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AUTH. NAME AUTHOR AFFILIATION  
 TUCKER, H.B. Duke Power Co.  
 RECIPIENT NAME RECIPIENT AFFILIATION  
 Document Control Branch (Document Control Desk)

SUBJECT: Application for amends to Licenses DPR-38, DPR-47 & DPR-55  
 changing primary to secondary leakage rate limit.

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# DUKE POWER COMPANY

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HAL B. TUCKER  
VICE PRESIDENT  
NUCLEAR PRODUCTION

TELEPHONE  
(704) 373-4531

May 31, 1988

U. S. Nuclear Regulatory Commission  
Attention: Document Control Desk  
Washington, D. C. 20555

Subject: Oconee Nuclear Station  
Docket Nos. 50-269, -270, -287  
Proposed Technical Specification Change  
Primary to Secondary Leakage Rate Limit

Dear Sir:

By letter dated October 2, 1987 the NRC Staff requested that Duke Power Company (Duke) propose revisions to the Oconee Nuclear Station (ONS) Technical Specifications (TSs) to provide specific limits for allowable primary to secondary leakage. In an initial response dated December 16, 1987, Duke informed the NRC that appropriate proposed revisions to the ONS TSs are being prepared and Duke anticipates a submittal by June 1, 1988.

Pursuant to 10 CFR 50, §50.90, please find attached (Attachment 1) a proposed amendment to the ONS Facility Operating Licenses and revisions to the ONS TSs. The proposed revisions to the ONS TSs are in response to the NRC letter dated October 2, 1987 and would establish a specific limit of 1.0 gpm per unit on allowable primary to secondary leakage for all ONS units. In addition, several administrative changes are included in this request.

The proposed TS revision (Attachment 1) includes two changes to the current TS 3.1.6.4. These changes are:

- o The limit of 1.0 gpm total primary to secondary leakage through both steam generators per unit for all Oconee units compared to a present limit of 0.3 gpm for Unit 1 and no specific limit for Units 2 and 3.
- o Deletion of the last sentence in the current TS 3.1.6.4 which require the NRC notification of steam generator tube leaks "in accordance with Section 6.6.2.1".

Other changes include:

- o Deletion of the current TS 4.17.6.c on page 4.17-4 which require the NRC notification of the results of steam generator tube inspections which fall into Category C-3 "pursuant to TS 6.6.2.1.a prior to resumption of plant operation."
- o Deletion of the current requirement for a "prompt notification to NRC pursuant to TS 6.6.2.1.a" in Table 4.17-1, Item C-3, on page 4.17-6.

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- o Changing of the term "Director" to "Regional Administrator" in present TS 4.17.6.a on page 4.17-4.

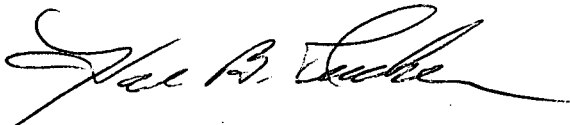
Also, pursuant to 10 CFR 50.36(a), included in this submittal is a proposed new paragraph under item (b) in Attachment 1 for addition to the "Bases" section beginning on page 3.1-14a of the ONS TSs. This change in the "Bases" section is not to be considered as part of the TS. Specifically, 10 CFR 50, 50.36(a) states that "a summary statement of the bases or reasons for such specifications, other than those covering administrative controls, shall also be included in the application, but not become part of technical specifications". As such the added paragraph to the "Bases" section is provided to advise the NRC of the need to update the bases. The requirements of 10 CFR 50.92 for determination of a no significant hazards consideration are not applicable for this change and the NRC approval is not required.

Duke has determined that the attached proposed amendment involves no significant hazards considerations. Attachments 2 and 3 provide the supporting technical justification and the No Significant Hazards Consideration Evaluation, respectively.

Duke is forwarding a copy of this application to the South Carolina Department of Health and Environmental Control for review and, as appropriate, subsequent consultation with the Staff.

Pursuant to 10 CFR 170, §170.12, please find enclosed a check in the amount of \$150.00 for the application fee.

Very truly yours,



Hal B. Tucker

MAH/14/sbn

Attachments

xc: Dr. J. Nelson Grace  
Regional Administrator, Region II  
U. S. Nuclear Regulatory Commission  
101 Marietta Street, NW, Suite 2900  
Atlanta, Georgia 30323

Ms. Helen Pastis  
Office of Nuclear Reactor Regulation  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Mr. Heyward, Shealy, Chief  
Bureau of Radiological Health  
S.C. Dept. of Health and  
Environmental Control  
2600 Bull Street  
Columbia, South Carolina 29201

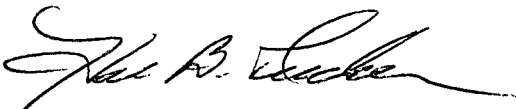
Mr. P. H. Skinner  
NRC Resident Inspector  
Oconee Nuclear Station

May 31, 1988

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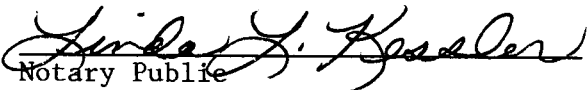
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N. A. Rutherford  
G. B. Swindlehurst  
BWOG TS Committee (8)  
Group File: OS-801.01

HAL B. TUCKER, being duly sworn, states that he is Vice President of Duke Power Company; that he is authorized on the part of said Company to sign and file with the Nuclear Regulatory Commission this request for amendment of the Oconee Nuclear Station Technical Specifications, Appendix A to Facility Operating Licenses DPR-38, DPR-47, and DPR-55; and that all statements and matter set forth therein are true and correct to the best of his knowledge.



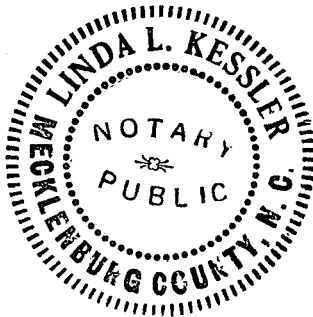
Hal B. Tucker, Vice President

Subscribed and sworn to before me this 31st day of May, 1988.

  
Notary Public

My Commission Expires:

May 1, 1989



Attachment 1

Duke Power Company  
Oconee Nuclear Station

Proposed Technical Specification Revision

- (a) Replace the current Technical Specification 3.1.6.4 on page 3.1-14 of the Oconee Technical Specifications with the following specification.

"3.1.6.4 If the total leakage through the tubes of both steam generators equals or exceeds 1.0 gpm, a reactor shutdown shall be initiated within 4 hours and the reactor shall be in a cold condition within the next 36 hours. If the leakage is less than 1.0 gpm, an assessment shall be made whether operations may be continued safely or the plant should be shutdown."

- (b) Add the following new paragraph to the end of the "Bases" section which begins on page 3.1-14a of the Oconee Technical Specifications.

"The steam generator tube leakage limit (i.e., primary to secondary leakage limits) in Specification 3.1.6.4 is intended to ensure timely shutdown of the plant for appropriate corrective action before rupture of the steam generator tubes occurs under normal operating or postulated accident conditions. These limits also serve to provide added assurance that the dosage contribution from tube leakage will be limited to a small fraction of 10 CFR Part 100 limits for a design basis steam generator tube rupture or main steam line break event."

- (c) Delete current Technical Specification 4.17.6.c on page 4.17-4.
- (d) Delete the requirement for a "Prompt notification to NRC pursuant to Specification 6.6.2.1.a" in Table 4.17-1, Item C-3, on page 4.17-6.
- (e) Change the term "Director" to "Regional Administrator" in present Specification 4.17.6.a on page 4.17-4.