

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)
DISTRIBUTION FOR INCOMING MATERIAL

50-287

REC: CASE E G
NRC

ORG: PARKER W O
DUKE PWR

DOCDATE: 02/08/78
DATE RCVD: 02/13/78

DOCTYPE: LETTER NOTARIZED: YES
SUBJECT:

COPIES RECEIVED
LTR 3 ENCL 40

LICENSE NO DPR-47 APPL FOR AMEND: TECH PROPOSED CHANGE CONCERNING
TEMPORARY EMERGENCY EXTENSION OF ALLOWABLE TIME FOR MAINTANCE TO
ALLOW CONTINUED UNIT OPERATION AT FULL PWR FOR AN ADDL 48 HOURS TO
REPAIR VALVE 3BS2...NOTORIZED 02/08/78.

PLANT NAME: OCONEE - UNIT 3

REVIEWER INITIAL: XJM
DISTRIBUTOR INITIAL:

***** DISTRIBUTION OF THIS MATERIAL IS AS FOLLOWS *****

NOTES:

1. M. CUNNINGHAM - ALL AMENDMENTS TO FSAR AND CHANGES TO TECH SPECS

GENERAL DISTRIBUTION FOR AFTER ISSUANCE OF OPERATING LICENSE.
(DISTRIBUTION CODE A001)

FOR ACTION: ~~BRANCH GETEF~~ SCHWENCER**W/7 ENC

INTERNAL: ~~REG FILE**W/ENCL~~
I & E**W/2 ENCL
HANAUER**W/ENCL
EISENHUT**W/ENCL
BAER**W/ENCL
GRIMES**W/ENCL
J. MCGOUGH**W/ENCL

NRC PDR**W/ENCL
OELD**LTR ONLY
CHECK**W/ENCL
SHAO**W/ENCL
BUTLER**W/ENCL
J COLLINS**W/ENCL

EXTERNAL: LPDR/S
WALHALLA, SC**W/ENCL
TIC**W/ENCL
NSIC**W/ENCL
ACRS CAT B**W/16 ENCL

DISTRIBUTION: LTR 40 ENCL 39
SIZE: 2P+1P

CONTROL NBR: 780440045

THE END

Handwritten: KJ
004

DUKE POWER COMPANY

POWER BUILDING

422 SOUTH CHURCH STREET, CHARLOTTE, N.C. 28202

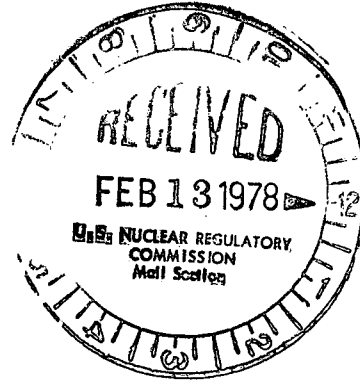
REGULATORY DOCKET FILE COPY

WILLIAM O. PARKER, JR.
VICE PRESIDENT
STEAM PRODUCTION

TELEPHONE: AREA 704
373-4083

February 8, 1978

Mr. Edson G. Case, Acting Director
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555



Re: Oconee Unit 3
Docket No. 50-287

Dear Mr. Case:

Technical Specification 3.3.4 requires two trains of Reactor Building spray to be operable. Valve 3BS2 failed a surveillance test in that it failed to operate. This made one train of Reactor Building spray inoperable. At the end of the allowed 24 hour maintenance period, the valve was not repaired. It is expected to be repaired by 6 p.m., February 9, 1978. An emergency temporary extension of the allowable time for maintenance is requested to allow continued unit operation at full power for an additional 48 hours to repair valve 3BS2.

This extension is considered reasonable in that the redundant Reactor Building spray train was verified operable and all containment cooling systems are operable. The unit is considered to be protected under all accident conditions. By Standard Technical Specifications for Babcock and Wilcox Pressurized Water Reactors, one Reactor Building spray system train is authorized to be inoperable for periods up to 72 hours. Additionally, an analysis was performed and submitted as Supplement 13 to the FSAR which states in part that "... without Reactor Building spray and only two coolers operable, a maximum building pressure of 53.8 psig was obtained." This maximum pressure is below the design pressure of the Containment Building.

The Nuclear Safety Review Board reviewed this proposal and approved it based on review of the FSAR, Standard Technical Specifications, and the probability of an accident occurring during the extension period requested is very small.

Very truly yours,

William O. Parker, Jr.

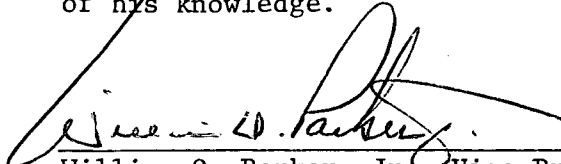
RLG:vr

780440045

A001/s
3/40

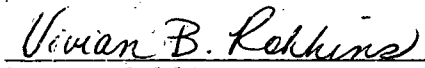
Mr. Edson G. Case
Page 2
February 8, 1978

WILLIAM O. PARKER, JR., being duly sworn, states that he is Vice President of Duke Power Company; that he is authorized on the part of said Company to sign and file with the Nuclear Regulatory Commission this request for amendment of the Oconee Nuclear Station Technical Specifications, Appendix A to Facility Operating Licenses DPR-38, DPR-47 and DPR-55; and that all statements and matters set forth therein are true and correct to the best of his knowledge.



William O. Parker, Jr., Vice President

Subscribed and sworn to before me this 8th day of February, 1978.



Notary Public

My Commission Expires:

February 15, 1982

- 3.3.2 In addition to 3.3.1 above, the following ECCS equipment shall be operable when the reactor coolant system is above 350°F and irradiated fuel is in the core:
- (a) Two high pressure injection pumps shall be maintained operable to provide redundant and independent flow paths.
 - (b) Engineered Safety Feature valves and interlocks associated with 3.3.2a above shall be operable.
- 3.3.3 In addition to 3.3.1 and 3.3.2 above, the following ECCS equipment shall be operable when the reactor coolant system is above 800 psig:
- (a) The two core flooding tanks shall each contain a minimum of $13 \pm .44$ ft. (1040 ± 30 ft³) of borated water at 600 ± 25 psig.
 - (b) Core flooding tank boron concentration shall not be less than 1,800 ppm boron.
 - (c) The electrically-operated discharge valves from the core flood tanks shall be open and breakers locked open and tagged.
 - (d) One pressure instrument channel and one level instrument channel per core flood tank shall be operable.
- 3.3.4 The reactor shall not be made critical unless the following equipment in addition to 3.3.1, 3.3.2, and 3.3.3 is operable.
- (a) The other reactor building spray pump and its associated spray nozzle header.
 - (b) The remaining reactor building cooling fan and associated cooling unit.
 - (c) Engineered Safety Feature valves and interlocks associated with 3.3.4a and 3.3.4b shall be operable.
- 3.3.5 Except as noted in 3.3.6 below, tests or maintenance shall be allowed during power operation on any component(s) in the high pressure injection, low pressure injection, low pressure service water, reactor building spray, reactor building cooling which will not remove more than one train of each system from service. Components shall not be removed from service so that the affected system train is inoperable for more than 24*consecutive hours. If the system is not restored to meet the requirements of Specification 3.3.1, 3.3.2, 3.3.3, or 3.3.4, within 24 hours, the reactor shall be placed in a hot shutdown condition within 12 hours. If the requirements of Specification 3.3.1, 3.3.2, 3.3.3, or 3.3.4 are not met within an additional 48 hours, the reactor shall be placed in a condition below that reactor coolant system condition required in Specification 3.3.1, 3.3.2, 3.3.3, or 3.3.4 for the component degraded.

*In the case of Reactor Building spray valve 3BS-2, 72 consecutive hours are allowed to return the valve to operable status on a one-time basis only; this 72 hour allowance shall expire on February 10, 1978.

RECEIVED DOCUMENT
CONTROL DESK

1978 FEB 15 AM 9 11

U.S. NRC
DISTRIBUTION SERVICES
BRANCH