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SUBJECT: Forwards clarification to Tech Spec 4.5.2 providing addl
 assurances that testing of valves associated w/reactor bldg
 cooling sys is per requirements specified in subsection IWV
 of Section IX of ASME boiler & pressure vessel code.

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December 1, 1987

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Subject: Oconee Nuclear Station
Docket Nos. 50-269, -270, -287
Reactor Building Cooling Systems

Gentlemen:

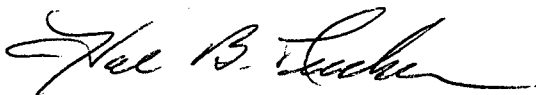
By letter dated February 6, 1986 Duke provided a proposed amendment to the Oconee Facility Operating License and revision to the Oconee Nuclear Station Technical Specifications (T.S.). The proposal consisted of a change to T.S. 4.5.2.1 Reactor Building Spray System. By letter dated August 20, 1986 Duke provided a supplement to the February 6, 1986 submittal which revised the No Significant Hazards Consideration (NSHC) evaluation.

Per subsequent conversations with the NRC Staff, it was requested that a clarification to T.S. 4.5.2 be added concerning testing of Reactor Building Cooling System valves. Accordingly, please find attached a supplement to the February 6, 1986 amendment request (Attachment 1) and appropriate revisions to the NSHC (Attachment 2) provided by the August 20, 1986 Duke letter.

The proposed clarification provides additional assurances that testing of valves associated with the Reactor Building Cooling System is in accordance with the requirements specified in subsection IWV of Section XI of the ASME Boiler and Pressure Vessel Code. The regulatory requirement for the implementation of Section XI by a licensee is provided by 10CFR 50.55a.

Inasmuch as the request for these proposed changes are a supplement to a previous Technical Specification amendment request, no license fees are required.

Very truly yours,



Hal B. Tucker

PJN/249/jgc

Attachment

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Page 2

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