

05/11/78

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)  
DISTRIBUTION FOR INCOMING MATERIAL 50-270

REC: OREILLY J P  
NRC

ORG: PARKER W O  
DUKE PWR

DOCDATE: 05/05/78  
DATE RCVD: 05/10/78

DOCTYPE: LETTER NOTARIZED: NO

COPIES RECEIVED

SUBJECT:

LTR 1 ENCL 1

FORWARDING LICENSEE EVENT REPT (RO 50-270/78-006) ON 04/07/78 CONCERNING THE  
RC LEAK RATE EXCEEDED 1 GPM... LEAK WAS PRIMARILY A PACKING LEAK ON 2RC-3,  
THE SPRAY CONTROL BLOCK VALVE, RESULTING FROM BLOWN PACKING...W/ATT.

PLANT NAME: OCONEE - UNIT 2

REVIEWER INITIAL: XJM

DISTRIBUTOR INITIAL: *Me*

\*\*\*\*\* DISTRIBUTION OF THIS MATERIAL IS AS FOLLOWS \*\*\*\*\*

NOTES:

1. M. CUNNINGHAM - ALL AMENDMENTS TO FSAR AND CHANGES TO TECH SPECS

INCIDENT REPORTS  
(DISTRIBUTION CODE A002)

FOR ACTION: BR CHIEF REID\*\*W/4 ENCL

INTERNAL:

REG FILE\*\*W/ENCL  
I & E\*\*W/2 ENCL  
SCHROEDER/IPPOLITO\*\*W/ENCL  
NOVAK/CHECK\*\*W/ENCL  
KNIGHT\*\*W/ENCL  
HANAUER\*\*W/ENCL  
EISENHUT\*\*W/ENCL  
SHAO\*\*W/ENCL  
KREGER/J. COLLINS\*\*W/ENCL  
K SEYFRIT/IE\*\*W/ENCL

NRC PDR\*\*W/ENCL  
MIPC\*\*W/3 ENCL  
HOUSTON\*\*W/ENCL  
EEB\*\*W/ENCL  
BUTLER\*\*W/ENCL  
TEDESCO\*\*W/ENCL  
BAER\*\*W/ENCL  
VOLLMER/BUNCH\*\*W/ENCL  
ROSA\*\*W/ENCL

EXTERNAL:

LPDR'S  
WALHALLA, SC\*\*W/ENCL  
TIC\*\*W/ENCL  
NSIC\*\*W/ENCL  
ACRS CAT B\*\*W/16 ENCL

COPIES NOT SUBMITTED PER  
REGULATORY GUIDE 10.1

DISTRIBUTION: LTR 45 ENCL 45  
SIZE: 1P+1P+1P

CONTROL NBR: 781310011

\*\*\*\*\*

THE END

\*\*\*\*\*

DUKE POWER COMPANY  
POWER BUILDING  
422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28242

REGULATORY DOCKET FILE COPY

WILLIAM O. PARKER, JR.  
VICE PRESIDENT  
STEAM PRODUCTION

May 5, 1978

US NRC  
DISTRIBUTION SERVICES  
BRANCH

1978 MAY 10 AM 11 40

TELEPHONE: AREA 704  
375-8083

DISTRIBUTION  
SERVICES UNIT

Mr. James P. O'Reilly, Director  
U. S. Nuclear Regulatory Commission  
Suite 1217  
230 Peachtree Street, Northwest  
Atlanta, GA 30303

RE: Oconee Unit 2  
Docket No. 50-270

Dear Mr. O'Reilly:

Pursuant to Sections 6.2 and 6.6.2 of the Oconee Nuclear Station Technical Specifications, please find attached Reportable Occurrence Report RO-270/78-6.

Very truly yours,

*William O. Parker by WAH*  
William O. Parker, Jr.

KRW/rpc

Attachment

cc: Director, Office of Management Information  
and Program Control

781310011

A002  
5  
1/1

DUKE POWER COMPANY  
OCONEE UNIT 2

Report Number: RO-270/78-6

Report Date: May 5, 1978

Occurrence Date: April 7, 1978

Facility: Oconee Unit 2, Seneca, South Carolina

Identification of Occurrence: Reactor Coolant Leakage in Excess of 1 gpm

Conditions Prior to Occurrence: 100% Full Power

Description of Occurrence:

At 1000, on April 6, 1978, it was determined that the reactor coolant leakage rate exceeded 1 gpm and an investigation was commenced. By 1500, personnel had entered the Reactor Building (RB) and observed the leak to be from a valve packing and would require unit shutdown to repair. A safety evaluation pursuant to Oconee Technical Specification 3.1.6.5 was performed. At 2200 on April 8, 1978, the leak was determined to be coming from Spray Control Outlet Block Valve, 2RC-3. This valve and an instrument line root valve were both repacked. On April 12, 1978, the unit was started up but prior to achieving criticality, an additional leak was discovered in the pressurizer heater bundle. This leak was repaired on April 22, 1978. The unit was then returned to service.

Cause of Occurrence:

The leaks on 2RC-3 and the instrument line root valve were both caused by blown packing. The leak in the pressurizer heater bundle was caused by a flange seal weld failure which was evidently initiated during startup after the valve repairs.

Analysis of Occurrence:

The leakage rate experience throughout the incident was well within the capacity of one HPI pump and no adverse effect on Reactor Coolant System capabilities resulted. The leakage was entirely contained within the Reactor Building. Public health and safety were not endangered by this incident.

A total of 40.885 man-rem of exposure were received by 104 persons involved in the investigation and repair operations.

Corrective Action:

The two leaking valves were repacked and the pressurizer heater bundle flange was cleaned and seal welded.

# LICENSEE EVENT REPORT

EXHIBIT A

CONTROL BLOCK: 1										(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)									
01 S C N E E 2 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4										5									
7 8 9 14 15 25 26 30 37 CAT 58																			
CON'T																			
01																			
REPORT SOURCE L 6 0 5 0 0 0 2 7 0 7 0 4 0 7 7 8 8 0 5 0 5 7 8 9																			
80 81 88 89 94 95 98 99																			
EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10																			
02 On April 6, 1978, the RC leak rate exceeded 1 gpm. It was determined that the																			
03 leak was primarily a packing leak on 2RC-3, the Spray Control Block Valve,																			
04 the valve and an instrument line root valve were repacked. During startup,																			
05 another leak was discovered in the pressurizer heater bundle. The weld was																			
06 repaired and the unit returned to service on April 22, 1978. No adverse																			
07 effects resulted since all leakage was contained and no loss of RCS capability,																			
08 was experienced.																			
7 8 9																			
09																			
SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE																			
C J 11 E 12 X 13 V A L V E X 14 F 15 D 16																			
9 10 11 12 13 18 19 20																			
17 LER/RO REPORT NUMBER EVENT YEAR SEQUENTIAL REPORT NO. OCCURRENCE CODE REPORT TYPE REVISION NO.																			
7 8 21 22 24 26 27 28 29 30 31 32																			
ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPRO-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER																			
B 18 Z 19 A 20 A 21 0 3 8 0 Y 23 Y 24 N 25 R 3 4 0 26																			
33 34 35 36 37 40 41 42 43 44 47																			
CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27																			
10 The valve leaks were a result of blown packing. The pressurizer heater																			
11 bundle leak was caused by a flange seal weld failure evidently initiated																			
12 during startup after the valve repairs. The valves were repacked and the																			
13 flange was re-welded to repair the leaks.																			
14																			
7 8 9																			
15 FACILITY STATUS % POWER OTHER STATUS METHOD OF DISCOVERY DISCOVERY DESCRIPTION																			
E 28 1 0 0 29 NA 30 A 31 Operator Observation 32																			
9 10 12 13 44 45 46 80																			
16 ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY LOCATION OF RELEASE																			
Z 33 Z 34 NA 35 NA 36																			
7 8 9 10 11 44 45 80																			
17 PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION																			
1 0 4 37 E 38 Total exposure for investigation and repair-40.885 man-rem																			
7 8 9 11 12 13 80																			
18 PERSONNEL INJURIES NUMBER DESCRIPTION																			
0 0 0 40 NA 41																			
7 8 9 11 12 80																			
19 LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION																			
Z 42 NA 43																			
7 8 9 10 80																			
20 PUBLICITY ISSUED DESCRIPTION																			
Y 44 Explanation of outage 45																			
7 8 9 10 80																			
NAME OF PREPARER K. R. Wilson																			
PHONE: (704) 373-8197																			
7 8 9 10 80																			