

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

FILE NUMBER

TO:
Mr. Benard C. RuscheFROM:
Duke Power Company
Charlotte, North Carolina
Mr. William O. Parker, Jr.DATE OF DOCUMENT
7/26/76DATE RECEIVED
7/30/76☒ LETTER
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PROP

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DESCRIPTION

Ltr. re their 3/19/76 ltr.....regarding
provided information concerning the effects
of a postulated spent fuel cask drop accident.

ENCLOSURE

PLANT NAME:

Oconee 1-2-3

(1-P)

ACKNOWLEDGED

DO NOT REMOVE

SAFETY

FOR ACTION/INFORMATION

ENVIRO 8/2/76

RJL

ASSIGNED AD:

BRANCH CHIEF:

PROJECT MANAGER:

LIC. ASST.:

Schwencer (6)

Sheppard

ASSIGNED AD:

BRANCH CHIEF:

PROJECT MANAGER:

LIC. ASST.:

INTERNAL DISTRIBUTION

REG FILE

NRC PDR

I & E (2)

OELD

GOSSICK & STAFF

MIPC

CASE

HANAUER

HARLESS

SYSTEMS SAFETY

HEINEMAN

SCHROEDER

ENGINEERING

MACCARRY

KNIGHT

SIHWEIL

PAWLICKI

PLANT SYSTEMS

TEDESCO

BENAROYA

LAINAS

IPPOLITO

KIRKWOOD

OPERATING REACTORS

STELLO

SITE SAFETY &

ENVIRO ANALYSIS

DENTON & MULLER

ENVIRO TECH.

ERNST

BALLARD

SPANGLER

SITE TECH.

CAMMILL

STAPP

HULMAN

SITE ANALYSIS

VOLLMER

BUNCH

J. COLLINS

KREGER

PROJECT MANAGEMENT

BOYD

P. COLLINS

HOUSTON

PETERSON

MELTZ

HELTEMES

SKOVHOLT

REACTOR SAFETY

ROSS

NOVAK

ROSZTOCZY

CHECK

AT & I

SALTZMAN

RUTBERG

OPERATING TECH.

EISENHUT

SHAO

BAER

BUTLER

GRIMES

EXTERNAL DISTRIBUTION

LPDR: Walhalla, S.C.

TIC:

NSIC:

ASLB:

ACRS 16 CYS HOLDING SENT: SHEPPARD

NAT LAB:

REG. VIE

LA PDR

CONSULTANTS

BROOKHAVEN NAT LAB

ULRIKSON(ORNL)

CONTROL NUMBER

7669 T

DUKE POWER COMPANY

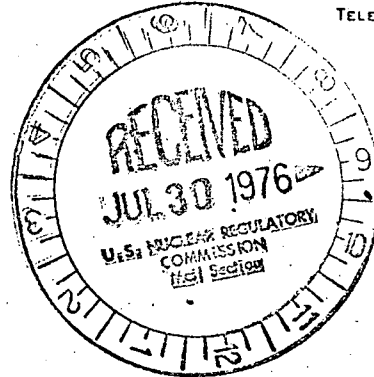
POWER BUILDING

422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28242

WILLIAM O. PARKER, JR.
VICE PRESIDENT
STEAM PRODUCTION

TELEPHONE: AREA 704
373-4083

July 26, 1976



Mr. Benard C. Rusche
Director of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Attention: Mr. A. Schwencer

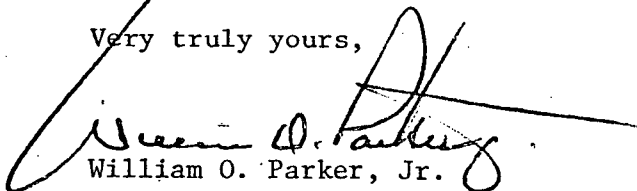
Re: Oconee Nuclear Station
Docket Nos. 50-269, -270, -287

Dear Mr. Rusche:

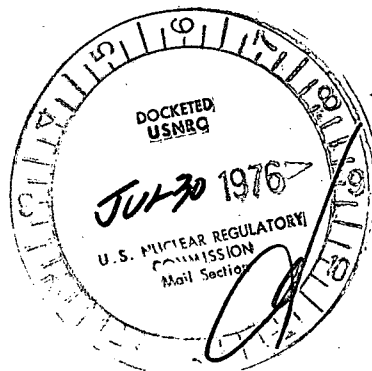
My letter dated March 19, 1976 provided information concerning the effects of a postulated spent fuel cask drop accident, including a description of the calculation performed to determine the maximum number of fuel assemblies affected and the corresponding dose at the exclusion area boundary.

A review of this calculation has indicated that the number of affected assemblies should be increased from 73 to 76 assemblies. This will raise the exclusion area boundary whole body dose to 8.5 Rem and the thyroid dose to 132.9 Rem. As can be seen, the radiological consequences of this postulated accident remain well within the established limits of 10CFR100.

Very truly yours,


William O. Parker, Jr.

MST:vr



7669

Regulatory Docket File