

**NRC CONTRIBUTION FOR PART 50 DOCKET MATERIAL
(TEMPORARY FORM)**

CONTROL NO: 7359

FILE: INCIDENT REPORT FILE

FROM: Duke Power Co. Charlotte, N.C. 28242 Wm. O. Parker, Jr.			DATE OF DOC 7-1-75	DATE REC'D 7-11-75	LTR XX	TWX	RPT	OTHER
TO: Mr. N. C. Moseley			ORIG 1 signed	CC	OTHER	SENT AEC PDR <u>XX</u> SENT LOCAL PDR <u>XX</u>		
CLASS	UNCLASS XXX	PROP INFO	INPUT	NO CYS REC'D 1		DOCKET NO: 50-287		

DESCRIPTION: Ltr trans the following:

ENCLOSURES: Unusual Event-287/75-7 on 5-23-75 re failure of valve 3FDW-108 to operate
(1 Orig cy encl rec'd)**Do Not Remove****ACKNOWLEDGED**

PLANT NAME: Oconee Unit 3

FOR ACTION/INFORMATION

DHL 7-11-75

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1 - ASLB	NEWMARK/BLUME/AGBABIAN	1 - AGMED (RUTH GUSSMAN) Rm B-127 GT
1 - Newton Anderson		1 - J. D. RUNKLES, Rm E-201 GT
1 - ACRS SENT TO LIC ASST Sheppard 7-11-75		
** SEND ONLY TEN DAY REPORTS		

DUKE POWER COMPANY

Regulatory Docket File

POWER BUILDING

422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28242

WILLIAM O. PARKER, JR.
VICE PRESIDENT
STEAM PRODUCTION

TELEPHONE: AREA 704
373-4083

July 1, 1975

Mr. Norman C. Moseley, Director
U. S. Nuclear Regulatory Commission
Suite 818
230 Peachtree Street, Northwest
Atlanta, Georgia 30303

Re: Oconee Unit 3
Docket No. 50-287

Dear Mr. Moseley:

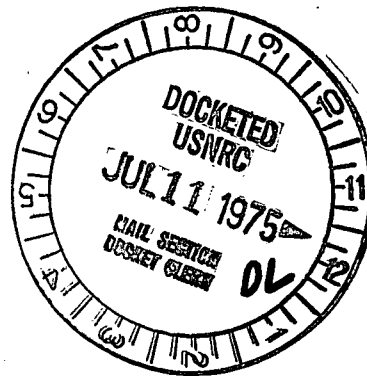
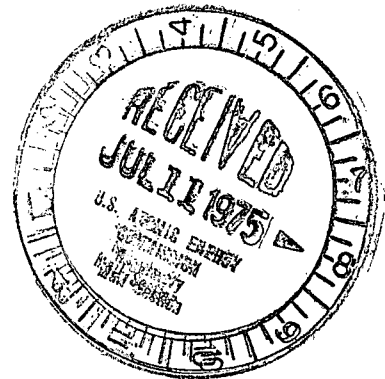
Pursuant to Sections 6.2 and 6.6.2 of the Oconee Nuclear Station
Technical Specifications, please find attached Unusual Event
Report UE-287/75-7.

Very truly yours,

William O. Parker, Jr.
William O. Parker, Jr. *by P.H.B.*

MST:vr
Attachment

cc: Mr. Angelo Giambusso



7359

DUKE POWER COMPANY
OCONEE UNIT 3Report No.: UE-287/75-7

Received w/ Ltr Dated 7-1-75

Report Date: July 1, 1975Event Date: May 23, 1975Facility: Oconee Unit 3, Seneca, South CarolinaIdentification of Event: Failure of valve 3FDW-108 to operateConditions Prior to Event: Unit at full powerDescription of Event:

On May 23, 1975, the quarterly performance test of Engineered Safeguards valves was conducted on Oconee Unit 3. The B steam generator sample line isolation valve, 3FDW-108, failed to indicate closed after being given the close command. This valve is normally closed during operation and is opened only for steam generator sampling. The valve has been closed and locked until corrective maintenance can be performed.

Designation of Apparent Cause of Event:

The apparent cause of this event was a weak spring in the valve operator. The valve was found to be remotely operable with no pressure in the sample line; however, due to a weak spring in the operator, it would not close under pressure.

Analysis of Event:

Valve 3FDW-108 is the Reactor Building containment isolation valve for the B steam generator sample line. This valve is one of two redundant valves which are closed by an Engineered Safeguards signal in the unlikely event containment integrity is required. This incident did not affect the redundant isolation valve. It is concluded that the health and safety of the public was not affected by this incident.

Corrective Action:

A new spring has been ordered for the operator of valve 3FDW-108. Until the valve operator can be repaired, 3FDW-108 has been locked closed with the redundant isolation valve, 3FDW-107, also closed.

Failure Data:

A previous failure was experienced with valve 3FDW-108 and was discussed in report UE-287/74-4. At that time, no cause was found for the valve's failure to close in a specified time. After further testing, the valve was returned to service. The apparent cause of both failures has now been identified and will be corrected.

[Faint, mostly illegible text covering the majority of the page, appearing to be a memorandum or report.]

U.S.A.E.S
REGULATORY OPERATIONS
REGION II
ATLANTA, GA.
JUL 7 10 10 AM '75