

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL  
(TEMPORARY FORM)CONTROL NO: 775FILE: INCIDENT FILE

|   |                   |           |                        |                       |           |  |     |       |
|---|-------------------|-----------|------------------------|-----------------------|-----------|--|-----|-------|
| FROM: Duke Pwr Co<br>Charlotte, NC<br>W O Parker Jr |                   |           | DATE OF DOC<br>1-22-76 | DATE REC'D<br>1-27-76 | LTR<br>XX | TWX  | RPT | OTHER |
| TO: Mr Moseley                                      |                   |           | ORIG<br>one signed     | CC                    | OTHER     | SENT NRC PDR <u>XX</u><br>SENT LOCAL PDR <u>XX</u> |     |       |
| CLASS   | UNCLASS<br>XXXXXX | PROP INFO | INPUT                  | NO CYS REC'D<br>1     |           | DOCKET NO:<br>50-270                               |     |       |

## DESCRIPTION:

Ltr trans the following:

## ENCLOSURES:

Reportable Occurrence #76-1 on 1-10-76  
concerning unidentified reactor coolant  
leakage exceeding lgpm.....PLANT NAME: Oconee #2

FOR ACTION/INFORMATION

1-29-76 ehf

BRANCH CHIEF Purple (3)LIC. ASST. Sheppard W/16 cys ACRS

## INTERNAL DISTRIBUTION

☒ REC FILE  
☒ NRC PDR  
☒ I&E (2)  
☒ MIPC (3)  
☒ SCHRODER/IPPOLITO  
☒ HOUSTON  
☒ NOVAK/CHECK  
☒ GRIMES/SCHWENCER  
☒ CASE  
☒ F. WILLIAMS  
☒ HANAUER  
☒ TEDESCO/MACCARY  
☒ EISENHUT  
☒ BAER  
☒ SHAO

☒ VOLLMER/BUNCH  
☒ \*KREGER/J. COLLINS

\*NOTE: IF PERSONEL EXPOSURE IS INVOLVED ONLY.

## EXTERNAL DISTRIBUTION

☒ LOCAL PDR Walthalla, SC  
☒ TIC  
☒ NSIC

DISTRIBUTION REVISED 1-19-76 by D. CRUTCHFIELD, TECH REVIEW COORDINATOR

# DUKE POWER COMPANY

POWER BUILDING

422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28242

WILLIAM O. PARKER, JR.  
VICE PRESIDENT  
STEAM PRODUCTION

TELEPHONE: AREA 704  
373-4083

January 22, 1976

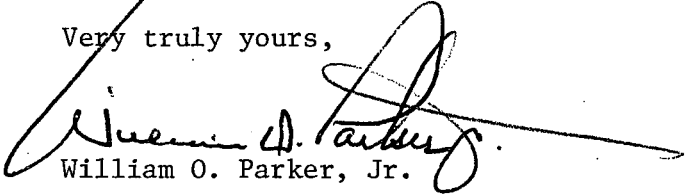
Mr. Norman C. Moseley, Director  
U. S. Nuclear Regulatory Commission  
Suite 818  
230 Peachtree Street, Northwest  
Atlanta, Georgia 30303

Re: Oconee Unit 2  
Docket No. 50-270

Dear Mr. Moseley:

Pursuant to Sections 6.2 and 6.6.2 of the Oconee Nuclear Station  
Technical Specifications, please find attached Reportable Occurrence  
Report RO-270/76-1.

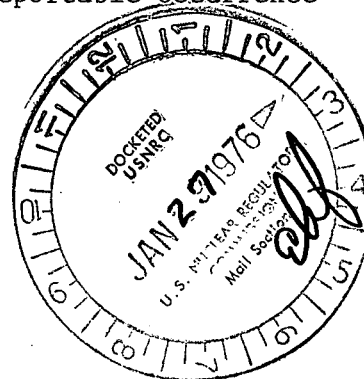
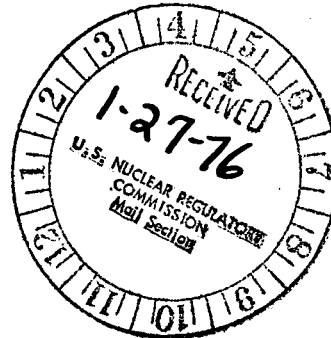
Very truly yours,

  
William O. Parker, Jr.

EDB:mmb

Attachment

CC Mr. Benard C. Rusche



DUKE POWER COMPANY  
OCONEE UNIT 2

Report No.: RO-270/76-1

Report Date: January 22, 1976

Occurrence Date: January 10, 1976

Facility: Oconee Unit 2, Seneca, South Carolina

Identification of Occurrence: Unidentified reactor coolant leakage exceeding 1 gpm.

Conditions Prior to Occurrence: Unit at 100% full power.

Description of Occurrence:

On January 10, 1976, surveillance testing was performed to determine the Reactor Coolant System leakage for Oconee Unit 2. A system leakage greater than 1 gpm, the maximum unidentified reactor coolant leakage as stated in Oconee Technical Specification 3.1.6.2, was detected. Within approximately 13 hours, the leakage was determined to be in an area of the Reactor Building which is inaccessible during reactor operation. A reactor shutdown was begun to facilitate inspection in these areas. Further investigation traced the leakage to an upper instrument root valve to pressurizer level transmitter LT-1. Within a total elapsed time of 19 hours following the initial determination of leakage, reactor shutdown was completed, and coolant leakage was reduced to allowable limits by isolating the pressurizer level transmitter and closing the root valve.

Apparent Cause of Occurrence:

This incident was apparently caused by a packing leak on an upper instrument root valve to pressurizer level transmitter LT-1.

Analysis of Event:

Estimated leakage during this incident was approximately 3 gpm. The leakage was contained inside the Reactor Building and did not present a problem in respect to in-plant radioactivity and contamination. The amount of leakage was well within the capacity of the Radioactive Waste Treatment System to handle, and the amount of make-up necessary to the primary system was well within the capacity of the High Pressure Injection System. Also, the leakage did not exist through a non-isolable fault in a reactor coolant strength boundary. It is concluded that the health and safety of the public was not affected by this incident.

RO-270/76-1

January 21, 1976

Page 2

Corrective Action:

Pressurizer level transmitter LT-1 was isolated, and the leaking root valve was closed to stop the leakage. The valve packing will be repaired during a subsequent reactor outage. In the meantime, pressurizer level will be monitored during operation by two additional back-up pressurizer level transmitters.

JAN 23 9 41 AM '76

U.S. AIR FORCE  
REGULATORY OPERATIONS  
REGION II  
ATLANTA, GA.