

Duke Power Company
Oconee Nuclear Station

Proposed Technical Specification Revision
Reactor Vessel Materials Surveillance

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Bases

The surveillance program has been developed to comply with the applicable edition of Section XI and addenda of the ASME Boiler and Pressure Vessel Code, Inservice Inspection of Nuclear Reactor Coolant Systems, as required by 10 CFR 50.55(a) to the extent practicable within limitations of design, geometry and materials of construction. The program places major emphasis on the area of highest stress concentrations and on areas where fast neutron irradiation might be sufficient to change material properties.

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Attachment 2

No Significant Hazards Consideration Evaluation

Attachment 2

No Significant Hazards Consideration Evaluation

Duke Power Company (Duke) has made the determination that this amendment request involves a No Significant Hazards Consideration by applying the standards established by the Commission's regulations in 10 CFR 50.92. This ensures that operation of the facility in accordance with the proposed amendment would not:

- (1) Involve a significant increase in the probability or consequences of an accident previously evaluated; or
- (2) Create the possibility of a new or different kind of accident from any accident previously evaluated; or
- (3) Involve a significant reduction in a margin of safety.

The proposed Technical Specifications amendment addressed in this submittal reflects the new process in which changes to Oconee's Reactor Vessel Surveillance Program (RVSP) will be handled in the future. The current Oconee Nuclear Station Technical Specifications 4.2.4 and 4.2.5 for reactor vessel materials surveillance satisfy the requirements of Appendix H, 10 CFR 50. However, as a part of the Babcock and Wilcox Owners Group (B&WOG) Materials Committee integrated reactor vessel materials surveillance program these Technical Specifications are affected by changes in the program.

By a letter dated May 8, 1985, NRC found the B&WOG Materials Committee Report, BAW-1543, Revision 2 and 2A, "Integrated Materials Vessel Surveillance Program, February 1984," acceptable for referencing in Oconee Nuclear Station license applications in accordance with Section II.C of Appendix H, 10 CFR 50. This document provides the basis for and explains the Oconee Nuclear Station reactor vessel materials surveillance program including the Surveillance Capsule Insertion and Withdrawal schedule. This document will be maintained current to reflect changes in the program. Subsequent changes or revisions to the program will be made through revision of BAW-1543. If affected by the change, Duke will request the NRC approval for use of the modified integrated surveillance program for Oconee Nuclear Station per Section II.C of Appendix H of 10 CFR 50.

Inasmuch as the proposed Technical Specification change is in support of this program and that the NRC staff have accepted the BAW-1543 and found it applicable for Oconee Nuclear Station reactor vessel surveillance program it is considered unnecessary to retain Technical Specifications 4.2.4, 4.2.5 and Table 4.2-1.

Duke has determined, based on the consideration that the requested amendment will not alter the Oconee reactor vessel surveillance program, which is in compliance with the regulations, that the revisions do not involve a significant increase in the probability or consequences of accidents previously considered, nor create the possibility of a new or different kind of accident and will not involve a significant decrease in a safety margin. Therefore, Duke concludes that there is no significant hazards consideration involve in this amendment request.

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Attachment 3

Technical Justification

Integrated Reactor Vessel Material Surveillance Program

BAW-1543, Revision 2 and 2A