

H 7/20

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)
DISTRIBUTION FOR INCOMING MATERIAL

50-269/270/287

REC: CASE E G
NRC

ORG: PARKER W O
DUKE PWR

DOCDATE: 07/17/78
DATE RCVD: 07/19/78

DOCTYPE: LETTER NOTARIZED: YES

SUBJECT:

COPIES RECEIVED

PROPOSED AMENDMENT TO FACILITY OPERATING LICENSE CONSISTING OF REVISION TO
TECH SPEC 3.1.6.4. LTR 3 ENCL 40

PLANT NAME: OCONEE - UNIT 1
OCONEE - UNIT 2
OCONEE - UNIT 3

REVIEWER INITIAL: XRS
DISTRIBUTER INITIAL:

***** DISTRIBUTION OF THIS MATERIAL IS AS FOLLOWS *****

NOTES:

1. M. CUNNINGHAM - ALL AMENDMENTS TO FSAR AND CHANGES TO TECH SPECS

GENERAL DISTRIBUTION FOR AFTER ISSUANCE OF OPERATING LICENSE.
(DISTRIBUTION CODE A001)

FOR ACTION: BR CHIEF ORB#4 BC**W/7 ENCL

INTERNAL: REG FILE**W/ENCL
E**W/2 ENCL
HANAUER**W/ENCL
AD FOR SYS & PROJ**W/ENCL
REACTOR SAFETY BR**W/ENCL
EEB**W/ENCL
J. MCGOUGH**W/ENCL

NRC PDR**W/ENCL
OELD**LTR ONLY
CORE PERFORMANCE BR**W/ENCL
ENGINEERING BR**W/ENCL
PLANT SYSTEMS BR**W/ENCL
EFFLUENT TREAT SYS**W/ENCL

EXTERNAL: LPDR'S
WALHALLA, SC**W/ENCL
TERA**W/ENCL
NSIC**W/ENCL
ACRS CAT 8**W/16 ENCL

\$ CHECK NBR: 221,308 \$
\$ AMOUNT: \$4,800.00 \$
\$ CHECK AND COPY OF TRANSMITTAL LTR ADVANCED \$
\$ TO W. MILLER (LFMB) (07/20/78) UPON RECIEPT \$

XA²
R

DISTRIBUTION: LTR 40 ENCL 39
SIZE: 1P+1P+1P

CONTROL NBR: 782010166

***** THE END *****

DUKE POWER COMPANY

POWER BUILDING

422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28242

WILLIAM O. PARKER, JR.
VICE PRESIDENT
STEAM PRODUCTION

July 17, 1978

TELEPHONE: AREA 704
373-4083

Mr. Edson G. Case, Acting Director
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Attention: Mr. R. Reid, Chief
Operating Reactors Branch #4

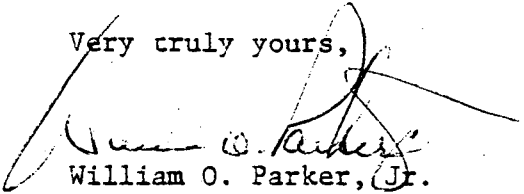
Reference: Oconee Nuclear Station
Docket Nos. 50-269, -270, -287

Dear Sir:

My letter of June 8, 1978 provided information concerning the steam generator leak rate limit for Oconee Nuclear Station, Unit 1. That submittal was considered to supplement previous submittals of June 21 and December 16, 1977 and as such no license fee was provided.

However, the staff determined that this was a new license amendment request and, in a letter dated June 28, 1978, requested a license fee of \$4,800 be provided. Therefore, pursuant to 10CFR50, §50.90 please find attached a proposed amendment to the Oconee Nuclear Station Facility Operating License which consists of a revision to Oconee Nuclear Station Technical Specification 3.1.6.4. Information in support of this request has been provided in my June 8, 1978 letter. Additionally and pursuant to 10CFR170, §170.12, please find attached a check in the amount of \$4,800 which represents one Class III and two Class I license fees required with this amendment request.

Very truly yours,


William O. Parker, Jr.

RLG:scs
Attachment

REGULATORY DOCKET FILE COPY

782010166

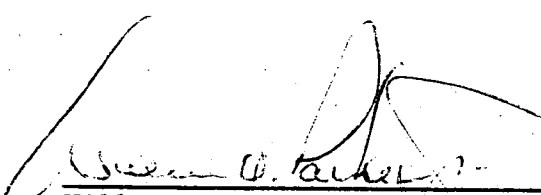
A001
3/40*

Mr. Edson G. Case

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July 17, 1978

WILLIAM O. PARKER, JR., being duly sworn, states that he is Vice President of Duke Power Company; that he is authorized on the part of said Company to sign and file with the Nuclear Regulatory Commission this request for amendment to the Oconee Nuclear Station Facility Operating License which consists of a revision to Oconee Nuclear Station Technical Specification 3.1.6.4; and that all statements and matters set forth therein are true and correct to the best of his knowledge.


William O. Parker, Jr.
William O. Parker, Jr., Vice President

Subscribed and sworn to before me this 17th day of July, 1978


Vivian B. Kehlman
Notary Public

My Commission Expires:

February 15, 1982

3.1.6 Leakage

Specification

- 3.1.6.1 If the total reactor coolant leakage rate exceeds 10 gpm, the reactor shall be shutdown within 24 hours of detection.
- 3.1.6.2 If unidentified reactor coolant leakage (excluding normal evaporative losses) exceeds 1 gpm or if any reactor coolant leakage is evaluated as unsafe, the reactor shall be shutdown within 24 hours of detection.
- 3.1.6.3 If any reactor coolant leakage exists through a non-isolable fault in a RCS strength boundary (such as the reactor vessel, piping, valve body, etc., except the steam generator tubes), the reactor shall be shutdown, and cooldown to the cold shutdown condition shall be initiated within 24 hours of detection.
- 3.1.6.4 If reactor coolant system leakage exceeds 1 gpm through the steam generator tubes, a reactor shutdown shall be initiated within 4 hours and the reactor shall be in a cold shutdown condition within the next 36 hours.
- 3.1.6.5 If reactor shutdown is required by Specification 3.1.6.1, 3.1.6.2 or 3.1.6.3, the rate of shutdown and the conditions of shutdown shall be determined by the safety evaluation for each case and justified in writing as soon thereafter as practicable.
- 3.1.6.6 Action to evaluate the safety implication of reactor coolant leakage shall be initiated within 4 hours of detection. The nature, as well as the magnitude, of the leak shall be considered in this evaluation. The safety evaluation shall assure that the exposure of offsite personnel to radiation is within the guidelines of 10CFR20.
- 3.1.6.7 If reactor shutdown is required per Specification 3.1.6.1, 3.1.6.2, 3.1.6.3 or 3.1.6.4, the reactor shall not be restarted until the leak is repaired or until the problem is otherwise corrected.
- 3.1.6.8 When the reactor is critical and above 2% power, two reactor coolant leak detection systems of different operating principles shall be operable, with one of the two systems sensitive to radioactivity. The systems sensitive to radioactivity may be out-of-service for 48 hours provided two other means to detect leakage are operable.
- 3.1.6.9 Loss of reactor coolant through reactor coolant pump seals and system valves to connecting systems which vent to the gas vent header and from which coolant can be returned to the reactor coolant system shall not be considered as reactor coolant leakage and shall not be subject to the consideration of Specifications 3.1.6.1, 3.1.6.2, 3.1.6.3, 3.1.6.4, 3.1.6.5, 3.1.6.6 or 3.1.6.7 except that such losses when added to leakage shall not exceed 30 gpm.

Bases

Every reasonable effort will be made to reduce reactor coolant leakage including evaporative losses (which may be on the order of .5 gpm) to the lowest possible rate and at least below 1 gpm in order to prevent a large