

Cheryl A. Gayheart
Vice President - Farley

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October 9, 2015

Docket Nos.: 50-348

NL-15-1837

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D. C. 20555-0001

Joseph M. Farley Nuclear Plant – Unit 1
Licensee Event Report 2015-004-00
Pressurizer Safety Valve Setpoint Pressure Outside
of Technical Specification Tolerance Band

Ladies and Gentlemen:

This Licensee Event Report is being submitted pursuant to the requirements of the Code of Federal Regulations, 10 CFR 50.73(a)(2)(i)(B), for a condition prohibited by Technical Specifications.

This letter contains no NRC commitments. If you have any questions regarding the submittal, please contact Ms. Julie Collier at (334) 814-4639.

Sincerely,

A handwritten signature in black ink, appearing to read "Cheryl A. Gayheart".

Ms. C. A. Gayheart
Vice President – Farley

CAG/JAC

Enclosure: Unit 1 Licensee Event Report 2015-004-00

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cc: Southern Nuclear Operating Company

Mr. S. E. Kuczynski, Chairman, President & CEO

Mr. D. G. Bost, Executive Vice President & Chief Nuclear Officer

Mr. M. D. Meier, Vice President – Regulatory Affairs

Mr. D. R. Madison, Vice President – Fleet Operations

Mr. B. J. Adams, Vice President – Engineering

Ms. B. L. Taylor, Regulatory Affairs Manager - Farley

RTYPE: CFA04.054

U. S. Nuclear Regulatory Commission

Mr. V. M. McCree, Regional Administrator

Mr. L. D. Wert, Regional Administrator (Acting)

Mr. S. A. Williams, NRR Project Manager - Farley

Mr. P. K. Niebaum, Senior Resident Inspector - Farley

Enclosure

Joseph M. Farley Nuclear Plant – Unit 1

Unit 1 Licensee Event Report 2015-004-00

NL-15-1837

Pressurizer Safety Valve Setpoint Pressure Outside
of Technical Specification Tolerance Band



LICENSEE EVENT REPORT (LER)

Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA, Privacy and Information Collections Branch (T-5 F53), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to infocollections.Resource@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

1. FACILITY NAME Joseph M. Farley Nuclear Plant, Unit 1	2. DOCKET NUMBER 05000 348	3. PAGE Page 1 of 3
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4. TITLE
Pressurizer Safety Valve Setpoint Pressure Outside of Technical Specification Tolerance Band

5. EVENT DATE			6. LER NUMBER			7. REPORT DATE			8. OTHER FACILITIES INVOLVED	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO.	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
08	11	2015	2015 - 004 - 00			10	09	2015	FACILITY NAME	DOCKET NUMBER

9. OPERATING MODE 1	11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check all that apply)									
	<input type="checkbox"/> 20.2201(b)	<input type="checkbox"/> 20.2203(a)(3)(I)	<input type="checkbox"/> 50.73(a)(2)(I)(C)	<input type="checkbox"/> 50.73(a)(2)(vii)						
	<input type="checkbox"/> 20.2201(d)	<input type="checkbox"/> 20.2203(a)(3)(II)	<input type="checkbox"/> 50.73(a)(2)(II)(A)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)						
	<input type="checkbox"/> 20.2203(a)(1)	<input type="checkbox"/> 20.2203(a)(4)	<input type="checkbox"/> 50.73(a)(2)(II)(B)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)						
	<input type="checkbox"/> 20.2203(a)(2)(I)	<input type="checkbox"/> 50.36(c)(1)(I)(A)	<input type="checkbox"/> 50.73(a)(2)(III)	<input type="checkbox"/> 50.73(a)(2)(ix)(A)						
10. POWER LEVEL 100 %	<input type="checkbox"/> 20.2203(a)(2)(II)	<input type="checkbox"/> 50.36(c)(1)(II)(A)	<input type="checkbox"/> 50.73(a)(2)(IV)(A)	<input type="checkbox"/> 50.73(a)(2)(x)						
	<input type="checkbox"/> 20.2203(a)(2)(III)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(V)(A)	<input type="checkbox"/> 73.71(a)(4)						
	<input type="checkbox"/> 20.2203(a)(2)(IV)	<input type="checkbox"/> 50.46(a)(3)(II)	<input type="checkbox"/> 50.73(a)(2)(V)(B)	<input type="checkbox"/> 73.71(a)(5)						
	<input type="checkbox"/> 20.2203(a)(2)(V)	<input type="checkbox"/> 50.73(a)(2)(I)(A)	<input type="checkbox"/> 50.73(a)(2)(V)(C)	<input type="checkbox"/> OTHER						
	<input type="checkbox"/> 20.2203(a)(2)(VI)	<input checked="" type="checkbox"/> 50.73(a)(2)(i)(B)	<input type="checkbox"/> 50.73(a)(2)(V)(D)	Specify in Abstract below or in NRC Form 368A						

12. LICENSEE CONTACT FOR THIS LER

FACILITY NAME Julie A. Collier – Licensing Engineer	TELEPHONE NUMBER (Include Area Code) (334) 814-4639
--	--

13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT

CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX
E	SB	RV	C710	Y					

14. SUPPLEMENTAL REPORT EXPECTED <input type="checkbox"/> YES (If yes, complete 15. EXPECTED SUBMISSION DATE) <input checked="" type="checkbox"/> NO	15. EXPECTED SUBMISSION DATE	MONTH	DAY	YEAR
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ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)

On 8/11/15 Farley Nuclear Plant (FNP) Unit 1 discovered that a pressurizer safety valve (PSV), which had been removed during the May 2015 maintenance outage and shipped offsite for testing, failed its as-found lift test below the Technical Specification (TS) allowable value. The cause of the test failure was attributed to the seat leakage of the valve that occurred during and after startup from the Spring 2015 Refueling Outage (1R26). This failure constitutes a condition that is reportable pursuant to 10CFR50.73 (a)(2)(i)(B), "Any operation or condition which was prohibited by the plant's Technical Specifications." The 1B pressurizer safety valve was replaced during a May 2015 planned maintenance outage.



**LICENSEE EVENT REPORT (LER)
CONTINUATION SHEET**

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NARRATIVE

PLANT AND SYSTEM IDENTIFICATION

Westinghouse - Pressurized Water Reactor
Energy Industry Identification System codes appear in the text as (EIS Code XX).

DESCRIPTION OF EVENT

On 8/11/15 during testing at an offsite facility the as-found lift setting for the Unit 1 1B pressurizer safety valve (PSV) (EIS Code RV) was discovered to be 2425 psig which is outside of the Technical Specification (TS) allowable lift settings of ≥ 2460 psig and ≤ 2510 psig. On 5/4/15 during startup from 1R26 with the plant in Mode 3, it was determined that the 1B PSV was leaking into the pressurizer relief tank (PRT) based on elevated tailpipe temperatures. On 5/8/15 the tailpipe temperature leakage indication stabilized with average PRT in-leakage at 0.12 gpm. Compensatory actions were established which included increased monitoring of leak rate, PRT parameters, radiation levels, PRT venting, and decision points for re-evaluating the need to replace the valve. Based upon the continued leakage Unit 1 entered a planned maintenance outage on 5/26/15 to replace the 1B PSV. The valve was then shipped to an offsite facility for lift testing and valve disassembly and inspection.

CAUSE OF EVENT

Visual inspections determined that the cause was seat leakage of the valve. The seat leakage resulted in 355 degree wire draw on the valve disc which reduced the setpoint to outside the TS band.

REPORTABILITY ANALYSIS AND SAFETY ASSESSMENT

This failure constitutes a condition that is reportable pursuant to 10CFR50.73 (a)(2)(i)(B), "Any operation or condition which was prohibited by the plant's Technical Specifications." Based upon a review of relevant information (e.g., equipment history and the cause of failure), there is no firm evidence for when the failure to meet the lift setting requirements occurred prior to the time of discovery at the test facility. The setpoint would have drifted below the allowable value at some time between startup from 1R26 and the time of discovery.

Since the as-found lift setpoint was lower than the allowed value in the Technical Specifications, the condition did not have an adverse impact on over pressurization, and the valve continued to perform its overpressure protection function. The as-found lift pressure was 2425 psig and the valve reclosed following the lift. This is within the safety analysis assumptions that are credited for pressurizer safety valves, and the plant remained bounded by the accident analyses in the FSAR. Therefore, this condition had no significant effect on the health and safety of the public.



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CONTINUATION SHEET**

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NARRATIVE

CORRECTIVE ACTION

The 1B pressurizer safety valve was replaced during the May 2015 planned maintenance outage.

ADDITIONAL INFORMATION

- Other system affected: No systems other than those mentioned in this report were affected by this event.
- Commitment Information: This report does not create any licensing commitments.
- Previous Similar Events: A search did not reveal any similarly reported events for Plant Farley.