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CONTROL NO: 13079

FILE: INCIDENT REPORT FILE

FROM: Duke Power Co. Charlotte, N.C. Willima O. Parker		DATE OF DOC 11-11-75	DATE REC'D 11-15-75	LTR XXX	TWX	RPT	OTHER
TO: Mr. Norman Moseley		ORIG 1 Signed	CC 0	OTHER	SENT AEC PDR XXX		XXX
				SENT LOCAL PDR XXX			
CLASS	UNCLASS XXX	PROP INFO	INPUT	NO CYS REC'D 1		DOCKET NO: 50-287	

DESCRIPTION:

Letter trans the following.....

ENCLOSURES:

Abnormal Occurrence # 75-12, on 10-28-75,
Concerning a Containment Isolation Valve that
was discovered inoperable.....

(1 Copy Received)

PLANT NAME: Ocone # 3

FOR ACTION/INFORMATION

SAB 11-19-75

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ACKNOWLEDGED

INTERNAL DISTRIBUTION

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1 - ASLB	NEWMARK/BLUME/AGBABIAN	1 - AGMED (RUTH GUSSMAN) Rm B-127 GT
1 - Newton Anderson		1 - J. D. RUNKLES, Rm E-201 GT
5 - ACRS SENT TO LIC ASST S. Sheppard		
** SEND ONLY TEN DAY REPORTS		

DUKE POWER COMPANY

POWER BUILDING

422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28242

WILLIAM O. PARKER, JR.
VICE PRESIDENT
STEAM PRODUCTION

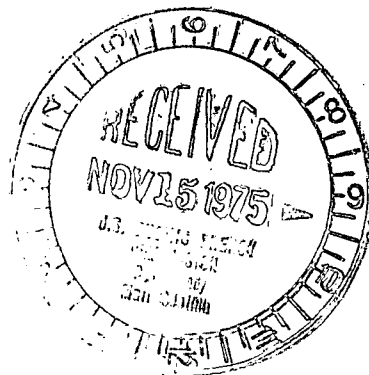
TELEPHONE: AREA 704
373-4083

November 11, 1975

Regulatory

File Cy.

Mr. Norman C. Moseley, Director
U. S. Nuclear Regulatory Commission
Suite 818
230 Peachtree Street, Northwest
Atlanta, Georgia 30303



Re: Oconee Unit 3
Docket No. 50-287

Dear Mr. Moseley:

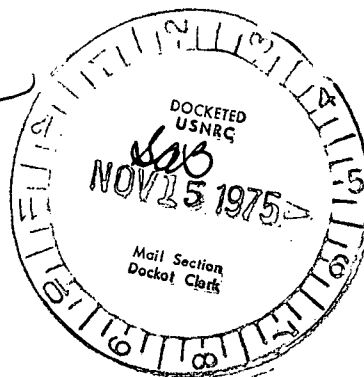
Pursuant to Sections 6.2 and 6.6.2 of the Oconee Nuclear Station
Technical Specifications, please find attached Abnormal Occurrence
Report AO-287/75-12.

Very truly yours,

W.O. Parker, Jr.
William O. Parker, Jr. *By [Signature]*

EDB/esw
Attachment

cc: Mr. Roger S. Boyd



13079

DUKE POWER COMPANY
OCONEE UNIT 3

Report No.: AO-287/75-12

Report Date: November 10, 1975

Occurrence Date: October 28, 1975

Facility: Oconee Unit 3, Seneca, South Carolina

Identification of Occurrence: Containment isolation valve discovered inoperable

Conditions Prior to Occurrence: Unit at 100% full power

Description of Occurrence:

On October 28, 1975, periodic Engineered Safeguards actuation testing was being performed on Oconee Unit 3. During this testing, attempts to close valve 3PR-7, the Reactor Building air sampling line inlet valve, failed. It was discovered that the valve's electrical breaker had been left open and, consequently, the valve was inoperable.

Designation of Apparent Cause of Occurrence:

Review of all logs and valve maintenance histories indicated that the valve had not been involved in any test or maintenance activity since periodic actuation testing on September 19, 1975. Therefore, there was no reason the electrical breaker should have been left open leaving the valve in an inoperable mode. This occurrence then, was apparently caused by a personnel error by which the valve breaker had inadvertently been left open.

Analysis of Event:

Valve 3PR-7 is a Reactor Building containment isolation valve used to isolate the Reactor Building sample line in the event containment is required. This incident resulted in one of two redundant Reactor Building valves being inoperable. The valve 3PR-8, located downstream in the sampling line was operable and would have closed if containment integrity had been necessary. Therefore, it is concluded that the health and safety of the public was not affected.

Corrective Action:

The electrical breaker was immediately closed and the valve tested to assure its operability. It is felt that present surveillance testing procedures are sufficient to discover errors of this type.

Nov 13 10 40 AM '75

U.S. AIR FORCE
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RECEIVED OPERATIONS