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 AUTH. NAME: PARKER, W.O. AUTHOR AFFILIATION: Duke Power Co.
 RECIPIENT NAME: O'REILLY, J.P. RECIPIENT AFFILIATION: Region 2, Atlanta, Office of the Director

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SUBJECT: Forwards response to IE Bulletin 79-050 per NRC 790726 ltr.
 "Analysis Summary in Support of Early Reactor Coolant Pump Trip" encl.

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WILLIAM O. PARKER, JR.
VICE PRESIDENT
STEAM PRODUCTION

August 24, 1979

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Mr. James P. O'Reilly, Director
U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street, Suite 3100
Atlanta, Georgia 30303

Re: RII:JPO
50-269
50-270
50-287

Dear Mr. O'Reilly:

With regard to your letter dated July 26, 1979 please find attached our response to IE Bulletin 79-05C, for Oconee Nuclear Station.

Very truly yours,

William O. Parker, Jr. W.A.H.
William O. Parker, Jr.

RLG:scs
Attachment

cc: Director, Office of Inspection and Enforcement
Washington, D. C. 20555

Director, Office of Nuclear Reactor Regulation
Washington, D. C. 20555

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ATTACHMENT 1

DUKE POWER COMPANY
OCOONEE NUCLEAR STATION

Response to IE Bulletin 79-03C

SHORT TERM ACTIONS

Item 1

On July 30, 1979, the following actions were taken:

- A. The appropriate Emergency Procedures, EP/O/A/1800/4, Loss of Reactor Coolant and EP/O/A/1800/3, Steam System Leak-Rupture, were revised to require that upon reactor trip and initiation of EPI caused by low reactor coolant system pressure, all operating reactor coolant pumps are tripped.
3. A letter was sent by the Superintendent of Operations to all Shift Supervisors requiring all Control Rooms to be manned as described in Bulletin Item 13.

Item 2

Attachment 2, "Analysis Summary in Support of an Early RC Pump Trip" is provided in response to this item. Section 3 of this report, "Impact Assessment of a RC Pump Trip on Non-LOCA Events," is included to allow the development of preliminary non-LOCA Guidelines as required by Item 3.

Item 3

Attachment 3, "Guidelines for Operator Action," provides guidelines on LOCA and non-LOCA transients.

Item 4

Following review and comment by B&W utilities, formal operating guidelines will be issued. At that time, appropriate emergency procedures will be revised and all licensed reactor operators and senior reactor operators on shift will be trained on the new guidelines. This will be completed on or before September 7, 1979.

Item 5

A preliminary response is provided in Attachment 2, Guidelines for Operator Action, Section III, "Criteria for RCP Restart." However, additional analysis is required and will be completed by October 31, 1979.

LONG TERM ACTION

Item 1

Attachment 4 provides a schematic of a conceptual design which would provide automatic tripping of operating reactor coolant pumps (RCP's) on coincident low reactor coolant system (RCS) pressure and low RCP power. The design would use existing engineered safety feature (ESF) low RCS pressure and reactor protective system (RPS) RCP power monitoring signals for input to an 'and' gate. Output would be to either the RCP breakers or the normal or startup feeder breakers. The power sources of each channel are completely independent. This RCP trip could be installed the first refueling outage for each unit following six months after NRC approval.

ATTACHEMENT 4

AUTOMATIC RCP TRIP SCHEMATIC

