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50-269/270/287

REC: CASE E G
NRC

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DOCDATE: 01/23/78
DATE RCVD: 01/31/78

DOCTYPE: LETTER NOTARIZED: NO
SUBJECT:

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ADVISING OF APPLICANT'S PREPARATION OF REPT THAT WILL ASSESS THE RECENT
SEISMIC ACTIVITY IN THE JOCASSEE/KEOWEE AREA, TO BE SUBMITTED IN MARCH OF
1978.

PLANT NAME: OCONEE - UNIT 1
OCONEE - UNIT 2
OCONEE - UNIT 3

REVIEWER INITIAL: XJM
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***** DISTRIBUTION OF THIS MATERIAL IS AS FOLLOWS *****

NOTES:

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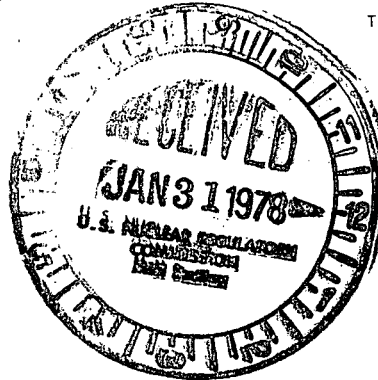
DUKE POWER COMPANY
POWER BUILDING
422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28242

WILLIAM O. PARKER, JR.
VICE PRESIDENT
STEAM PRODUCTION

January 23, 1978

TELEPHONE: AREA 704
373-4083

Mr. E. G. Case, Acting Director
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555



RE: Oconee Nuclear Station
Docket No. 50-269, -270, -287

Dear Mr. Case:

In recent weeks there has been a general increase of seismic activity in the Jocassee/Keowee area. Since initial detection, the number of microearthquake events along with maximum local magnitude and hypocentral locations have been reported to you daily. Portable seismic instruments have been located in the area since January 2, 1978.

The daily reports of these events as well as local seismic monitoring will continue as long as the seismic activity continues at a significant level.

Duke Power Company is currently preparing a report that will assess the recent activity in this area. It is anticipated that the report will be submitted in March 1978 and will include to the extent feasible the following:

- a) a catalog of hypocentral locations and composite focal plane solutions,
- b) a determination of any potential relationships between microearthquake activity and significant geologic structures.

Very truly yours,

William O. Parker, Jr.

RLG:ge

780310022

App'ls
1/0

Docket Nos. 50-269

JAN 18 1978

Duke Power Company
ATTN: Mr. William O. Parker, Jr.
Vice President
Steam Production
Post Office Box 2178
422 South Church Street
Charlotte, North Carolina 28242

Gentlemen:

RE: Oconee Unit 1

By letter dated October 1, 1976, you submitted a proposed inservice inspection and testing program description and a request for relief from selected ASME Code requirements pursuant to 10 CFR 50.55a(g). Although we have not completed our detailed review of your submittal, our preliminary review makes clear to us that your proposed program to implement those ASME Code requirements that you have found to be practical would increase the scope of inservice inspection and testing for your facility beyond that currently required by your Technical Specifications. We have concluded that this upgrading of your inservice inspection and testing program will further enhance safety.

Based on our preliminary review, we agree with your determination that it is impractical within the limitations of design, geometry and materials of construction of components, for you to meet certain of the specified ASME Code requirements and that imposition of those requirements would result in hardships or unusual difficulties without a compensating increase in the level of quality or safety. Therefore, pursuant to 10 CFR 50.55a(g)(6)(i), we hereby grant relief, on an interim basis, pending completion of our detailed review, from those inservice inspection and testing requirements of the ASME Code that you have requested. Moreover, since the scope of the inservice inspection and testing will be increased by your proposed program, and the granting of this relief is based only on the impracticality of selected ASME Code requirements, we have determined that the relief granted neither increases the probability or consequences of accidents previously considered nor decreases safety margins and that, therefore, it does not involve a significant hazards consideration. Therefore, you are authorized to, and should proceed to, implement your proposed program (except where your current Technical Specifications are more restrictive).

OFFICE >						
SURNAME >						
DATE >						

JAN 18 1978

During the period between now and the date we complete our detailed review of your submittal, you must comply with both your existing Technical Specifications and your proposed inservice inspection and testing program. In the event conflicting requirements arise for some components, you must comply with the more restrictive requirements (e.g., shorter inspection intervals, increased number of parameters measured). In other words, the granting of this relief from ASME Code requirements should not be interpreted to give you relief from any of the requirements in your existing Technical Specifications.

When our detailed review of your submittal is complete we will: (1) issue final approval of your program (which may contain modifications resulting from the staff's review), (2) grant relief from any ASME Code requirements that are determined to be impractical for your facility for the duration of the inspection interval and (3) issue appropriate changes to your Technical Specifications.

A copy of the Federal Register Notice related to this action is enclosed.

Sincerely,

Original Signed By

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Enclosure:
Federal Register Notice

cc w/enclosure:
See next page

DISTRIBUTION
Docket
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DATE >						

Duke Power Company

- 2 -

January 18, 1978

cc: Mr. William L. Porter
Duke Power Company
P. O. Box 2178
422 South Church Street
Charlotte, North Carolina 28242

J. Michael McGarry, III, Esquire
DeBevoise & Liberman
700 Shoreham Building
806-15th Street, NW.,
Washington, D.C. 20005

Oconee Public Library
201 South Spring Street
Walhalla, South Carolina 29691

UNITED STATES NUCLEAR REGULATORY COMMISSION

Docket No. 50-269

DUKE POWER COMPANY

NOTICE OF GRANTING OF RELIEF FROM ASME SECTION XI
INSERVICE INSPECTION (TESTING) REQUIREMENTS

The U. S. Nuclear Regulatory Commission (the Commission) has granted relief from certain requirements of the ASME Code, Section XI, "Rules for Inservice Inspection of Nuclear Power Plant Components" to Duke Power Company. The relief relates to the inservice inspection (testing) program for the Oconee Nuclear Station, Units 1, located in Oconee County, South Carolina. The ASME Code requirements are incorporated by reference into the Commission's rules and regulations in 10 CFR Part 50. The relief is effective as of its date of issuance.

The relief is granted, on an interim basis, pending completion of our detailed review, from those inservice inspection and testing requirements of the ASME Code that the licensee has determined to be impractical within the limitations of design, geometry and materials of construction of components, because compliance would result in hardships and unusual difficulties without a compensating increase in the level of quality or safety.

The request for relief complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the letter granting relief. Prior public notice of this action was not required since the granting of this relief from ASME Code requirements does not involve a significant hazards consideration.

The Commission has determined that the granting of this relief will not result in any significant environmental impact and that pursuant to 10 CFR §1.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with this action.

For further details with respect to this action, see (1) the request for relief dated October 1, 1976, and (2) the Commission's letter to the licensee dated January 18, 1978.

These items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Oconee County Library, 201 South Spring, Walhalla, South Carolina 29691. A copy of item (2) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 18th day of January 1978.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in cursive script, appearing to read "A. Schwencer".

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Confidential Files

January 18, 1978

Docket No. 50-269/270/287

Duke Power Company
ATTN: Mr. W. O. Parker, Jr.,
Vice President
Oconee Nuclear Station
P.O. Box 1175
Seneca, South Carolina 29678

Gentlemen:

On January 18, 1978, a member of our operator licensing staff was assigned to the NRC Region II Office as the Operator Licensing Branch Regional Representative. This transfer has been made as part of a pilot test program which includes evaluation of the feasibility of regionalizing the Operator Licensing Branch functions. Mr. Joseph J. Buzy of our office has been selected for this position.

The duties of the Regional Representative will include administration of all nonpower reactor examinations. When requested by Headquarters, he will assist in conducting examinations at power reactors and audit requalification program written examinations in the Region. He will review any changes to the licensed operator requalification programs. Periodically, the Representative will audit training programs at utility and vendor training centers. All these assignments will be performed in accordance with NRR procedures. When audits and reviews are completed, he will forward his recommendations to Headquarters.

When assigned by the Regional Director, the OLB Representative will audit training and requalification programs at any facility. The results of these audits will be included in the inspection reports

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which are issued by the Regional Office. Audits of this nature will be performed in accordance with IE procedures. Inspection results of training programs are used by the NRC to validate commitments made in the safety analysis report, the facility license and on individual applications. Inspection results of requalification programs are also used in evaluation of applications of renewal of licenses.

Correspondence, including applications for licenses, renewal of licenses and changes to requalification programs should continue to be sent to the U. S. Nuclear Regulatory Commission, Office of Nuclear Reactor Regulation, Washington, D. C. 20555.

Mr. Buzy should be contacted directly at the Region II Office (Area Code 404, 221-5509) for the following:

1. Scheduling of nonpower reactor examinations;
2. Proposed changes to licensed operator training or requalification programs for all facilities with operating licenses.

If you have any questions concerning this pilot test program, please contact Mr. Buzy in Region II or Paul F. Collins, Chief, Operator Licensing Branch.

Sincerely,

Original Signed By
Roger S. Boyd

Roger S. Boyd, Director
Division of Project Management
Office of Nuclear Reactor Regulation

DISTRIBUTION:

OLB Reading File	DPM PM
OLB Facility File	DOR PM
R.S. Boyd	DPM Branch Chief
R.C. DeYoung	DOR Branch Chief
D.J. Skovholt	DPM Licensing Assistant
D.B. Vassallo	DOR Licensing Assistant
R.P. Denise	I&E (3)
V. Stello	
J.W. Reece	
K.R. Goller	
D.G. Eisenhut	
J.R. Miller	
PDR	
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OFFICE ➤	OLB: PM	PM: ADO	PM: NDR			
SURNAME ➤	PFC Collins: ib	D.J. Skovholt	R.S. Boyd			
DATE ➤	1/17/78	1/17/78	1/17/78			