

Docket Nos. 50-269  
50-270  
and 50-287

JAN 14 1977

Duke Power Company  
ATTN: Mr. William O. Parker, Jr.  
Vice President  
Steam Production  
Post Office Box 2178  
422 South Church Street  
Charlotte, North Carolina 28242

Gentlemen:

RE: OCONEE NUCLEAR STATION, UNITS NOS. 1, 2 AND 3

Enclosed for your information is a copy of a letter dated January 3, 1977 from Robert D. Pollard, which is being considered as a Request for Action pursuant to 10 CFR 2.206, regarding your License Nos. DPR-38, DPR-47 and DPR-55 and others.

A copy of the Notice we are filing with the Office of the Federal Register relating to this request is also enclosed for your information.

The letter from Mr. Pollard, among other things, states that a refueling accident inside the containment building may not have been adequately considered during the licensing review of your facility. As you are aware, we recently contacted members of your staff to obtain information for an independent assessment of such a postulated accident at your facility. Based on our preliminary review, potential site boundary radiation exposures due to such an accident at your facility would be relatively low and well within 10 CFR Part 100 guidelines even assuming no containment isolation or effluent filtration. On the basis of this preliminary review we expect that modifications to your facility will not be required.

In order to confirm these results and to document the factors involved in the evaluation, we request that you provide a detailed evaluation of the potential consequences of such an accident at your facility within 60 days of receipt of this letter. Your analysis should utilize assumptions comparable to those given in Regulatory Guide 1.25, "Assumptions Used for Evaluating the Potential Radiological Consequences of a Fuel Handling Accident in the Fuel Handling and Storage Facility for Boiling and Pressurized Water Reactors" and consider, in a conservative manner, any mixing in the containment

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atmosphere which would delay release of material, any filtration of effluent which would reduce releases and any automatic isolation of the containment which would limit releases. Your evaluation should consist of two parts: (1) a conservative analysis using parameters (e.g., maximum allowable valve closure times) as limited by the technical specifications and (2) an analysis using parameters associated with current known facility operating conditions (e.g., actual valve closure times). Clearly indicate the environment for which the equipment is qualified, including seismic, and whether the equipment can withstand a single failure.

Sincerely,

Original signed by

A. Schwencer, Chief  
Operating Reactors Branch #1  
Division of Operating Reactors

## Enclosures:

1. Letter dtd. 1/3/77  
from R. D. Pollard
2. Federal Register Notice

cc w/enclosures:

See next page

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Duke Power Company

- 3 - January 14, 1977

cc: Mr. William L. Porter  
Duke Power Company  
P. O. Box 2178  
422 South Church Street  
Charlotte, North Carolina 28242

Mr. Troy B. Conner  
Conner & Knotts  
1747 Pennsylvania Avenue, N. W.  
Washington, D. C. 20006

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