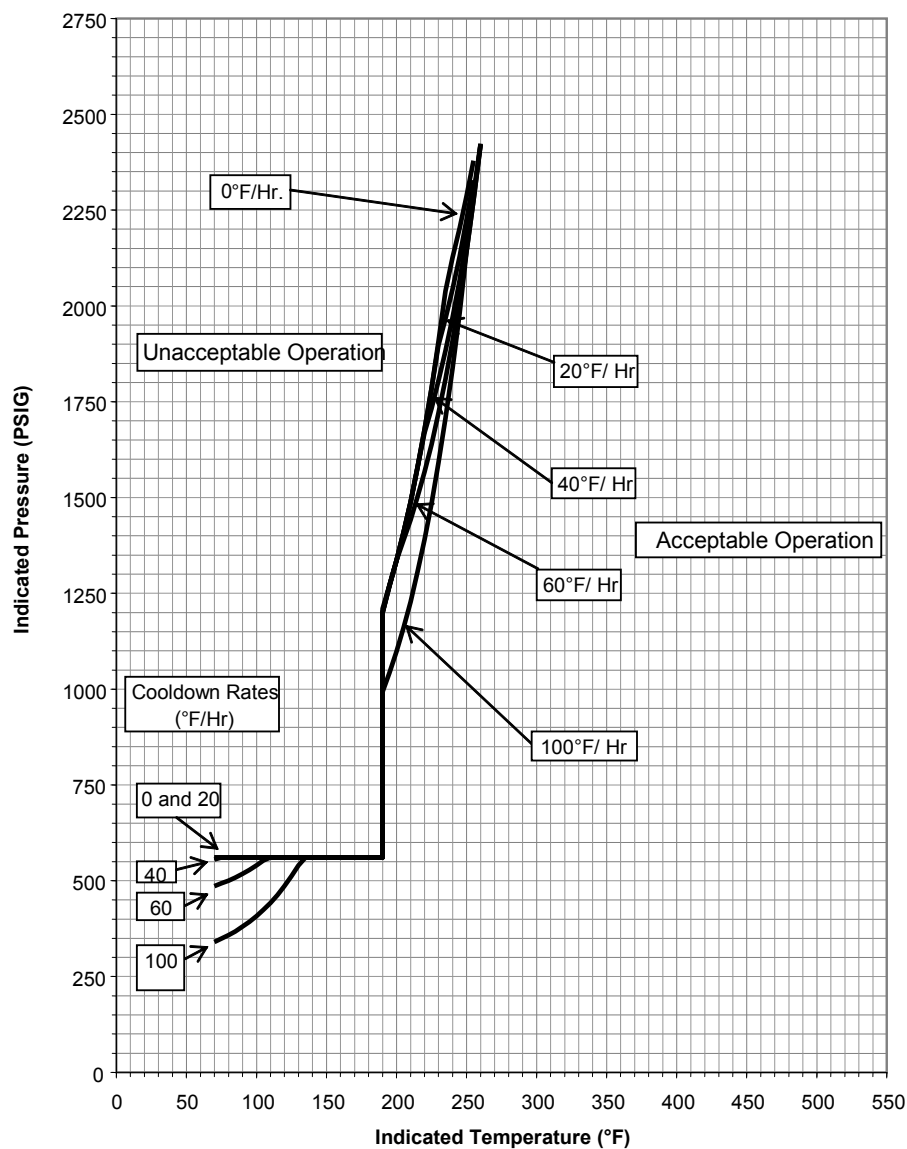
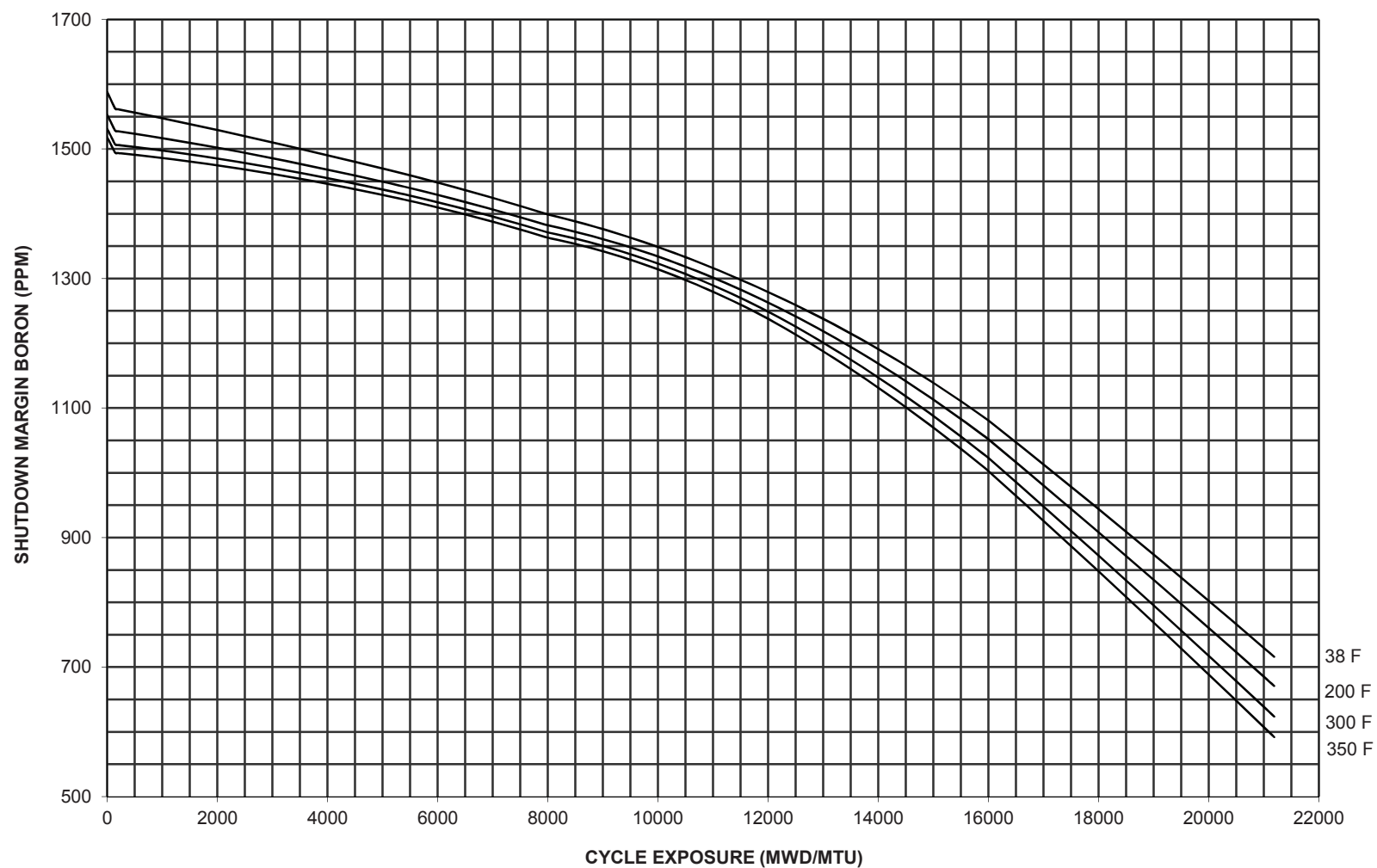


Figure 3.4.3-2  
Reactor Coolant System Cooldown Limits  
Applicable Up to 35 EFY

CURVE 1.14  
HBR2 CYCLE 30  
BORON CONCENTRATION REQUIRED TO MAINTAIN  
A MINIMUM OF 2.6% SHUTDOWN MARGIN (ARI-MRR)





STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

Attachment 3RCS Pressure And Charging Flow Control Handout

(Page 1 of 1)

\*\*\*\*\*

CAUTIONRCS AND Ruptured S/G(s) pressures must be maintained LESS THAN 1060 PSIG.

\*\*\*\*\*

1. **CONTROL RCS Pressure AND Charging Flow To MINIMIZE RCS-To-Secondary Leakage:**

- a. PERFORM appropriate action(s) from table below:

PZR Level	RUPTURED S/G(s) LEVEL		
	RISING	LOWERING	OFF SCALE HIGH
LESS THAN OR EQUAL TO 27% [44%]	<ul style="list-style-type: none"> <li>● RAISE Charging flow</li> <li>● DEPRESSURIZE RCS using Step 1.c</li> </ul>	RAISE Charging flow	<ul style="list-style-type: none"> <li>● RAISE Charging flow</li> <li>● MAINTAIN RCS <u>AND</u> Ruptured S/G(s) pressures equal</li> </ul>
BETWEEN 27% [44%] <u>AND</u> 50%	DEPRESSURIZE RCS using Step 1.c	TURN ON PZR Heaters	MAINTAIN RCS <u>AND</u> Ruptured S/G(s) pressures equal
BETWEEN 50% <u>AND</u> 73% [66%]	<ul style="list-style-type: none"> <li>● DEPRESSURIZE RCS using Step 1.c</li> <li>● REDUCE Charging flow</li> </ul>	TURN ON PZR Heaters	MAINTAIN RCS <u>AND</u> Ruptured S/G(s) pressures equal
GREATER THAN OR EQUAL TO 73% [66%]	REDUCE Charging flow	TURN ON PZR Heaters	MAINTAIN RCS <u>AND</u> Ruptured S/G(s) pressures equal

- b. CHECK Ruptured S/G Pressure - LESS THAN 1060 PSIG

- b. CONTROL Ruptured S/G pressure LESS THAN 1060 psig using Step 1.a.

- c. USE Normal PZR Spray as necessary to depressurize RCS per Table in Step 1.a

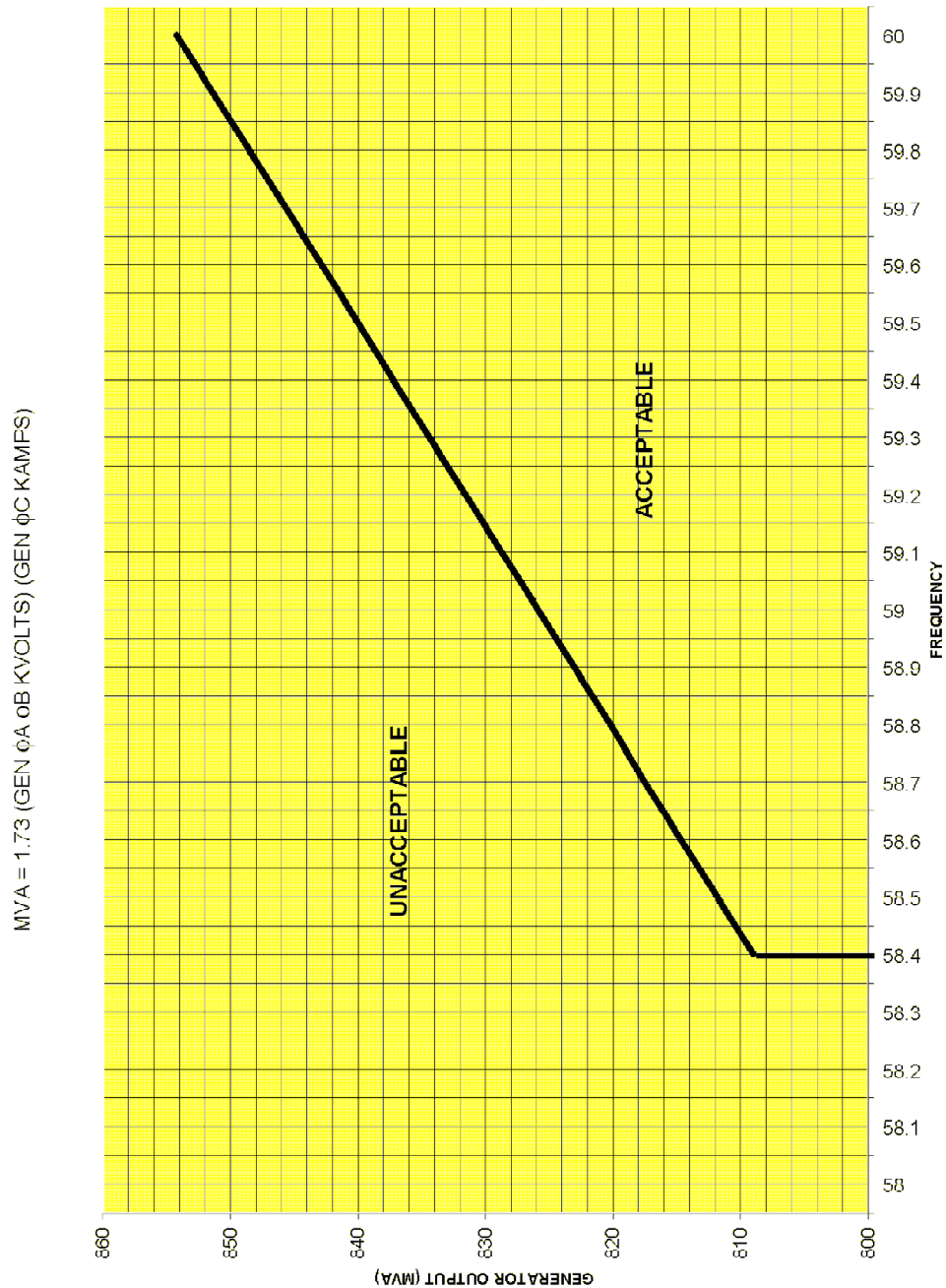
- c. IF Letdown is in service,
- THEN
- USE Aux PZR Spray per Supplement G, Establishing Aux PZR Spray.

IF Aux PZR Spray is NOT available OR effective, THEN USE one PZR PORV.

- END -

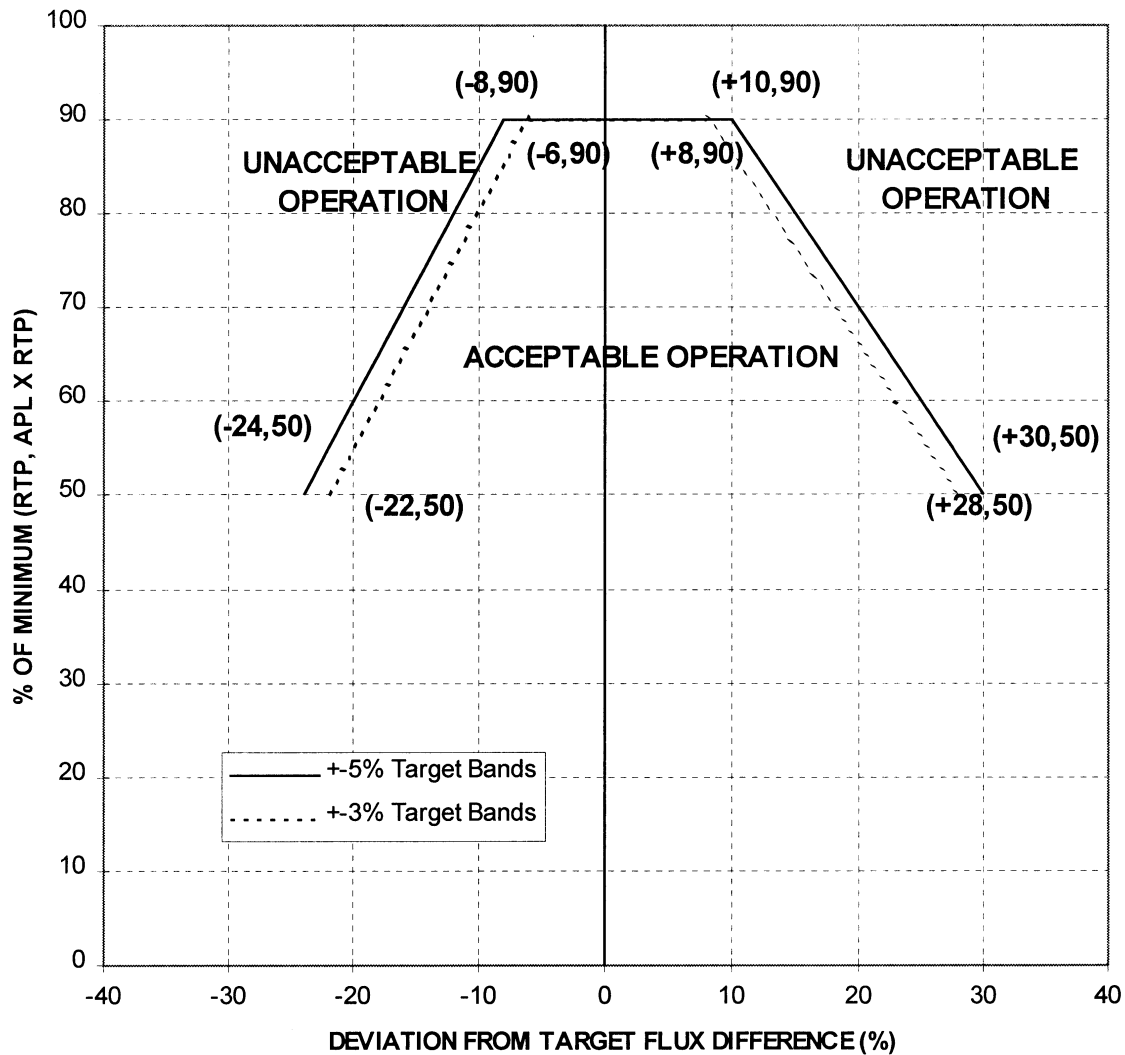
ATTACHMENT 1MAXIMUM GENERATOR MVA OUTPUT vs SYSTEM FREQUENCY

(Page 1 of 1)

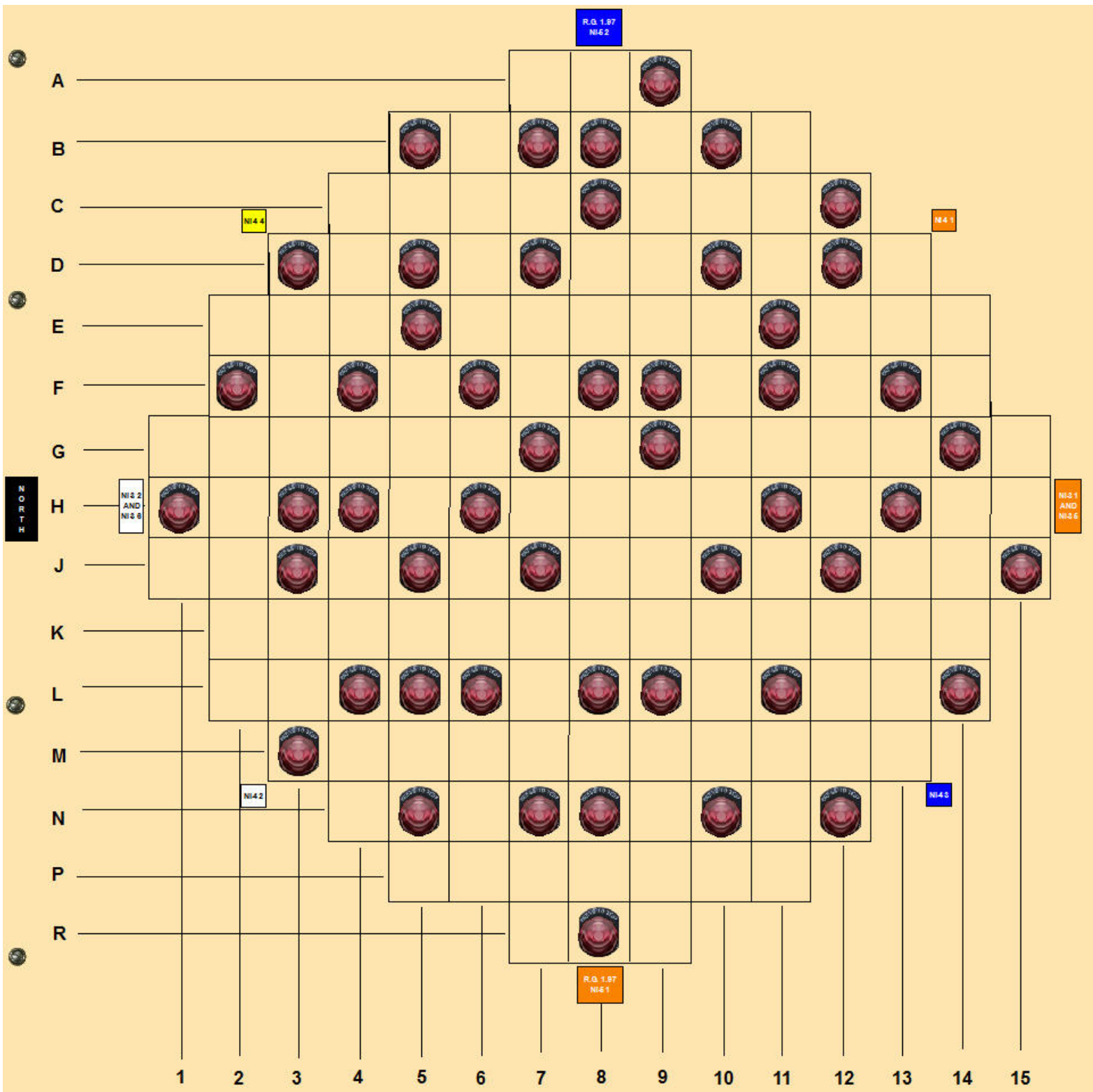


ATTACHMENT 10.1  
Page 12 of 13  
**HBRSEP UNIT NO. 2, CYCLE 30**  
**CORE OPERATING LIMITS REPORT**  
**REVISION 0**

Figure 4.0, Allowable Deviation from Target Flux  
Difference



**NOTE:** For power levels above 90%, power operation is allowed within the target bands ( $\pm 3\%$  and  $\pm 5\%$ ).



ATTACHMENT 11.1  
Page 1 of 9  
**IMMEDIATE (ONE HOUR) NOTIFICATIONS TO THE NRC**

IMMEDIATE (ONE HOUR) NOTIFICATIONS TO THE NRC			
10 CFR 50.72 states that immediate reports shall be made to the NRC Operations Center of these Emergency Events via the NRC Emergency Telecommunications System (ETS) as specified in the Emergency Plan. 10 CFR 50.72 additionally identifies Non-Emergency Events which are to be reported within one-hour, four-hours, or eight hours to the NRC. ETS Telephones, which are identified, are located in the Control Room, the TSC, and the EOF. In the event that the ETS is not available, 10 CFR 50.72(a)(2) permits the use of commercial telephone.			
EVENT	KEY WORDS	REQUIREMENT	EXAMPLES
<b>NOTE:</b> 10 CFR 50.72 recognizes the Emergency Plan and its four Emergency Classes of Unusual Event, Alert, Site Area Emergency and General Emergency.			
<b>EMERGENCIES</b>  10 CFR 50.72(a)(i) 10 CFR 30.32(i)(3)(viii) 10 CFR 40.31(i)(3)(viii) 10 CFR 72.75(a)	Emergency Unusual Event Alert Site Area Emergency General Emergency ISFSI	HBRSEP shall notify the NRC of the declaration of any of the Emergency Classes specified in the Emergency Plan.  (See EPNOT-01)	<ul style="list-style-type: none"> <li>– Declaration of an Unusual Event, Alert, Site Area Emergency, or General Emergency.</li> <li>– Discovery of an event that should have resulted in an Emergency Classification, but no emergency was declared.</li> <li>– Discovery that a declared emergency exceeded the Emergency Action Levels for a higher emergency declaration, but the higher classification was not declared.</li> </ul>
<b>ERDS ACTIVATION</b>  10 CFR 50.72(a)(4)	ERDS Emergency	HBRSEP shall activate the ERDS as soon as possible but not later than one hour after declaring an Alert, Site Area Emergency, or General Emergency.	<ul style="list-style-type: none"> <li>– An Alert, Site Area Emergency, or General Emergency is declared.</li> </ul>
<b>DEVIATION FROM TS (10 CFR 50.54(X))</b>  10 CFR 50.72(b)(1)	Deviation Departure License Condition	Any deviation from the TS authorized pursuant to 10 CFR 50.54(x).	<ul style="list-style-type: none"> <li>– Intentional deviation from an approved plant procedure in order to preserve plant safety 10 CFR 50.54(x).</li> </ul>

ATTACHMENT 11.1  
Page 2 of 9  
**IMMEDIATE (ONE HOUR) NOTIFICATIONS TO THE NRC**

<b>IMMEDIATE (ONE HOUR) NOTIFICATIONS TO THE NRC</b>			
HBRSEP shall immediately notify the NRC Operations Center via ETS as soon as practical and in all cases within one hour of the occurrence of any of the following:			
<b>EVENT</b>	<b>KEY WORDS</b>	<b>REQUIREMENT</b>	<b>EXAMPLES</b>
<b>SAFETY LIMIT, LIMITING SAFETY SYSTEM SETTING EXCEEDED</b>  10 CFR 50.36(c)(1)(i)(A) UFSAR Section 17.3A, Paragraph 3.1.a	Safety Limit Limiting Safety System Setting	If any safety limit is exceeded, shut down the reactor. HBRSEP shall notify the NRC [within 1 hour via ETS per 10 CFR 50.72(a)(1), See Emergency Plan Procedures]. Operation must not be resumed until authorized by the NRC.  NRC Region II must also be notified within 1 hour and the Vice President - Robinson Nuclear Plant within 24 hours	– The limits of TS Figure 2.1.1-1 are exceeded.
<b>SAFETY SYSTEM DOES NOT FUNCTION AS REQUIRED</b>  10 CFR 50.36(c)(1)(ii)(A)	ESF RPS Limiting Safety System Setting	HBRSEP shall notify the NRC if the automatic safety system [to correct an abnormal situation before a safety limit is exceeded] has been determined not to function as required.	– A failure mechanism is discovered that indicates that the RPS will not function to trip the reactor under certain required conditions.

ATTACHMENT 11.1  
Page 3 of 9  
**IMMEDIATE (ONE HOUR) NOTIFICATIONS TO THE NRC**

<b>IMMEDIATE (ONE HOUR) NOTIFICATIONS TO THE NRC - SECURITY SAFEGUARDS EVENTS</b>			
HBRSEP shall notify the NRC Operations Center via the ETS within one hour* after discovery of the safeguards events described as follows (10 CFR 73.71(b)(1)):			
<b>EVENT</b>	<b>KEY WORDS</b>	<b>REQUIREMENT</b>	<b>EXAMPLES</b>
<b>THEFT/UNLAWFUL DIVERSION OF SNM OR SPENT FUEL SHIPMENT</b>  10 CFR 73.71(a)(1)	SNM Spent Fuel Security Safeguards	Any discovery of the loss of any shipment of SNM or spent fuel, and within one hour after recovery of or accounting for such lost shipment	– Shipment Emergency Event
<b>THEFT/UNLAWFUL DIVERSION OF SNM</b>  10 CFR 73.71(b)(1) 10 CFR 73, Appendix G, I(a)(1)	Theft of SNM Diversion Security Safeguards	Any event in which there is reason to believe that a person has committed or caused, or attempted to commit or cause, or has made a credible threat to commit or cause: (1) A theft or unlawful diversion of SNM	– Shipment Emergency Event
<b>SABOTAGE OF PLANT EQUIPMENT</b>  10 CFR 73.71(b)(1) 10 CFR 73, Appendix G, I(a)(2)	Sabotage Damage to Plant SNM Spent Fuel Security Safeguards	[Any event in which there is reason to believe that a person has committed or caused, or attempted to commit or cause, or has made a credible threat to commit or cause:] (2) Significant physical damage to a power reactor...or its equipment or carrier equipment transporting nuclear fuel or spent nuclear fuel, or to the nuclear fuel or spent fuel a facility or carrier possesses.	– Shipment Emergency Event – Security Event (AD-SY-ALL-0150)

- \* In response to NRC Bulletin 2005-02, RNP committed to make an accelerated call to the NRC within approximately 15 minutes following discovery of an imminent threat or attack against the station. The primary purpose is to allow for the NRC to timely notify other licensees of a potential common threat. The accelerated call should not be allowed to interfere with plant or personnel safety, physical security response, or notification of local law enforcement agencies. The information provided in the accelerated call can be limited to:
- Site name
  - Emergency Classification – if already determined – do not delay call for the purpose of classifying
  - Nature of the threat – briefly described, if known, including the type of attack (e.g., armed assault by land, water or aircraft) and the attack status (e.g., imminent, in progress, or repelled)

ATTACHMENT 11.1  
Page 4 of 9  
**IMMEDIATE (ONE HOUR) NOTIFICATIONS TO THE NRC**

<b>IMMEDIATE (ONE HOUR) NOTIFICATIONS TO THE NRC - SECURITY SAFEGUARDS EVENTS</b>			
HBRSEP shall notify the NRC Operations Center via the ETS within one hour* after discovery of the safeguards events described as follows (10 CFR 73.71(b)(1)):			
<b>EVENT</b>	<b>KEY WORDS</b>	<b>REQUIREMENT</b>	<b>EXAMPLES</b>
<b>UNAUTHORIZED TAMPERING WITH PLANT EQUIPMENT</b>  10 CFR 73, Appendix G, I(a)(3)	Unauthorized Use Tampering Security System Safeguards	[Any event in which there is reason to believe that a person has committed or caused, or attempted to commit or cause, or has made a credible threat to commit or cause:] (3) Interruption of normal operation of HBRSEP through the unauthorized use of or tampering with its machinery, components, or controls including the security system.	– Security Event (AD-SY-ALL-0150)
<b>ENTRY OF UNAUTHORIZED PERSON INTO PROTECTED OR VITAL AREA</b> 10 CFR 73, Appendix G, I(b)	Unauthorized Entry Security Safeguards	An actual entry of an unauthorized person into a protected area, material access area, controlled access area, vital area, or transport.	– Security Event (AD-SY-ALL-0150)
<b>FAILURE, DEGRADATION, OR DISCOVERED VULNERABILITY OF SAFEGUARD SYSTEM</b>  10 CFR 73, Appendix G, I(c)	Degradation Vulnerability Safeguards Unauthorized Undetected Access Security	Any failure, degradation, or the discovered vulnerability in a safeguard system that could allow unauthorized or undetected access to a protected area, material access area, controlled access area, vital area or transport for which compensatory measures have not been employed.	– Procedure AD-SY-ALL-0150
<b>INTRODUCTION OF CONTRABAND INTO VITAL OR PROTECTED AREA</b> 10 CFR 73, Appendix G, I(d)	Contraband Unauthorized Security Safeguards	The actual or attempted introduction of contraband into a protected area, material process area, vital area, or transport.	– Contraband applies to items that could be used to commit radiological sabotage as defined in 10 CFR 73.2.

\* See footnote on the previous page regarding a goal for a 15 minute call to the NRC in regard to an imminent security threat or attack.



ATTACHMENT 11.1  
Page 5 of 9  
**IMMEDIATE (ONE HOUR) NOTIFICATIONS TO THE NRC**

IMMEDIATE (ONE HOUR) NOTIFICATIONS TO THE NRC - SOURCE, BYPRODUCT AND SNM			
HBRSEP shall immediately notify the NRC Operations Center via ETS, when:			
EVENT	KEY WORDS	REQUIREMENT	EXAMPLES
EXTERNAL EXPOSURE FROM BYPRODUCT, SOURCE, OR SNM (5X ANNUAL LIMIT)	Byproduct Source SNM Exposure Dose Release Occupational	Notwithstanding any other requirements for notification, immediately notify the NRC of any event involving byproduct, source, or SNM possessed by HBRSEP that may have caused or threatens to cause any of the following conditions: 1. An individual to receive: (i) A total effective dose equivalent of 25 rems or more; or (ii) An eye dose equivalent of 75 rems or more; or (iii) A shallow dose equivalent to the skin or extremities of 250 rads or more; or 2. The release of radioactive material, inside or outside the restricted area, so that, had an individual been present for 24 hours, the individual could have received an intake five times the occupational annual limit on intake.	
10 CFR 20.2202(a)(1)			

## ATTACHMENT 11.1

Page 6 of 9

**IMMEDIATE (ONE HOUR) NOTIFICATIONS TO THE NRC**

<b>IMMEDIATE (ONE HOUR) NOTIFICATIONS TO THE NRC - SOURCE, BYPRODUCT AND SNM</b>			
HBRSEP shall immediately notify the NRC Operations Center via ETS, when:			
<b>EVENT</b>	<b>KEY WORDS</b>	<b>REQUIREMENT</b>	<b>EXAMPLES</b>
<b>INTERNAL EXPOSURE FROM BYPRODUCT, SOURCE, SNM (&gt;5X OCCUPATIONAL LIMIT)</b>  10 CFR 20.2202(a)(2)	Intake Ingestion Release Source Byproduct SNM	The release of radioactive material, inside or outside the restricted area, so that, had an individual been present for 24 hours, the individual could have received an intake five times the occupational annual limit on intake.	
<b>IMMEDIATE (ONE HOUR) NOTIFICATIONS TO THE NRC - ISFSI</b>			
HBRSEP shall immediately notify the NRC Operations Center via ETS, when:			
<b>EVENT</b>	<b>KEY WORDS</b>	<b>REQUIREMENT</b>	<b>EXAMPLES</b>
<b>ISFSI - ACCIDENTAL CRITICALITY OR LOSS OF SNM</b>  10 CFR 72.74	ISFSI Criticality SNM Loss	The licensee shall notify the NRC Operations Center via ETS within one hour of discovery of accidental criticality or any loss of SNM.	– Unusually high radiation readings discovered in the vicinity of the ISFSI that could indicate possibility of a criticality event
<b>IMMEDIATE (ONE HOUR) NOTIFICATIONS TO THE NRC - SNM SHIPMENTS</b>			
HBRSEP shall immediately notify the NRC Operations Center via ETS, when:			
<b>EVENT</b>	<b>KEY WORDS</b>	<b>REQUIREMENT</b>	<b>EXAMPLES</b>
<b>LOST OR UNACCOUNTED SHIPMENT OF SNM</b>  10 CFR 70.52(b) 10 CFR 73.71(a)(1)	Shipment Loss SNM Spent Fuel Diversion Safeguards Security	HBRSEP shall notify the NRC Operations Center via the ETS within one hour after discovery of any loss of any shipment of SNM or spent fuel or any incident in which an attempt has been made, or is believed to have been made, to commit a theft or unlawful diversion of SNM.	– Shipment Emergency Event – Security Event (AD-SY-ALL-0150)
<b>LOST OR UNACCOUNTED SHIPMENT OF SNM - RECOVERY</b>  10 CFR 73.71(a)(1)	Recovery Accounting Shipment SNM Security Safeguards	HBRSEP shall notify the NRC Operations Center via the ETS within one hour after recovery of, or accounting for, any lost shipment of SNM.	

ATTACHMENT 11.1  
Page 7 of 9  
**IMMEDIATE (ONE HOUR) NOTIFICATIONS TO THE NRC**

<b>IMMEDIATE (ONE HOUR) NOTIFICATIONS TO THE NRC - FOLLOW-UP</b>			
With respect to the telephone notifications made under paragraphs (a) and (b) of 10 CFR 50.72 or paragraphs (a), (b), (c), or (d) of 10 CFR 72.75, in addition to making the required initial notification, HBRSEP shall during the course of the event immediately report:			
<b>EVENT</b>	<b>KEY WORDS</b>	<b>REQUIREMENT</b>	<b>EXAMPLES</b>
<b>FOLLOW-UP NOTIFICATION</b>  10 CFR 50.72(c)(1) 10 CFR 72.75(f)(1)	Degradation Emergency Class Change Update Termination ISFSI	(i) any further degradation in the level of safety of the plant or ISFSI or other worsening conditions, including those that require the declaration of any of the Emergency Classes, if such a declaration has not been previously made, or (ii) any change from one Emergency Class to another, or (iii) a termination of the Emergency Class.	– Refer to EPNOT-01
<b>FOLLOW-UP NOTIFICATION</b>  10 CFR 50.72(c)(2) 10 CFR 72.75(f)(2)	Result Evaluation Effectiveness Unknown ISFSI	(i) the results of ensuing evaluations or assessments of plant or ISFSI conditions, (ii) the effectiveness of response or protective measures taken, and (iii) information related to plant or ISFSI behavior that is not understood.	
<b>FOLLOW-UP NOTIFICATION</b>  10 CFR 50.72(c)(3) 10 CFR 50.72.75(f)(3)	Open Continuous Communication ISFSI	Maintain an open, continuous communication channel with the NRC Operations Center upon request by the NRC.	– Refer to EPNOT-01

ATTACHMENT 11.1  
Page 8 of 9  
**IMMEDIATE (ONE HOUR) NOTIFICATIONS TO THE NRC**

<b>IMMEDIATE (ONE HOUR) NOTIFICATIONS - NRC REGION II OFFICE</b>			
HBRSEP shall immediately notify the final delivery carrier and, by telephone and telegram, mailgram, or facsimile, the NRC Region II Office when:			
<b>EVENT</b>	<b>KEY WORDS</b>	<b>REQUIREMENT</b>	<b>EXAMPLES</b>
<b>THEFT/UNLAWFUL DIVERSION OF TRITIUM</b>  10 CFR 30.55(c)	Incident Theft Tritium Attempt Security Safeguards	Any incident in which an attempt has been made or is believed to have been made to commit a theft of more than 10 curies of tritium (outside of spent fuel) at any one time or more than 100 curies of tritium in one calendar year	– 10 Curies of tritium discovered missing from the Chemistry Laboratory, and reason exists to suspect that the tritium was stolen
<b>THEFT/UNLAWFUL DIVERSION OF SOURCE MATERIAL</b>  10 CFR 40.64(c)	Incident Attempt Theft Diversion Source Security Safeguards	Any incident in which an attempt has been made or is believed to have been made to commit a theft or unlawful diversion of more than 15 pounds of Source Material at any one time or 150 pounds of Source Material in any one calendar year	– A source assembly is discovered missing from a new fuel shipment.
<b>SHIPPING PACKAGE RADIOACTIVELY CONTAMINATED</b>  10 CFR 20.1906(d)(1)	Contamination Shipment	Removable radioactive surface contamination exceeds the limits of 10 CFR 71.87	– New or Spent Fuel Shipment Cask arrives with surface contamination in excess of limits.
<b>SHIPPING PACKAGE EXCEEDING EXTERNAL DOSE RATE LIMITS</b>  10 CFR 20.1906(d)(2)	Radiation Dose Rate Shipment	External radiation levels exceeds of the limits of 10 CFR 71.47	– New or Spent Fuel Shipment Cask arrives with external radiation levels in excess of limits.

ATTACHMENT 11.1  
Page 9 of 9  
**IMMEDIATE (ONE HOUR) NOTIFICATIONS TO THE NRC**

<b>IMMEDIATE (ONE HOUR) NOTIFICATIONS TO THE NRC - FFD</b>			
The NRC Region II Administrator must be notified immediately by telephone of the following:			
<b>EVENT</b>	<b>KEY WORDS</b>	<b>REQUIREMENT</b>	<b>EXAMPLES</b>
<b>NRC EMPLOYEE NOT FIT FOR DUTY</b>  10 CFR 26.77(c)	Alcohol Influence Substance NRC employee FFD Fitness for Duty	If HBRSEP has a reasonable belief that an NRC employee or NRC contractor may be under the influence of any substance, or is otherwise unfit for duty, the licensee or other entity may not deny access but shall escort the individual. In any such instance, the licensee or other entity shall immediately notify the Region II Administrator by telephone, followed by written notification (e.g., e-mail or fax) to document the oral notification. If the Region II Administrator cannot be reached, the licensee or other entity shall notify the NRC Operations Center.	
<b>IMMEDIATE (ONE HOUR) NOTIFICATIONS TO THE NRC - IAEA</b>			
The NRC Director, NRR or Director, NMSS must be notified immediately by telephone of the following:			
<b>SURPRISE VISIT OF IAEA OFFICIAL</b>  10 CFR 75.8(c)	IAEA International Atomic Energy Agency Credential	HBRSEP shall immediately communicate by telephone, within one hour with respect to the credentials of any person who claims to be an IAEA representative and shall accept written or electronic confirmation of the credentials from the NRC.	– If the IAEA representative's credentials have not been confirmed by the NRC, the licensee shall not admit the person until the NRC has confirmed the person's credentials. The licensee, shall notify the Commission promptly, by telephone, whenever an IAEA representative arrives at a facility or location without advance notification.

ATTACHMENT 11.2  
Page 1 of 3  
**FOUR HOUR NOTIFICATIONS TO THE NRC**

<b>FOUR HOUR NOTIFICATIONS TO THE NRC</b>			
If not reported under paragraphs (a) or (b)(1) of 10 CFR 50.72, HBRSEP shall notify the NRC Operations Center via ETS as soon as practical and in all cases, within four hours of the occurrence of any of the following:			
<b>EVENT</b>	<b>KEY WORDS</b>	<b>REQUIREMENT</b>	<b>EXAMPLES</b>
<b>SHUTDOWN REQUIRED BY TS</b>            10 CFR 50.72(b)(2)(i)	Shutdown TS Shutdown Power Reduction	The <u>initiation</u> of any shutdown required by the TS.	<ul style="list-style-type: none"> <li>- Reactor is in MODEs 1 or 2 and the Control Room takes action to reduce power (i.e., negative reactivity insertion) in order to comply with a Required Action to be in MODE 3 within a Completion Time. Reduction in power for some other purpose than compliance with the shutdown requirement is not reportable. MODE changes required by TS when reactor is in MODEs 3, 4, or other non-power conditions, are not reportable.</li> <li>- If allowed outage time plus required shutdown time to MODE 3 is less than the expected restoration time of the LCO and power is reduced in anticipation of the required shutdown, the shutdown is reportable.</li> </ul>
<b>ECCS DISCHARGE INTO RCS</b>            10 CFR 50.72(b)(2)(iv)(A)	ECCS Actuation Safety Injection	Any event that results or should have resulted in emergency core cooling system (ECCS) discharge into the reactor coolant system as a result of a valid signal except when the actuation results from and is part of a pre-planned sequence during testing or reactor operation.	<ul style="list-style-type: none"> <li>- Manual or automatic Safety Injection System actuation in response to a valid signal that resulted in or should have resulted in discharge into the reactor coolant system.</li> </ul>
<b>RPS INITIATION (MANUAL/AUTOMATIC) DURING OPERATION</b>            10 CFR 50.72(b)(2)(iv)(B)	RPS Actuation Reactor Protection System RPS Reactor Trip	Any event or condition that results in actuation of the reactor protection system (RPS) when the reactor is critical except when the actuation results from and is part of a pre-planned sequence during testing or reactor operation.	<ul style="list-style-type: none"> <li>- Manual or automatic reactor trip from critical through RTP of 100%. Trips which occur as part of planned evolutions in accordance with procedures are not reportable.</li> </ul>

ATTACHMENT 11.2  
Page 2 of 3  
**FOUR HOUR NOTIFICATIONS TO THE NRC**

FOUR HOUR NOTIFICATIONS TO THE NRC			
If not reported under paragraphs (a) or (b)(1) of 10 CFR 50.72, HBRSEP shall notify the NRC Operations Center via ETS as soon as practical and in all cases, within four hours of the occurrence of any of the following:			
EVENT	KEY WORDS	REQUIREMENT	EXAMPLES
<b>PRESS RELEASES AND GOVERNMENT NOTIFICATIONS</b>           10 CFR 50.72(b)(2)(xi) 10 CFR 72.75(b)(2)	News Release Press Radio Television Fatality Environment Public Health and Safety Release ISFSI	Any event or situation, related to the health and safety of the public or on-site personnel, or protection of the environment, for which a news release is planned or notification to other government agencies has been or will be made. Such an event may include an on-site fatality or inadvertent release of radioactively contaminated materials.  Licensees are required to notify the NRC within 4 hours of whichever of the following occurs first:  – A plan to report to either the press or another government agency is approved by an individual authorized to make the final decision, or  – A report has actually been made to the press or another government agency.	– Any News release concerning <ul style="list-style-type: none"> <li>- A fatality,</li> <li>- Inadvertent release of radioactively contaminated materials to public areas</li> <li>- unusual or abnormal releases of radioactive effluents (See Attachment 11.14), or</li> <li>- Information associated with an Emergency Event except when the ERO is activated (EPNOT-01).</li> </ul> – Notification to other government agencies concerning: <ul style="list-style-type: none"> <li>- A fatality on site,</li> <li>- Health and safety of the public or site personnel,</li> <li>- Inadvertent release of radioactively contaminated materials to public areas,</li> <li>- Discovered endangered species kill.</li> <li>- Notifications to the National Response Center (EPA) related to Lake Robinson</li> </ul>

ATTACHMENT 11.2  
Page 3 of 3  
**FOUR HOUR NOTIFICATIONS TO THE NRC**

FOUR HOUR NOTIFICATIONS TO THE NRC			
HBRSEP shall notify the NRC Operations Center via ETS as soon as possible but not later than 4 hours after the discovery of any of the following events or conditions involving sources or spent fuel.			
EVENT	KEY WORDS	REQUIREMENT	EXAMPLES
<b>LOSS OR THEFT OF LICENSED MATERIAL (&gt;1000X 10 CFR 20 LIMITS)</b>  10 CFR 20.2201	Loss Theft Missing Licensed Radioactive Material Recovery	Immediately notify the NRC, after its occurrence becomes known, any lost, stolen, or missing licensed material in an aggregate quantity equal to or greater than 1,000 times the quantity specified in [10 CFR 20] Appendix C under such circumstances that it appears to HBRSEP that an exposure could result to persons in unrestricted areas. Follow-up written report required within subsequent 30 days.  Note – If the lost, stolen, or missing source exceeds a “Quantity of Concern” as specified in HPP-018, then the NRC desires to be notified within 4 hours of any subsequent recovery of the source.	<ul style="list-style-type: none"> <li>– A radiography source is discovered missing. The source is licensed to the radiography contractor. If the contractor does not make the required notification, HBRSEP should notify the NRC Operations Center via ETS.</li> </ul>
<b>ISFSI - DEPARTURE FROM LICENSE CONDITION</b>  10 CFR 72.75(b)(1)	ISFSI Emergency Departure Deviation Health and Safety License Condition	An action taken in an emergency that departs from a condition or a technical specification contained in a license or certificate of compliance issued under 10 CFR 72 when the action is immediately needed to protect the public health and safety and no action consistent with license conditions or technical specifications that can provide adequate or equivalent protection is immediately apparent.	<ul style="list-style-type: none"> <li>– Action taken in an emergency that departs from procedure that is deemed necessary to prevent releases or radiation doses to the public in excess of 10 CFR 20 limits (See AD-HU-ALL-0004).</li> </ul>



## ***Examination KEY for: ILC15 RNP SRO NRC Examin***

<b><i>Question Number</i></b>	<b><i>Answer</i></b>
1	B
2	D
3	C
4	C
5	B
6	B
7	B
8	B
9	D
10	C
11	A
12	D
13	D
14	B
15	A
16	A
17	B
18	B
19	C
20	A
21	B
22	D
23	D
24	D
25	C

## ***Examination KEY for: ILC15 RNP SRO NRC Examin***

<b><i>Question Number</i></b>	<b><i>Answer</i></b>
26	A
27	D
28	C
29	B
30	C
31	B
32	B
33	C
34	B
35	B
36	A
37	D
38	D
39	C
40	C
41	B
42	C
43	C
44	C
45	C
46	C
47	B
48	A
49	A
50	D

## ***Examination KEY for: ILC15 RNP SRO NRC Examin***

<b><i>Question Number</i></b>	<b><i>Answer</i></b>
51	D
52	C
53	D
54	D
55	C
56	C
57	B
58	B
59	C
60	D
61	A
62	B
63	A
64	B
65	C
66	C
67	D
68	B
69	A
70	A
71	B
72	B
73	A
74	D
75	B

## ***Examination KEY for: ILC15 RNP SRO NRC Examin***

<b><i>Question Number</i></b>	<b><i>Answer</i></b>
76	A
77	C
78	D
79	B
80	B
81	A
82	C
83	D
84	C
85	A
86	D
87	A
88	C
89	A
90	D
91	C
92	B
93	C
94	A
95	D
96	B
97	D
98	B
99	B
100	B