

50-269/270/287

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

FILE NUMBER

| | | | | |
|---|--|--|--|---|
| TO: Mr Rusche | | FROM: Duke Power Company Charlotte, NC W O Parker Jr | | DATE OF DOCUMENT 8-23-76 |
| <input checked="" type="checkbox"/> LETTER <input checked="" type="checkbox"/> ORIGINAL <input type="checkbox"/> COPY | | <input type="checkbox"/> NOTORIZED <input checked="" type="checkbox"/> UNCLASSIFIED | | DATE RECEIVED 8-26-76 |
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DESCRIPTION

Ltr furnishing info concerning limiting conditions for operation with regard to the electrical system.....& advising of related incident which occurred on 8-18-76
.....

ENCLOSURE

Do Not Remove

ACKNOWLEDGED

PLANT NAME: Oconee 1-3

SAFETY

FOR ACTION/INFORMATION

ENVIRO

8-30-76

ehf

| | | |
|------------------|--------------------|------------------|
| ASSIGNED AD: | | ASSIGNED AD: |
| BRANCH CHIEF: | Schwencer (5) | BRANCH CHIEF: |
| PROJECT MANAGER: | 2 cc's Sheppard | PROJECT MANAGER: |
| LIC. ASST.: | | LIC. ASST.: |

INTERNAL DISTRIBUTION

| | | | |
|---|----------------|--------------------|-----------------|
| <input checked="" type="checkbox"/> REG FILE | SYSTEMS SAFETY | PLANT SYSTEMS | SITE SAFETY & |
| <input checked="" type="checkbox"/> NRC PDR | HEINEMAN | TEDESCO | ENVIRO ANALYSIS |
| <input checked="" type="checkbox"/> I & E (2) | SCHROEDER | BENAROYA | DENTON & MULLER |
| <input checked="" type="checkbox"/> OELD | | LAINAS | |
| <input checked="" type="checkbox"/> GOSSICK & STAFF | ENGINEERING | IPPOLITO | ENVIRO TECH. |
| MIPC | MACCARRY | KIRKWOOD | ERNST |
| CASE | KNIGHT | | BALLARD |
| HANAUER | SIHWEIL | OPERATING REACTORS | SPANGLER |
| HARLESS | PAWLICKI | STELLO | |
| | | | SITE TECH. |
| PROJECT MANAGEMENT | REACTOR SAFETY | OPERATING TECH. | GAMMILL |
| BOYD | ROSS | EISENHUT | STEPP |
| P. COLLINS | NOVAK | SHAO | HULMAN |
| HOUSTON | ROSZTOCZY | BAER | |
| PETERSON | CHECK | BUTLER | SITE ANALYSIS |
| MELTZ | | GRINES | VOLLMER |
| HELTEMES | AT & I | | BUNCH |
| SKOVHOLT | SALTZMAN | | J. COLLINS |
| | RUTBERG | | KREGER |

EXTERNAL DISTRIBUTION

CONTROL NUMBER

| | | | |
|--|-------------|--------------------|------|
| <input checked="" type="checkbox"/> LPDR: <i>Nathalie, SC</i> | NAT LAB: | BROOKHAVEN NAT LAB | 8709 |
| <input checked="" type="checkbox"/> TIC: | REG. VIE | ULRIKSON (ORNL) | |
| <input checked="" type="checkbox"/> NSIC: | LA PDR | | |
| <input checked="" type="checkbox"/> ASLB: | CONSULTANTS | | |
| <input checked="" type="checkbox"/> ACRS/6 CYS HOLDING/SENT To LA Sheppard | | | |

DUKE POWER COMPANY

POWER BUILDING

422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28242

WILLIAM O. PARKER, JR.
VICE PRESIDENT
STEAM PRODUCTION

TELEPHONE: AREA 704
373-4083

August 23, 1976



Mr. Benard C. Rusche
Director of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Re: Oconee Nuclear Station
Docket Nos. 50-269, -270, -287

Dear Mr. Rusche:

Regulatory Docket File

The Oconee Nuclear Station Technical Specifications 3.7, "Auxiliary Electrical Systems," provides Limiting Conditions for Operation relative to the Oconee Electrical System. Provisions are made for operation in degraded modes for specified periods of time to perform maintenance or test or if equipment is not operable. Further, any degradation beyond the detailed conditions of the specification are required, by Specification 3.7.8, to be reported to the NRC Office of Inspection and Enforcement within 24 hours and a safety evaluation for the specific situation is to be performed by Duke Power Company. The results of this evaluation together with the plans for expediting the return to unrestricted operation shall be submitted to the NRC, Office of Nuclear Reactor Regulation within five days. The purpose of this letter is to describe a situation which has occurred at Oconee Nuclear Station and to provide the required written report.

On August 18, 1976, at approximately 1300 hours, it was determined that the Oconee Unit 3 125 volt DC instrumentation and control power system batteries (3CA and 3CB) had not been tested in accordance with Technical Specification 4.6.7.c within the specified surveillance time period. This is a one-hour discharge test at the required maximum safeguards load. Since these batteries had not been tested, they were considered inoperable based upon the definition "operable" in Specification 1.3.

An evaluation of this situation was performed. It was determined that ten battery discharge tests have been performed at Oconee (four I&C battery tests for Oconee 1, two I&C battery tests for Oconee 2, two switchyard battery tests and two Keowee Hydro Station battery tests) with all batteries performing satisfactorily. The voltage and temperature

Mr. Benard C. Rusche

Page 2

August 23, 1976

of pilot cells have been measured five times per week and the specific gravity and voltage of each cell have been measured monthly with no indications of battery degradation. Annunciators provided in the control room give assurance of continued proper operation of the battery chargers. From the above tests and indications and a review of the service life and capacity of these batteries, it was concluded that the Oconee 3 instrument and control batteries should be capable of performing the one-hour test discharge at the required maximum safeguards load.

In view of the above discussion concerning the operability of the Oconee 3 I&C batteries and the fact that redundancy is provided to the Oconee 3 125 VDC I&C system from the Oconee 1 I&C batteries, it was considered that maintaining Oconee 3 at power was the safest course of action based upon the short period of time necessary to perform the test discharge and the minimization of maneuvering transients. Oconee 3 battery 3CB was removed from service and the discharge test was completed by 1850 hours on August 18, 1976 with the battery restored to operable status by 1630, August 20, 1976. The 3CA battery was then removed from service and the discharge test was completed at 1925 August 20, 1976. Battery 3CA was returned to operable status on August 23, 1976 following the battery charge. It is considered that the limiting conditions for operation specified in Technical Specification 3.7.1 are now being met.

Very truly yours,

W. O. Parker, Jr.

William O. Parker, Jr. *By [Signature]*

MST:vr

cc: Mr. Norman C. Moseley