

ATTACHMENT IV

MARKED CHANGES TO THE
MCGUIRE NUCLEAR STATION TECHNICAL SPECIFICATIONS

Technical Specifications Pages: 6-8, 6-9, 6-10, 6-11, 6-14, 6-16,
6-23, and Inserts "A" and "B"

IV-1

ADMINISTRATIVE CONTROLS

6.5 REVIEW AND AUDIT

INSERT "A"

6.5.1 TECHNICAL REVIEW AND CONTROL

ACTIVITIES

6.5.1.1 Each procedure and program required by Specification 6.8 and other procedures which affect nuclear safety, and changes thereto, shall be prepared by a ^{Knowledgeable} ~~qualified~~ individual/organization. Each such procedure, and changes thereto, shall be reviewed by an individual/group ^{organization} ~~other than the individual/group~~ which prepared the procedure, or changes thereto, ~~but who may be from the same organization as the individual/group which prepared the procedure, or changes thereto.~~ Procedures, or changes thereto, shall be approved ⁴ in accordance with Specifications 6.8.2 and 6.8.3.

6.5.1.2 Proposed changes to the Appendix A Technical Specifications shall be prepared by a ^{Knowledgeable} ~~qualified~~ individual/organization. The preparation of each proposed Technical Specifications change shall be reviewed by an individual/group ^{organization} ~~other than the individual/group~~ which prepared the proposed change, ~~but who may be from the same organization as the individual/group which prepared the proposed change.~~ Proposed changes to the Technical Specifications shall be approved by the Station Manager.

6.5.1.3 Proposed modifications to unit nuclear ^{Knowledgeable} ~~safety-related~~ structures, systems and components shall be designed by a ~~qualified~~ individual/organization. Each such modification shall be reviewed by an individual/group ^{organization} ~~other than the individual/group~~ which designed the modification, ~~but who may be from the same organization as the individual/group which designed the modification.~~ Proposed modifications to nuclear safety related structures, systems, and components shall be approved prior to implementation by the Station Manager or the Manager of Engineering; or for the Station Manager by the Mechanical Maintenance Superintendent, the Operations Superintendent, the I and E/Maintenance Support Superintendent, or the Work Control Superintendent, as previously designated by the Station Manager.

6.5.1.4 ³ Individuals responsible for reviews performed in accordance with Specifications 6.5.1.1, 6.5.1.2, and ~~6.5.1.3~~ shall be members of the site supervisory staff, previously designated by the Site Vice President to perform such reviews. Each such review shall include a determination of whether or not additional, cross-disciplinary, review is necessary. If deemed necessary, such review shall be performed by the appropriate designated site review personnel. ^{assigned to the site}

6.5.1.5 Proposed tests and experiments which affect station nuclear safety and are not addressed in the FSAR or Technical Specifications shall be reviewed by the Station Manager; or for the Station Manager by the Mechanical Maintenance Superintendent, the Operations Superintendent, the I and E/Maintenance Support Superintendent, or the Work Control Superintendent, as previously designated by the Station Manager.

INSERT "B"

INSERT "A" FOR MCGUIRE

Programs shall be established for the preparation, review, approval, and retention of documents required by the activities described in Specifications 6.5.1.1 through 6.5.1.11. Approvals shall be at the appropriate manager/superintendent level or above or as required by applicable Technical Specifications.

INSERT "B" FOR MCGUIRE

Proposed tests and experiments which affect station nuclear safety and are not addressed in the FSAR or Technical Specifications shall be prepared and approved in a manner identical to that of Specification 6.5.1.1. These proposed tests and experiments shall be reviewed by a knowledgeable individual/organization other than the individual/organization which prepared the proposed tests and experiments.

ADMINISTRATIVE CONTROLS

ACTIVITIES (Continued)

6.5.1.6 ALL REPORTABLE EVENTS and all violations of Technical Specifications shall be investigated and a report prepared which evaluates the occurrence and which provides recommendations to prevent recurrence. ~~The Manager, Safety Assurance shall assure such reports are developed and transmitted to the Site Vice President, or designee, who approves such reports, and to the Director of the Nuclear Safety Review Board.~~

6.5.1.7 ~~The Manager, Safety Assurance shall assure the performance of special reviews and investigations, and the preparation and submittal of reports thereon, as requested by the Site Vice President.~~

Such reports shall be reviewed by a knowledgeable individual/organization other than the individual/organization which prepared the report.
shall be performed by a knowledgeable individual/organization.

~~6.5.1.8 Deleted~~

~~6.5.1.9 Deleted~~

6.5.1.10 ~~The Manager, Safety Assurance shall assure the performance of a review by a qualified individual/organization of every unplanned onsite release of radioactive material to the environs including the preparation and forwarding of reports covering evaluation, recommendations, and disposition of the corrective ACTION to prevent recurrence, to the Site Vice President, and to the Nuclear Safety Review Board.~~

6.5.1.11 ~~The Manager, Safety Assurance shall assure the performance of a review by a qualified individual/organization of changes to the PROCESS CONTROL PROGRAM, OFFSITE DOSE CALCULATION MANUAL, and Radwaste Treatment Systems.~~

6.5.1.12 ~~The Manager, Safety Assurance shall ensure the performance of a review by a qualified individual/organization of the Fire Protection Program and implementing procedures, and submittal of recommended changes to the Nuclear Safety Review Board and Manager, Human Resources.~~

6.5.1.13 ~~Reports documenting each of the activities performed under Specifications 6.5.1.1 through 6.5.1.12 shall be maintained. Copies shall be provided to the Site Vice President, and the Nuclear Safety Review Board.~~

6.5.2 NUCLEAR SAFETY REVIEW BOARD (NSRB)

FUNCTION

6.5.2.1 The NSRB shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations,
- b. Nuclear engineering,
- c. Chemistry and radiochemistry,

ADMINISTRATIVE CONTROLS

FUNCTION (Continued)

- d. Metallurgy,
- e. Instrumentation and control,
- f. Radiological safety,
- g. Mechanical and electrical engineering, and
- h. Administrative control and quality assurance practices.

ORGANIZATION

6.5.2.2 The Director, members and alternate members of the NSRB shall be appointed in writing by the Senior Vice President, Nuclear Generation and shall have an academic degree in an engineering or physical science field; and in addition, shall have a minimum of 5 years technical experience, of which a minimum of 3 years shall be in one or more areas given in Specification 6.5.2.1. In special cases, candidates for appointment without an academic degree in engineering or physical science may be qualified with a minimum of ten years experience in one of the areas in Specification 6.5.2.1. No more than two alternates shall participate as voting members in NSRB activities at any one time.

6.5.2.3 The NSRB shall be composed of at least five members, including the Director. Members of the NSRB may be from the Nuclear Generation Department, from other departments within the Company, or from external to the Company. A maximum of one member of the NSRB may be from the McGuire Nuclear Site staff.

6.5.2.4 Consultants shall be utilized as determined by the NSRB Director to provide expert advice to the NSRB.

6.5.2.5 Staff assistance may be provided to the NSRB in order to promote the proper, timely, and expeditious performance of its functions.

6.5.2.6 The NSRB shall meet at least once per calendar quarter during the initial year of unit operation following fuel loading and at least ~~once per 6 months~~ thereafter. *twice*

6.5.2.7 The quorum of the NSRB necessary for the performance of the NSRB review and audit functions of these Technical Specifications shall consist of the Director, or ~~his~~ designated alternate, and at least four other NSRB members including alternates. No more than a minority of the quorum shall have line responsibility for operation of McGuire Nuclear Station.

ADMINISTRATIVE CONTROLS

REVIEW

6.5.2.8 The NSRB shall review:

- a. The safety evaluations for: (1) changes to procedures, equipment, or systems, and (2) tests or experiments completed under the provision of Section 50.59, 10 CFR to verify that such actions did not constitute an unreviewed safety question;
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR;
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR;
- ~~d. Proposed changes in Technical Specifications or this Operating License;~~
- d x.* Violations of Codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance;
- e x.* Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety;
- f x.* All REPORTABLE EVENTS;
- g x.* All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems or components that could affect nuclear safety;
- h x.* Quality Assurance Program audits relating to station operations and actions taken in response to these audits; and
- i x.* Reports of activities performed under the provisions of Specifications 6.5.1.1 through ~~6.5.1.12.~~

6.5.1.10

AUDITS

6.5.2.9 Audits of site activities shall be performed under the cognizance of the NSRB. These audits shall encompass:

- a. The conformance of unit operation to provisions contained within the Technical Specifications and applicable license conditions;
- b. The performance, training, and qualifications of the entire station staff;

ADMINISTRATIVE CONTROLS

SAFETY LIMIT VIOLATION (Continued)

- c. The Safety Limit Violation Report shall be submitted to the Commission, the NSRB and the Site Vice President, within 14 days of the violation; and
- d. Critical operation of the unit shall not be resumed until authorized by the Commission.

6.8 PROCEDURES AND PROGRAMS

6.8.1 Written procedures shall be established, implemented, and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix A of Regulatory Guide 1.33, Revision 2, February 1978;
- b. The applicable procedures required to implement the requirements of NUREG-0737;
- c. Deleted
- d. Deleted
- e. PROCESS CONTROL PROGRAM implementation;
- f. OFFSITE DOSE CALCULATION MANUAL implementation; and
- g. Quality Assurance Program for effluent and environmental monitoring.
- h. Technical Review and Control Program implementation.*
- i. Fire Protection Program implementation.*
- j. Plant Operations Review Committee implementation.*
- k. Commitments contained in FSAR Chapter 16.0*

6.8.2 Each procedure of Specification 6.8.1 above, and changes thereto, shall be reviewed and approved by an appropriate division manager, superintendent/manager, or one of their designated direct reports prior to implementation and shall be reviewed periodically as set forth in administrative procedures. For procedures which implement offsite environmental, technical, and laboratory activities, the above review and approval may be performed by the General Manager, Environmental Services or designee.

6.8.3 Temporary changes to procedures of Specification 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered;
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Operator license on the unit affected; and

ADMINISTRATIVE CONTROLS

PROCEDURES AND PROGRAMS (Continued)

- 4) Procedures for the recording and management of data,
- 5) Procedures defining corrective actions for all off-control point chemistry conditions, and
- 6) A procedure identifying: (a) the authority responsible for the interpretation of the data, and (b) the sequence and timing of administrative events required to initiate corrective action.

d. Backup Method for Determining Subcooling Margin

A program which will ensure the capability to accurately monitor the Reactor Coolant System subcooling margin. This program shall include the following:

- 1) Training of personnel, and
- 2) Procedures for monitoring.

e. Post-accident Sampling

A program which will ensure the capability to obtain and analyze reactor coolant, radioactive iodines, and particulates in plant gaseous effluents, and containment atmosphere samples under accident conditions. The program shall include the following:

- 1) Training of personnel,
- 2) Procedures for sampling and analysis, and
- 3) Provisions for maintenance of sampling and analysis equipment.

f. Radioactive Effluent Controls Program

A program shall be provided conforming with 10 CFR 50.36a for the control of radioactive effluents and for maintaining the doses to MEMBERS OF THE PUBLIC from radioactive effluents as low as reasonably achievable. The program (1) shall be contained in FSAR Chapter 16, (2) shall be implemented by operating procedures, and (3) shall include remedial actions to be taken whenever the program limits are exceeded. The program shall include the following elements:

- 1) Limitations on the operability of radioactive liquid and gaseous monitoring instrumentation including surveillance tests and setpoint determination in accordance with the methodology in the ~~ODCM~~, *Offsite Dose Calculation Manual (ODCM)*
- 2) Limitations on the concentrations of radioactive material released in liquid effluents to UNRESTRICTED AREAS conforming to 10 times 10 CFR Part 20.1001-20.2401, Appendix B, Table 2, Column 2,

ADMINISTRATIVE CONTROLS

RECORD RETENTION (Continued)

- g. Records of training and qualification for current members of the unit staff;
- h. Records of inservice inspections performed pursuant to these Technical Specifications;
- i. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59;
- j. Records of meetings of the NSRB and reports required by Specification ~~6.5.1.12~~, 6.5.1.11
- k. Records of the service lives of all snubbers including the date at which the service life commences and associated installation and maintenance records;
- l. Records of secondary water sampling and water quality; and
- m. Records of analyses required by the Radiological Environmental Monitoring Program that would permit evaluation of the accuracy of the analysis at a later date. This should include procedures effective at specified times and QA records showing that these procedures were followed.
- n. Records of reviews performed for changes made to the ODCM and the PCP.

6.10.3 Records of quality assurance activities required by the QA Manual shall be retained for a period of time required by ANSI N45.2.9-1974.

6.11 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

6.12 HIGH RADIATION AREA

6.12.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR 20, each high radiation area, as defined in 10 CFR Part 20, in which the intensity of radiation is equal to or less than 1000 mrem/hr at 45 CM (18 in.) from the radiation source or from any surface which the radiation penetrates shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit (RWP). Individuals qualified in radiation protection procedures (e.g., Radiation Protection Technician) or personnel continuously escorted by such individuals may be exempt from the RWP issuance requirement during the performance of their assigned duties in high radiation areas with exposure rates equal to or less than 1000 mrem/hr provided they are otherwise following plant radiation protection procedures for entry into high radiation areas.

ATTACHMENT V

MARKED CHANGES TO THE
CATAWBA NUCLEAR STATION TECHNICAL SPECIFICATIONS

Technical Specifications Pages: 6-7, 6-8, 6-9, 6-10, 6-13, 6-15,
6-21, and Inserts "A" and "B"

6.5 REVIEW AND AUDIT

INSERT "A"

6.5.1 TECHNICAL REVIEW AND CONTROL ACTIVITIES

6.5.1.1 ^{Knowledgeable} Each procedure and program required by Specification 6.8 and other procedures which affect nuclear safety, and changes thereto, shall be prepared by a ~~qualified individual/organization~~. Each such procedure, and changes thereto, shall be reviewed by an individual/group other than the individual/group which prepared the procedure, or changes thereto, ~~but who may be from the same organization as the individual/group which prepared the procedure, or changes thereto.~~ ^{Procedures, or changes thereto, shall be approved}

4 ^{Knowledgeable} 6.5.1.2 Proposed changes to the Appendix A Technical Specifications shall be prepared by a ~~qualified individual/organization~~. The preparation of each proposed Technical Specification change shall be reviewed by an individual/group other than the individual/group which prepared the proposed change, ~~but who may be from the same organization as the individual/group which prepared the proposed change.~~ ^{Proposed changes to the Technical Specifications shall be approved by the Station Manager.}

2 ^{Knowledgeable} 6.5.1.3 Proposed modifications to unit nuclear safety-related structures, systems, and components shall be designed by a ~~qualified individual/organization~~. Each such modification shall be reviewed by an individual/group other than the individual/group which designed the modification, ~~but who may be from the same organization as the individual/group which designed the modification.~~ ^{Proposed modifications to nuclear safety-related structures, systems, and components shall be approved prior to implementation by the Station Manager or the Manager of Engineering; or for the Station Manager by the Mechanical Maintenance Superintendent, the Operations Superintendent, the I and E/Maintenance Support Superintendent, or the Work Control Superintendent, as previously designated by the Station Manager.}

3 ^{and} 6.5.1.4 Individuals responsible for reviews performed in accordance with Specifications 6.5.1.1, 6.5.1.2, ~~and 6.5.1.3~~ shall be members of the site supervisory staff, previously designated by the Site Vice President to perform such reviews. Review of environmental radiological analysis procedures shall be performed by the General Manager, Environmental Services or his designee. Each such review shall include a determination of whether or not additional, cross-disciplinary, review is necessary. If deemed necessary, such review shall be performed by the appropriate designated site review personnel. ^{assigned to the site}

6.5.1.5 ~~Proposed tests and experiments which affect station nuclear safety and are not addressed in the FSAR or Technical Specifications shall be reviewed by the Station Manager; or for the Station Manager by the Mechanical Maintenance Superintendent, the Operations Superintendent, the I and E/Maintenance Support Superintendent, or the Work Control Superintendent, as previously designated by the Station Manager.~~

INSERT "B"

INSERT "A" FOR CATAWBA

Programs shall be established for the preparation, review, approval, and retention of documents required by the activities described in Specifications 6.5.1.1 through 6.5.1.11. Approvals shall be at the appropriate manager/superintendent level or above or as required by applicable Technical Specifications.

INSERT "B" FOR CATAWBA

Proposed tests and experiments which affect station nuclear safety and are not addressed in the FSAR or Technical Specifications shall be prepared and approved in a manner identical to that of Specification 6.5.1.1. These proposed tests and experiments shall be reviewed by a knowledgeable individual/organization other than the individual/organization which prepared the proposed tests and experiments.

TECHNICAL REVIEW AND CONTROL ACTIVITIES (Continued)

6.5.1.6 All REPORTABLE EVENTS and all violations of Technical Specifications shall be investigated and a report prepared which evaluates the occurrence and which provides recommendations to prevent recurrence. ~~The Manager, Safety Assurance shall assure such reports are developed and transmitted to the Site Vice President, or designee, who approves such reports, and to the Nuclear Safety Review Board.~~

Such reports shall be reviewed by a knowledgeable individual/organization other than the individual/organization which prepared the report.
 6.5.1.7 ~~The Manager, Safety Assurance shall assure the performance of special reviews and investigations, and the preparation and submittal of reports thereon, as requested by the Site Vice President.~~ *shall be performed by a knowledgeable individual/organization.*

~~6.5.1.8 Deleted~~

~~6.5.1.9 Deleted~~

8 Knowledgeable shall review
 A 6.5.1.10 ~~The Manager, Safety Assurance shall assure the performance of a review by a qualified individual/organization of every unplanned onsite and prepare release of radioactive material to the environs including the preparation and forwarding of reports covering evaluation, recommendations, and disposition of the corrective ACTION to prevent recurrence, to the Site Vice President, and to the Nuclear Safety Review Board.~~

9 Knowledgeable shall review
 A 6.5.1.11 ~~The Manager, Safety Assurance shall assure the performance of a review by a qualified individual/organization of changes to the PROCESS CONTROL PROGRAM, OFFSITE DOSE CALCULATION MANUAL, and Radwaste Treatment Systems.~~

10 Knowledgeable shall review
 A 6.5.1.12 ~~The Manager, Safety Assurance shall assure the performance of a review by a qualified individual/organization of the Fire Protection Program and implementing procedures, and the submittal of recommended changes to the Nuclear Safety Review Board, and Manager, Human Resources.~~

11 10
 6.5.1.13 ~~Reports documenting each of the activities performed under Specifications 6.5.1.1 through 6.5.1.12 shall be maintained. Copies shall be provided to the Site Vice President and the Nuclear Safety Review Board.~~

6.5.2 NUCLEAR SAFETY REVIEW BOARD (NSRB)

FUNCTION

6.5.2.1 The NSRB shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations,
- b. Nuclear engineering,
- c. Chemistry and radiochemistry,

FUNCTION (Continued)

- d. Metallurgy,
- e. Instrumentation and control,
- f. Radiological safety,
- g. Mechanical and electrical engineering, and
- h. Administrative control and quality assurance practices.

The NSRB shall report to and advise the Senior Vice President, Nuclear Generation on those areas of responsibility specified in Specifications 6.5.2.8 and 6.5.2.9.

ORGANIZATION

6.5.2.2 The Director, members, and alternate members of the NSRB shall be appointed in writing by the Senior Vice President, Nuclear Generation and shall have an academic degree in an engineering or physical science field; and in addition, shall have a minimum of 5 years technical experience, of which a minimum of 3 years shall be in one or more areas given in Specification 6.5.2.1. In special cases, candidates for appointment without an academic degree in engineering or physical science may be qualified with a minimum of ten years experience in one of the areas in Specification 6.5.2.1. No more than two alternates shall participate as voting members in NSRB activities at any one time.

6.5.2.3 The NSRB shall be composed of at least five members, including the Director. Members of the NSRB may be from the Nuclear Generation Department, from other departments within the Company, or from external to the Company. A maximum of one member of the NSRB may be from the Catawba Nuclear Site staff.

6.5.2.4 Consultants shall be utilized as determined by the NSRB Director to provide expert advice to the NSRB.

6.5.2.5 Staff assistance may be provided to the NSRB in order to promote the proper, timely, and expeditious performance of its functions.

6.5.2.6 The NSRB shall meet at least once per calendar quarter during the initial year of unit operation following fuel loading and at least ~~once~~ ^{twice} per ~~6 months~~ ^{year} thereafter.

6.5.2.7 The quorum of the NSRB necessary for the performance of the NSRB review and audit functions of these Technical Specifications shall consist of the Director, or ~~his~~ designated alternate, and at least four other NSRB members including alternates. No more than a minority of the quorum shall have line responsibility for operation of Catawba Nuclear Station.

REVIEW

6.5.2.8 The NSRB shall be responsible for the review of:

- a. The safety evaluation for: (1) changes to procedures, equipment, or systems, and (2) tests or experiments completed under the provision of Section 50.59, 10 CFR to verify that such actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment, or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR;
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR;
- ~~d. Proposed changes in Technical Specifications or this Operating License;~~
- d. Violations of Codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance;
- e. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety;
- f. All REPORTABLE EVENTS;
- g. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety;
- h. Quality Assurance Program audits relating to station operations and actions taken in response to these audits; and
- i. Reports of activities performed under the provisions of Specifications 6.5.1.1 through ~~6.5.1.12.~~ 6.5.1.10

AUDITS

6.5.2.9 Audits of site activities shall be performed under the cognizance of the NSRB. These audits shall encompass:

- a. The conformance of unit operation to provisions contained within the Technical Specifications and applicable license conditions;
- b. The performance, training, and qualifications of the entire station staff;

6.8 PROCEDURES AND PROGRAMS

6.8.1 Written procedures shall be established, implemented, and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix A of Regulatory Guide 1.33, Revision 2, February 1978;
- b. The emergency operating procedures required to implement the requirements of NUREG-0737 and Supplement No. 1 to NUREG-0737 as stated in Generic Letter No. 82-33;
- c. Deleted
- d. Deleted
- e. PROCESS CONTROL PROGRAM implementation;
- f. OFFSITE DOSE CALCULATION MANUAL implementation;
- g. Quality Assurance Program implementation for effluent and environmental monitoring;
- h. *Technical Review and Control Program implementation.*
- i. *Fire Protection Program implementation;*
- j. *Plant Operations Review Committee implementation.*
- k. Commitments contained in FSAR Chapter 16.0.

6.8.2 Each procedure of Specification 6.8.1, and changes thereto, shall be reviewed and approved by an appropriate division manager, superintendent/manager, or one of their designated direct reports prior to implementation and shall be reviewed periodically as set forth in administrative procedures. For procedures which implement offsite environmental, technical and laboratory activities, the above review and approval may be performed by the General Manager, Environmental Services or designee.

6.8.3 Temporary changes to procedures of Specification 6.8.1 may be made provided:

- a. The intent of the original procedure is not altered;
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Operator license on the unit affected; and
- c. The change is approved by an appropriate division manager, superintendent/manager or one of their designated direct reports within 14 days of implementation.

6.8.4 The following programs shall be established, implemented, and maintained:

- a. Primary Coolant Sources Outside Containment

A program to reduce leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. The systems include the containment spray, Safety Injection, chemical

ADMINISTRATIVE CONTROLS

PROCEDURES AND PROGRAMS (Continued)

e. Post-Accident Sampling

A program which will ensure the capability to obtain and analyze reactor coolant, radioactive iodines and particulates in plant gaseous effluents, and containment atmosphere samples under accident conditions. The program shall include the following:

- 1) Training of personnel,
- 2) Procedures for sampling and analysis, and
- 3) Provisions for maintenance of sampling and analysis equipment.

f. Radioactive Effluent Controls Program

A program shall be provided conforming with 10 CFR 50.36a for the control of radioactive effluents and for maintaining the doses to MEMBERS OF THE PUBLIC from radioactive effluents as low as reasonably achievable. The program (1) shall be contained in Chapter 16 of the FSAR, (2) shall be implemented by operating procedures, and (3) shall include remedial actions to be taken whenever the program limits are exceeded. The program shall include the following elements:

- 1) Limitations on the operability of radioactive liquid and gaseous monitoring instrumentation including surveillance tests and set-point determination in accordance with the methodology in the ~~ODCM~~, *Offsite Dose Calculation Manual (ODCM)*
- 2) Limitations on the concentrations of radioactive material released in liquid effluents to UNRESTRICTED AREAS conforming to 10 times 10 CFR Part 20.1001-20.2401, Appendix B, Table 2, Column 2,
- 3) Monitoring, sampling, and analysis of radioactive liquid and gaseous effluents in accordance with 10 CFR 20.1302 and with the methodology and parameters in the ODCM,
- 4) Limitations on the annual and quarterly doses or dose commitment to a MEMBER OF THE PUBLIC from radioactive materials in liquid effluents released from each unit to UNRESTRICTED AREAS conforming to Appendix I to 10 CFR Part 50,
- 5) Determination of cumulative and projected dose contributions from radioactive effluents for the current calendar quarter and current calendar year in accordance with the methodology and parameters in the ODCM at least every 31 days,
- 6) Limitations on the operability and use of the liquid and gaseous effluent treatment systems to ensure that the appropriate portions of these systems are used to reduce releases of radioactivity when the projected doses in a 31-day period would exceed 2 percent of

ADMINISTRATIVE CONTROLS

RECORD RETENTION (Continued)

- h. Records of inservice inspections performed pursuant to these Technical Specifications;
- i. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59;
- j. Records of meetings of the NSRB and reports required by Specification ~~6.5.1.10~~, 6.5.1.11
- k. Records of the service lives of all hydraulic and mechanical snubbers required by Specification 3.7.8 including the date at which the service life commences and associated installation and maintenance records;
- l. Records of secondary water sampling and water quality;
- m. Records of analyses required by the Radiological Environmental Monitoring Program that would permit evaluation of the accuracy of the analysis at a later date. This should include procedures effective at specified times and QA records showing that these procedures were followed; and
- n. Records of reviews performed for changes made to Section 16.11 (Radiological Effluent Controls) of the FSAR.

6.10.3 Records of quality assurance activities required by the Operational Quality Assurance Manual shall be retained for a period of time as recommended by ANSI N.45.2.9-1974.

6.11 RADIATION PROTECTION PROGRAM

6.11 Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained, and adhered to for all operations involving personnel radiation exposure.

6.12 HIGH RADIATION AREA

6.12.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR Part 20, each high radiation area, as defined in 10 CFR Part 20, in which the intensity of radiation is equal to or less than 1000 mR/h at 45 cm (18 in.) from the radiation source or from any surface which the radiation penetrates shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit (RWP). Individuals qualified in radiation protection procedures (e.g., Radiation Protection Technician) or personnel continuously escorted by such individuals may be exempt from the RWP issuance requirement during the performance of their assigned duties in high radiation areas with exposure rates equal to or less than 1000 mR/h, provided they are otherwise following plant radiation protection procedures for entry into such high radiation areas. Any individual or group of individuals permitted to enter such areas shall be provided with or accompanied by one or more of the following:

- a. A radiation monitoring device which continuously indicates the radiation dose rate in the area; or

ATTACHMENT VI

MARKED CHANGES TO THE
OCONEE NUCLEAR STATION TECHNICAL SPECIFICATIONS

Technical Specifications Pages:
v, 6.1-2, 6.1-3, 6.1-3a(1), 6.1-3a(2), 6.1-3b, 6.1-4, 6.4-1,
6.4-2, 6.4-2a, 6.4-2b, 6.5-1, 6.5-2, and Inserts "A" and "B"

3. Special reviews and investigations as requested by the Site Vice President; and
 4. Reports on unplanned onsite releases of radioactive material to the environs.
- c. The PORC shall review additional programs, procedures and plant activities as directed by the Site Vice President.

REMEDIAL ACTION: Not Applicable

BASIS:

- a. The PORC shall be established to recommend to the Station Manager approval or disapproval of the items listed under APPLICABILITY prior to their final approval.
- b. The PORC shall report to the Site Vice President on the areas of responsibility specified in this selected licensee commitment.

TESTING REQUIREMENTS: Not Applicable

REFERENCES:

1. ANSI N18.1-1971, Selection and Training of Nuclear Power Plant Personnel
2. Nuclear System Directive 308, Plant Operations Review Committee

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6.1.2.1 Activities

- a. Procedures required by Technical Specification 6.4 and other procedures which affect station nuclear safety, and changes (other than editorial or typographical changes) thereto, shall be prepared by a qualified individual/organization. Each such procedure, or procedure change, shall be reviewed by an individual/group other than the individual/group which prepared the procedure, or procedure change, but who may be from the same organization as the individual/group which prepared the procedure, or procedure change. Such procedures and procedure changes may be approved for temporary use by two members of the station staff, at least one of whom holds a Senior Reactor Operator's License on the unit(s) affected. Procedures and procedure changes shall be approved prior to use or within seven days of receiving temporary approval by an appropriate division manager, superintendent/manager, or one of their designated direct reports.
- b. Proposed changes to the Technical Specifications shall be prepared by a qualified individual/organization. The preparation of each proposed Technical Specifications change shall be reviewed by an individual/group other than the individual/group which prepared the proposed change, but who may be from the same organization as the individual/group which prepared the proposed change. Proposed changes to the Technical Specifications shall be approved by the Station Manager.
- c. Proposed modifications to station nuclear safety-related structures, systems and components shall be designed by a qualified individual/ organization. Each such modification shall be reviewed by an individual/group other than the individual/group which designed the modification, but who may be from the same organization as the individual/group which designed the modification. Proposed modifications to nuclear safety-related structures, systems, and components shall be approved prior to implementation by the Station Manager or the Manager of Engineering; or for the Station Manager by the Mechanical Maintenance Superintendent, the Operations Superintendent, the I and E/Maintenance Support Superintendent, or the Work Control Superintendent, as previously designated by the Station Manager.
- d. Individuals responsible for reviews performed in accordance with 6.1.2.1.a, 6.1.2.1.b, and 6.1.2.1.c shall be members of the Site supervisory staff, previously designated by the Site Vice President to perform such reviews. Each such review shall include a determination of whether or not additional, cross-disciplinary, review is necessary. If deemed necessary, such review shall be performed by the appropriate designated Site review personnel.
- e. Proposed tests and experiments which affect station nuclear safety and are not addressed in the FSAR or Technical Specifications shall be reviewed by the Station Manager; or for the Station Manager by the Mechanical Maintenance Superintendent, the Operations Superintendent, the I and E/Maintenance Support Superintendent, or the Work Control Superintendent, as previously designated by the Station Manager.
- f. Incidents reportable pursuant to Technical Specification 6.6.2.1 and violations of Technical Specifications shall be investigated and a report prepared which evaluates the occurrence and which provides recommendations to prevent recurrence. The Manager, Safety Assurance shall assure such reports are developed and transmitted to the Site Vice President, or designee, who approves such reports, and to the Director of the Nuclear Safety Review Board.
- g. The Manager, Safety Assurance shall assure the performance of special reviews and investigations, and the preparation and submittal of reports thereon, as requested by the Site Vice President.

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j. The Manager, Safety Assurance shall assure that an independent fire protection and loss prevention inspection and audit shall be performed utilizing qualified off-site personnel and that an inspection and audit by a qualified fire consultant shall be performed at intervals no greater than three years.

k. Unplanned onsite releases of radioactive material to the environs shall be investigated and a report prepared which evaluates the occurrence and which provides recommendations to prevent recurrence. The Manager, Safety Assurance shall assure such reports are developed and transmitted to the Site Vice President, or designee, and to the Director of the Nuclear Safety Review Board.

l. Licensee-initiated changes to the Offsite Dose Calculation Manual (ODCM) shall be documented and records of reviews performed shall be retained for the duration of the unit operating license. This documentation shall contain:

- 1) Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s); and,
- 2) A determination that the change will maintain the level of radioactive effluent control required by 10 CFR 20.1302, 40 CFR Part 190, 10 CFR 50.36a, and Appendix I to 10 CFR Part 50 and not adversely impact the accuracy or reliability of effluent, dose, or setpoint calculations; and,

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- 3) Shall become effective upon review and acceptance by the Station Manager and a qualified individual/organization; and,
 - 4) Shall be submitted to the Commission in the form of a complete, legible copy of the entire ODCM as a part of or concurrent with the Annual Radioactive Effluent Release Report for the period of the report in which any change to the ODCM was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed, and shall indicate the date (e.g., month/year) the change was implemented.
- m. Licensee-initiated changes to the Process Control Program shall be documented and records of reviews performed shall be retained for the duration of the unit license. This documentation shall contain:
- 1) Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s);
 - 2) A determination that the change will maintain the overall conformance of the solidified waste product to existing requirements of Federal, State, or other applicable regulations; and,
 - 3) Changes shall become effective upon review and acceptance by the Station Manager and a qualified individual/organization.

6.1.2.2 Records

Records of the above activities shall be maintained.

6.1.3 Nuclear Safety Review Board

6.1.3.1 Function

The NSRB shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations
- b. Nuclear Engineering
- c. Chemistry and radiochemistry
- d. Metallurgy
- e. Instrumentation and control
- f. Radiological safety
- g. Mechanical and electrical engineering
- h. Administrative control and quality assurance practices

6.1.3.2 Organization

- a. The Director, members and alternate members of the NSRB shall be formally appointed by the Senior Vice President, Nuclear Generation, and shall have an academic degree in an engineering or physical science field; and in addition, shall have a minimum of five years technical experience, of which a minimum of three years shall be in one or more areas given in 6.1.3.1.

In special cases, candidates for appointment without an academic degree in engineering or a physical science may be qualified with a minimum of ten years experience in one of the areas specified in Specification 6.1.3.1.

- b. ~~The NSRB shall be composed of at least five members, including the Director, Members of the NSRB may be from the Nuclear Generation Department, from other departments within the Company or from external to the~~

Replace with Insert "A"

~~Company. A maximum of one member of the NSRB may be from the Oconee Nuclear Site staff.~~

- ~~c. Consultants may be utilized by the NSRB to provide expert advice to the NSRB, as determined necessary by the Director of the NSRB.~~
- ~~d. Staff assistance may be provided to the NSRB in order to promote the proper, timely and expeditious performance of its functions.~~
- ~~e. The NSRB shall meet at least once per six months. The period between such meetings shall not exceed eight months.~~
- ~~f. A quorum of the NSRB shall consist of the Director, or his designated alternate, and at least two other NSRB members or alternate members. No more than a minority of the quorum shall have line responsibility for operation of Oconee Nuclear Station.~~

Replace with Insert "A"

6.1.3.3 Subjects Requiring Review

The following subjects shall be reported to and reviewed by the NSRB:

- a. The safety evaluations for (1) changes to procedures, equipment or systems, and (2) tests or experiments completed under the provisions of 10 CFR 50.59(a)(1) to verify that such actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in 10 CFR 50.59.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in 10 CFR 50.59.
- d. Proposed changes in Technical Specifications or the Facility Operating Licenses.
- e. Violations of applicable statutes, codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of station equipment that affect nuclear safety.
- g. Incidents that are the subject of non-routine reports submitted to the Commission.
- h. Quality Assurance Program audits relating to station operations and actions taken in response to these audits.
- i. Proposed changes to Fire Protection Program

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6.1.2 TECHNICAL REVIEW AND CONTROL

6.1.2.1 Activities

Programs shall be established for the preparation, review, approval, and retention of documents required by the activities described in Specifications 6.1.2.1a through 6.1.2.1k. Approvals shall be at the appropriate manager/superintendent level or above or as required by applicable Technical Specifications.

- a. Each procedure and program required by Specification 6.4 and other procedures which affect nuclear safety, and changes thereto, shall be prepared by a knowledgeable individual/organization. Each such procedure, and changes thereto, shall be reviewed by an individual/organization other than the individual/organization which prepared the procedure, or changes thereto. Procedures, or changes thereto, shall be approved in accordance with Specifications 6.4.2 and 6.4.3.
- b. Proposed modifications to unit nuclear safety-related structures, systems and components shall be designed by a knowledgeable individual/organization. Each modification shall be reviewed by an individual/organization other than the individual/organization which designed the modification.
- c. Individuals responsible for reviews performed in accordance with Specifications 6.1.2.1.a and 6.1.2.1.b shall be members of the supervisory staff assigned to the site, and previously designated by the Site Vice President to perform such reviews. Each such review shall include a determination of whether or not additional, cross-disciplinary, review is necessary. If deemed necessary, such review shall be performed by the appropriate designated site review personnel.
- d. Proposed changes to the Technical Specifications shall be prepared by a knowledgeable individual/organization. The preparation of each proposed Technical Specification change shall be reviewed by an individual/organization other than the individual/organization which prepared the proposed change.

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- e. Proposed tests and experiments which affect station nuclear safety and are not addressed in the FSAR or Technical Specifications shall be prepared and approved in a manner identical to that of Specification 6.1.2.1.a. These proposed tests and experiments shall be reviewed by an individual/organization other than the individual/organization which prepared the proposed tests and experiments.
- f. Incidents reportable pursuant to Technical Specification 6.6.2.1 and all violations of Technical Specifications shall be investigated and a report prepared which evaluates the occurrence and which provides recommendations to prevent recurrence. Such reports shall be reviewed by a knowledgeable individual/organization other than the individual/organization which prepared the report.
- g. Special reviews and investigations, and the preparation and submittal of reports thereon, shall be performed by a knowledgeable individual/organization.
- h. A knowledgeable individual/organization shall review every unplanned onsite release of radioactive material to the environs, and prepare reports covering evaluation, recommendations, and disposition of the corrective action to prevent recurrence.
- i. A knowledgeable individual/organization shall review changes to the Process Control Program, Offsite Dose Calculation Manual (ODCM), and Radwaste Treatment Systems.
- j. A knowledgeable individual/organization shall review the Fire Protection Program and implementing procedures.
- k. Reports documenting each of the activities performed under Specifications 6.1.2.1.a through 6.1.2.1.j shall be maintained.

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6.1.3 Nuclear Safety Review Board (NSRB)

6.1.3.1 Function

The NSRB shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations,
- b. Nuclear engineering,
- c. Chemistry and radiochemistry,
- d. Metallurgy,
- e. Instrumentation and control,
- f. Radiological safety,
- g. Mechanical and electrical engineering, and
- h. Administrative control and quality assurance practices.

6.1.3.2 Organization

- a. The Director, members and alternate members of the NSRB shall be appointed in writing by the Senior Vice President, Nuclear Generation and shall have an academic degree in an engineering or physical science field; and in addition, shall have a minimum of 5 years technical experience, of which a minimum of 3 years shall be in one or more areas given in Specification 6.1.3.1.
- b. In special cases, candidates for appointment without an academic degree in engineering or physical science may be qualified with a minimum of ten years experience in one of the areas in Specification 6.1.3.1. No more than two alternates shall participate as voting members in NSRB activities at any one time.
- c. The NSRB shall be composed of at least five members, including the Director. Members of the NSRB may be from the Nuclear Generation Department, from other departments within the Company, or from external to the Company. A maximum of one member of the NSRB may be from the Oconee Nuclear Site staff.

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- d. Consultants shall be utilized as determined by the NSRB Director to provide expert advice to the NSRB.
- e. Staff assistance may be provided to the NSRB in order to promote the proper, timely, and expeditious performance of its functions.
- f. The NSRB shall meet at least once per calendar quarter during the initial year of unit operation following fuel loading and at least twice per year thereafter.
- g. The quorum of the NSRB necessary for the performance of the NSRB review and audit functions of these Technical Specifications shall consist of the Director, or designated alternate, and at least four other NSRB members including alternates. No more than a minority of the quorum shall have line responsibility for operation of Oconee Nuclear Station.

6.1.3.3 Review

The NSRB shall review:

- a. The safety evaluations for: (1) changes to procedures, equipment, or systems, and (2) tests or experiments completed under the provision of Section 50.59, 10 CFR to verify that such actions did not constitute an unreviewed safety question;
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR;
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR;
- d. Violations of Codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance;
- e. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety;

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- f. ALL REPORTABLE EVENTS;
- g. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems or components that could affect nuclear safety;
- h. Quality Assurance Program audits relating to station operations and actions taken in response to these audits; and
- i. Reports of activities performed under the provisions of Specifications 6.1.2.1a through 6.1.2.1j.

6.4 STATION OPERATING PROCEDURES

Specification

6.4.1

The station shall be operated and maintained in accordance with approved procedures. Written procedures with appropriate check-off lists and instructions shall be provided for the following conditions:

- a. Normal startup, operation, and shutdown of the complete facility and of all systems and components involving nuclear safety of the facility.
- b. Refueling operations.
- c. Actions taken to correct specific and foreseen potential malfunctions of systems or components involving nuclear safety and radiation levels, including responses to alarms, suspected primary system leaks and abnormal reactivity changes.
- d. Emergency procedures involving potential or actual release of radioactivity.
- e. Preventive or corrective maintenance which could affect nuclear safety or radiation exposure to personnel.
- f. Station survey following an earthquake.
- g. Personnel radiation protection procedures.
- h. Operation of radioactive waste management systems.
- i. Control of pH in recirculated coolant after loss-of-coolant accident. Procedure shall state that pH will be measured and the addition of appropriate caustic to coolant will commence within 30 minutes after switchover to recirculation mode of core cooling to adjust the pH to a range of 7.0 to 8.0 within 24 hours.
- j. Nuclear safety-related periodic test procedures.
- k. Long-term emergency core cooling systems. Procedures shall include provision for remote or local operation of system components necessary to establish high and low pressure injection within 15 minutes after a line break.
- l. Fire Protection Program implementation.
- m. Offsite Dose Calculation Manual implementation.
- n. Process Control Program implementation.
- o. Technical Review and Control Program implementation.
- p. Plant Operations Review Committee implementation.

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6.4.2

Each procedure of specification 6.4.1 above, and changes thereto, shall be reviewed and approved by an appropriate division manager, superintendent/manager, or one of their designated direct reports prior to implementation, and shall be reviewed periodically as set forth in administrative procedures. For procedures which implement offsite environmental, technical, and laboratory activities, the above review and approval may be performed by the General Manager, Environmental Services or designee.

6.4.3

Temporary changes to procedures of Specification 6.4.1 above may be made provided:

- a. The intent of the original procedure is not altered;
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operators license on the affected unit; and
- c. The change is approved by a group manager, superintendent/manager, or one of their designated direct reports within 14 days of implementation.

6.4.4 The following programs shall be established, implemented, and maintained:

~~6.4.2~~ A respiratory protective program approved by the Commission shall be in force.

~~6.4.3~~ Administrative procedures shall be developed and implemented to limit the working hours of station staff who perform safety-related functions, e.g., senior reactor operators, reactor operators, nuclear equipment operators, and certain maintenance personnel.

Any deviations from the above procedures shall be authorized by the Station Manager (or designee) in accordance with established procedures and with documentation of the basis for granting the deviation. Individual overtime shall be periodically reviewed to assure that excessive hours have not been worked. Routine deviation from the above guidelines is not authorized.

~~6.4.4~~ The station shall have a program that ensures the capability to obtain and analyze reactor coolant and containment atmosphere samples under accident conditions which includes training of personnel, procedures for sampling and analysis, and provisions for testing and required maintenance of sampling and analysis equipment.

~~6.4.5~~ The station shall have a program that ensures the capability to collect and analyze or measure representative samples of radioactive iodines and particulates in plant gaseous effluents during and following an accident which includes training of personnel, procedures for sampling and analysis, and provisions for testing and required maintenance of sampling and analysis equipment.

~~6.4.6~~ The station shall have a program conforming with 10 CFR 50.36a for the control of radioactive effluents and for maintaining the doses to MEMBERS OF THE PUBLIC from radioactive effluents as low as reasonably achievable. The program (1) shall be contained in FSAR Chapter 16, (2) shall be implemented by operating procedures, and (3) shall include remedial actions to be taken whenever the program limits are exceeded. The program shall include the following elements:

1. Limitations on the operability of radioactive liquid and gaseous monitoring instrumentation including surveillance tests and set-point determination in accordance with the methodology in the ODCM,
2. Limitations on the concentrations of radioactive material released in liquid effluents to UNRESTRICTED AREAS conforming to 10 times 10 CFR Part 20.1001-20.2401, Appendix B, Table 2, Column 2,
3. Monitoring, sampling, and analysis of radioactive liquid and gaseous effluents in accordance with 10 CFR 20.1302 and with the methodology and parameters in the ODCM,

4 d. Limitations on the annual and quarterly doses or dose commitment to a MEMBER OF THE PUBLIC from radioactive materials in liquid effluents released from each unit to UNRESTRICTED AREAS conforming to Appendix I to 10 CFR Part 50,

5 g. Determination of cumulative and projected dose contributions from radioactive effluents for the current calendar quarter and current calendar year in accordance with the methodology and parameters in the ODCM at least every 31 days.

6 f. Limitations on the operability and use of the liquid and gaseous effluent treatment systems to ensure that the appropriate portions of these systems are used to reduce releases of radioactivity when the projected doses in a 31-day period would exceed 2 percent of the guidelines for the annual dose or dose commitment conforming to Appendix I to 10 CFR Part 50 as clarified by FSAR Chapter 16.

7 g. Limitations on the dose rate resulting from radioactive material released in gaseous effluents from the site to areas at or beyond the SITE BOUNDARY shall be limited to the following:

- a. For noble gases: Less than or equal to a dose rate of 500 mrem/yr to the total body and less than or equal to a dose rate of 3000 mrem/yr to the skin, and
- b. For Iodine-131, for Iodine-133, for tritium, and for all radionuclides in particulate form with half-lives greater than 8 days; Less than or equal to a dose rate of 1500 mrem/yr to any organ.

8 h. Limitations on the annual and quarterly air doses resulting from noble gases released in gaseous effluents from each unit to areas beyond the SITE BOUNDARY conforming to Appendix I to 10 CFR Part 50,

9 f. Limitations on the annual and quarterly doses to a MEMBER OF THE PUBLIC from Iodine-131, Iodine-133, tritium, and all radionuclides in particulate form with half-lives greater than 8 days in gaseous effluents released from each unit to areas beyond the SITE BOUNDARY conforming to Appendix I to 10 CFR Part 50; and,

10 f. Limitations on the annual dose or dose commitment to any MEMBER OF THE PUBLIC due to releases of radioactivity and to radiation from uranium fuel cycle sources conforming to 40 CFR Part 190.

~~6.4.7~~ f. The station shall have a program to monitor the radiation and radionuclides in the environs of the plant. The program shall provide (1) representative measurements of radioactivity in the highest potential exposure pathways, and (2) verification of the accuracy of the effluent monitoring program and modeling of environmental exposure pathways. The program shall (1) be contained in FSAR Chapter 16, (2) conform to the guidance of Appendix I to 10 CFR Part 50, and (3) include the following:

1 f. Monitoring, sampling, analysis, and reporting of radiation and radionuclides in the environment in accordance with the methodology and parameters in the ODCM,

- 2 ~~4~~. A Land Use Census to ensure that changes in the use of areas at and beyond the SITE BOUNDARY are identified and that modifications to the monitoring program are made if required by the results of this census; and,
- 3 ~~4~~. Participation in an Interlaboratory Comparison Program to ensure that independent checks on the precision and accuracy of the measurements of radioactive materials in environmental sample matrices are performed as part of the quality assurance program for environmental monitoring.

o. Records of meetings of the NSRB and reports required by Specification G.1.Z.1.K.

6.5 STATION OPERATING RECORDS

Specification

6.5.1 The following records shall be prepared and permanently retained in a manner convenient for review:

- a. Records of modifications to the station as described in the FSAR.
- b. Special nuclear material physical inventory records.
- c. Special nuclear material isotopic inventory records.
- d. Radiation monitoring records, including records of radiation and contamination surveys.
- e. Records of off-site environmental surveys.
- f. Personnel radiation exposure records as required by 10CFR20.
- g. Records of radioactive releases and waste disposal.
- h. Records of reactor coolant system in-service inspections.
- i. Preoperational testing records.
- j. Records of special reactor tests or experiments.
- k. Records of changes to safety-related operating procedures.
- l. Records for Environmental Qualification which are covered under the provisions of paragraph 6.7.

m. Records of the seal service lives of hydraulic snubbers.

n. Records of reviews performed for changes made to the ODCM and the Process Control Program.

6.5.2 The following records shall be prepared and retained for a minimum of six (6) years in a manner convenient for review:

- a. Switchboard Record.
- b. Reactor Operations Logbook.
- c. Shift Supervisor Logbook.
- d. Maintenance histories for station safety-related structures, systems and components.
- e. Records of safety-related inspections, other than reactor coolant system in-service inspections.
- f. Records of reportable events.
- g. Periodic testing records and records of other periodic checks, calibrations, etc. performed in accordance with surveillance requirements for safety-related parameters, structures, systems and components.

h. By-product material inventory records.

~~i. Minutes of Nuclear Safety Review Board Meetings.~~

c. ~~j.~~ Training records.

f. ~~g.~~ Test results, in units of microcuries, for leak tests performed pursuant to Specification 4.16.

k. ~~l.~~ Radioactive liquid effluent, gaseous effluent, and gaseous process monitoring instrumentation alarm/trip setpoints.

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OCONEE UNITS 1,2,3

v

Amendment No.	(Unit 1)
Amendment No.	(Unit 2)
Amendment No.	(Unit 3)

6.1.2.1 Activities

Programs shall be established for the preparation, review, approval, and retention of documents required by the activities described in Specifications 6.1.2.1a through 6.1.2.1k. Approvals shall be at the appropriate manager/superintendent level or above or as required by applicable Technical Specifications.

- a. Each procedure and program required by Specification 6.4 and other procedures which affect nuclear safety, and changes thereto, shall be prepared by a knowledgeable individual/organization. Each such procedure, and changes thereto, shall be reviewed by an individual/ organization other than the individual/organization which prepared the procedure, or changes thereto. Procedures, or changes thereto, shall be approved in accordance with Specifications 6.4.2 and 6.4.3.
- b. Proposed modifications to unit nuclear safety-related structures, systems and components shall be designed by a knowledgeable individual/organization. Each modification shall be reviewed by an individual/ organization other than the individual/organization which designed the modification.
- c. Individuals responsible for reviews performed in accordance with Specifications 6.1.2.1.a and 6.1.2.1.b shall be members of the supervisory staff assigned to the site, and previously designated by the Site Vice President to perform such reviews. Each such review shall include a determination of whether or not additional, cross-disciplinary, review is necessary. If deemed necessary, such review shall be performed by the appropriate designated site review personnel.
- d. Proposed changes to the Technical Specifications shall be prepared by a knowledgeable individual/organization. The preparation of each proposed Technical Specification change shall be reviewed by an individual/organization other than the individual/organization which prepared the proposed change.

- e. Proposed tests and experiments which affect station nuclear safety and are not addressed in the FSAR or Technical Specifications shall be prepared and approved in a manner identical to that of Specification 6.1.2.1.a. These proposed tests and experiments shall be reviewed by an individual/organization other than the individual/organization which prepared the proposed tests and experiments.
- f. Incidents reportable pursuant to Technical Specification 6.6.2.1 and all violations of Technical Specifications shall be investigated and a report prepared which evaluates the occurrence and which provides recommendations to prevent recurrence. Such reports shall be reviewed by a knowledgeable individual/ organization other than the individual/organization which prepared the report.
- g. Special reviews and investigations, and the preparation and submittal of reports thereon, shall be performed by a knowledgeable individual/organization.
- h. A knowledgeable individual/organization shall review every unplanned onsite release of radioactive material to the environs, and prepare reports covering evaluation, recommendations, and disposition of the corrective action to prevent recurrence.
- i. A knowledgeable individual/organization shall review changes to the Process Control Program, Offsite Dose Calculation Manual (ODCM), and Radwaste Treatment Systems.
- j. A knowledgeable individual/organization shall review the Fire Protection Program and implementing procedures.
- k. Reports documenting each of the activities performed under Specifications 6.1.2.1.a through 6.1.2.1.j shall be maintained.

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OCONEE UNITS 1,2,3

Amendment No.	(Unit 1)
Amendment No.	(Unit 2)
Amendment No.	(Unit 3)

6.1.3 Nuclear Safety Review Board (NSRB)

6.1.3.1 Function

The NSRB shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations,
- b. Nuclear engineering,
- c. Chemistry and radiochemistry,
- d. Metallurgy,
- e. Instrumentation and control,
- f. Radiological safety,
- g. Mechanical and electrical engineering, and
- h. Administrative control and quality assurance practices.

6.1.3.2 Organization

- a. The Director, members and alternate members of the NSRB shall be appointed in writing by the Senior Vice President, Nuclear Generation and shall have an academic degree in an engineering or physical science field; and in addition, shall have a minimum of 5 years technical experience, of which a minimum of 3 years shall be in one or more areas given in Specification 6.1.3.1.
- b. In special cases, candidates for appointment without an academic degree in engineering or physical science may be qualified with a minimum of ten years experience in one of the areas in Specification 6.1.3.1. No more than two alternates shall participate as voting members in NSRB activities at any one time.
- c. The NSRB shall be composed of at least five members, including the Director. Members of the NSRB may be from the Nuclear Generation Department, from other departments within the Company, or from external to the Company. A maximum of one member of the NSRB may be from the Oconee Nuclear Site staff.

6.1-4

OCONEE UNITS 1,2,3

Amendment No.	(Unit 1)
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- d. Consultants shall be utilized as determined by the NSRB Director to provide expert advice to the NSRB.
- e. Staff assistance may be provided to the NSRB in order to promote the proper, timely, and expeditious performance of its functions.
- f. The NSRB shall meet at least once per calendar quarter during the initial year of unit operation following fuel loading and at least twice per year thereafter.
- g. The quorum of the NSRB necessary for the performance of the NSRB review and audit functions of these Technical Specifications shall consist of the Director, or designated alternate, and at least four other NSRB members including alternates. No more than a minority of the quorum shall have line responsibility for operation of Oconee Nuclear Station.

6.1.3.3 Review

The NSRB shall review:

- a. The safety evaluations for: (1) changes to procedures, equipment, or systems, and (2) tests or experiments completed under the provision of Section 50.59, 10 CFR to verify that such actions did not constitute an unreviewed safety question;
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR;
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR;
- d. Violations of Codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance;
- e. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety;

6.1-4a

OCONEE UNITS 1,2,3

Amendment No.	(Unit 1)
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- f. ALL REPORTABLE EVENTS;
- g. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems or components that could affect nuclear safety;
- h. Quality Assurance Program audits relating to station operations and actions taken in response to these audits; and
- i. Reports of activities performed under the provisions of Specifications 6.1.2.1a through 6.1.2.1j.

6.1-4b

OCONEE UNITS 1,2,3

Amendment No.	(Unit 1)
Amendment No.	(Unit 2)
Amendment No.	(Unit 3)

6.4 STATION OPERATING PROCEDURES

Specification

- 6.4.1 The station shall be operated and maintained in accordance with approved procedures. Written procedures with appropriate check-off lists and instructions shall be provided for the following conditions:
- a. Normal startup, operation, and shutdown of the complete facility and of all systems and components involving nuclear safety of the facility.
 - b. Refueling operations.
 - c. Actions taken to correct specific and foreseen potential malfunctions of systems or components involving nuclear safety and radiation levels, including responses to alarms, suspected primary system leaks and abnormal reactivity changes.
 - d. Emergency procedures involving potential or actual release of radioactivity.
 - e. Preventive or corrective maintenance which could affect nuclear safety or radiation exposure to personnel.
 - f. Station survey following an earthquake.
 - g. Personnel radiation protection procedures.
 - h. Operation of radioactive waste management systems.
 - i. Control of Ph in recirculated coolant after loss-of-coolant accident. Procedure shall state that pH will be measured and the addition of appropriate caustic to coolant will commence within 30 minutes after switchover to recirculation mode of core cooling to adjust the pH to a range of 7.0 to 8.0 within 24 hours.
 - j. Nuclear safety-related periodic test procedures.

6.4-1

OCONEE UNITS 1,2,3

Amendment No.	(Unit 1)
Amendment No.	(Unit 2)
Amendment No.	(Unit 3)

- k. Long-term emergency core cooling systems. Procedures shall include provision for remote or local operation of system components necessary to establish high and low pressure injection within 15 minutes after a line break.
- l. Fire Protection Program implementation.
- m. Offsite Dose Calculation Manual implementation.
- n. Process Control Program implementation.
- o. Technical Review and Control Program implementation.
- p. Plant Operations Review Committee implementation.

6.4.2 Each procedure of specification 6.4.1 above, and changes thereto, shall be reviewed and approved by an appropriate division manager, superintendent/manager, or one of their designated direct reports prior to implementation and shall be reviewed periodically as set forth in administrative procedures. For procedures which implement offsite environmental, technical, and laboratory activities, the above review and approval may be performed by the General Manager, Environmental Services or designee.

6.4.3 Temporary changes to procedures of Specification 6.4.1 above may be made provided:

- a. The intent of the original procedure is not altered;
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operators license on the affected unit; and
- c. The change is approved by an appropriate division manager, superintendent/manager, or one of their designated direct reports within 14 days of implementation.

6.4-2

OCONEE UNITS 1,2,3

Amendment No.	(Unit 1)
Amendment No.	(Unit 2)
Amendment No.	(Unit 3)

6.4.4

The following programs shall be established, implemented, and maintained:

- a. A respiratory protective program approved by the Commission shall be in force.
- b. Administrative procedures shall be developed and implemented to limit the working hours of station staff who perform safety-related functions, e.g.; senior reactor operators, reactor operators, nuclear equipment operators, and certain maintenance personnel.

Any deviations from the above procedures shall be authorized by the Station Manager (or designee) in accordance with established procedures and with documentation of the basis for granting the deviation. Individual overtime shall be periodically reviewed to assure that excessive hours have not been worked. Routine deviation from the above guidelines is not authorized.

- c. The station shall have a program that ensures the capability to obtain and analyze reactor coolant and containment atmosphere samples under accident conditions which includes training of personnel, procedures for sampling and analysis, and provisions for testing and required maintenance of sampling and analysis equipment.
- d. The station shall have a program that ensures the capability to collect and analyze or measure representative samples of radioactive iodines and particulates in plant gaseous effluents during and following an accident which includes training of personnel, procedures for sampling and analysis, and provisions for testing and required maintenance of sampling and analysis equipment.

6.4-3

OCONEE UNITS 1,2,3

Amendment No.	(Unit 1)
Amendment No.	(Unit 2)
Amendment No.	(Unit 3)

- e. The station shall have a program conforming with 10 CFR 50.36a for the control of radioactive effluents and for maintaining the doses to MEMBERS OF THE PUBLIC from radioactive effluents as low as reasonably achievable. The program (1) shall be contained in FSAR Chapter 16, (2) shall be implemented by operating procedures, and (3) shall include remedial actions to be taken whenever the program limits are exceeded. The program shall include the following elements:
1. Limitations on the operability of radioactive liquid and gaseous monitoring instrumentation including surveillance tests and set-point determination in accordance with the methodology in the ODCM,
 2. Limitations on the concentrations of radioactive material released in liquid effluents to UNRESTRICTED AREAS conforming to 10 CFR Part 20.1001-20.2401, Appendix B, Table 2, Column 2,
 3. Monitoring, sampling, and analysis of radioactive liquid and gaseous effluents in accordance with 10 CFR 20.1302 and with the methodology and parameters in the ODCM.
 4. Limitations on the annual and quarterly doses or dose commitment to a MEMBER OF THE PUBLIC from radioactive materials in liquid effluents released from each unit to UNRESTRICTED AREAS conforming to Appendix I to 10 CFR Part 50,
 5. Determination of cumulative and projected dose contributions from radioactive effluents for the current calendar quarter and current calendar year in accordance with the methodology and parameters in the ODCM at least every 31 days.

6.4-4

OCONEE UNITS 1,2,3

Amendment No.	(Unit 1)
Amendment No.	(Unit 2)
Amendment No.	(Unit 3)

6. Limitations on the operability and use of the liquid and gaseous effluent treatment systems to ensure that the appropriate portions of these systems are used to reduce releases of radioactivity when the projected doses in a 31-day period would exceed 2 percent of the guidelines for the annual dose or dose commitment conforming to Appendix I to 10 CFR Part 50 as clarified by FSAR Chapter 16.
7. Limitations on the dose rate resulting from radioactive material released in gaseous effluents from the site to areas at or beyond the SITE BOUNDARY shall be limited to the following:
 - a. For noble gases: Less than or equal to a dose rate of 500 mrem/yr to the total body and less than or equal to a dose rate of 3000 mrem/yr to the skin, and
 - b. For Iodine-131, for Iodine-133, for tritium, and for all radionuclides in particulate form with half-lives greater than 8 days; Less than or equal to a dose rate of 1500 mrem/yr to any organ.
8. Limitations on the annual and quarterly air doses resulting from noble gases released in gaseous effluents from each unit to areas beyond the SITE BOUNDARY conforming to Appendix I to 10 CFR Part 50,
9. Limitations on the annual and quarterly doses to a MEMBER OF THE PUBLIC from Iodine-131, Iodine-133, tritium, and all radionuclides in particulate form with half-lives greater than 8 days in gaseous effluents released from each unit to areas beyond the SITE BOUNDARY conforming to Appendix I to 10 CFR Part 50; and,
10. Limitations on the annual dose or dose commitment to any MEMBER OF THE PUBLIC due to releases of radioactivity and to radiation from uranium fuel cycle sources conforming to 40 CFR Part 190.

6.4-5

OCONEE UNITS 1,2,3

Amendment No.	(Unit 1)
Amendment No.	(Unit 2)
Amendment No.	(Unit 3)

- f. The station shall have a program to monitor the radiation and radionuclides in the environs of the plant. The program shall provide (1) representative measurements of radioactivity in the highest potential exposure pathways, and (2) verification of the accuracy of the effluent monitoring program and modeling of environmental exposure pathways. The program shall (1) be contained in FSAR Chapter 16, (2) conform to the guidance of Appendix I to 10 CFR Part 50, and (3) include the following:
1. Monitoring, sampling, analysis, and reporting of radiation and radionuclides in the environment in accordance with the methodology and parameters in the ODCM;
 2. A Land Use Census to ensure that changes in the use of areas at and beyond the SITE BOUNDARY are identified and that modifications to the monitoring program are made if required by the results of this census; and,
 3. Participation in an Interlaboratory Comparison Program to ensure that independent checks on the precision and accuracy of the measurements of radioactive materials in environmental sample matrices are performed as part of the quality assurance program for environmental monitoring.

6.4-6

OCONEE UNITS 1,2,3

Amendment No.	(Unit 1)
Amendment No.	(Unit 2)
Amendment No.	(Unit 3)

6.5 STATION OPERATING RECORDS

Specification

6.5.1 The following records shall be prepared and permanently retained in a manner convenient for review:

- a. Records of modifications to the station as described in the FSAR.
- b. Special nuclear material physical inventory records.
- c. Special nuclear material isotopic inventory records.
- d. Radiation monitoring records, including records of radiation and contamination surveys.
- e. Records of off-site environmental surveys.
- f. Personnel radiation exposure records as required by 10CFR20.
- g. Records of radioactive releases and waste disposal.
- h. Records of reactor coolant system in-service inspections.
- i. Preoperational testing records.
- j. Records of special reactor tests or experiments.
- k. Records of changes safety-related operating procedures.
- l. Records for Environmental Qualification which are covered under the provisions of paragraph 6.7.
- m. Records of the seal service lives of hydraulic snubbers.
- n. Records of reviews performed for changes made to the ODCM and Process Control Program.
- o. Records of meetings of the NSRB and reports required by Specification 6.1.2.1k.

6.5-1

OCONEE UNITS 1,2,3

Amendment No.	(Unit 1)
Amendment No.	(Unit 2)
Amendment No.	(Unit 3)

6.5.2

The following records shall be prepared and retained for a minimum of six (6) years in a manner convenient for review:

- a. Switchboard Record.
- b. Reactor Operations Logbook.
- c. Shift Supervisor Logbook.
- d. Maintenance histories for station safety-related structures, systems and components.
- e. Record of safety-related inspections, other than reactor coolant system in-service inspections.
- f. Records of reportable events.
- g. Periodic testing records and records of other periodic checks, calibrations, etc. performed in accordance with surveillance requirements for safety-related parameters, structures, systems and components.
- h. By-product material inventory records.
- i. Training records.
- j. Test results, in units of microcuries, for leak tests performed pursuant to Specification 4.16.
- k. Radioactive liquid effluent, gaseous effluent, and gaseous process monitoring instrumentation alarm/trip setpoints.

6.5-2

OCONEE UNITS 1,2,3

Amendment No.	(Unit 1)
Amendment No.	(Unit 2)
Amendment No.	(Unit 3)

ATTACHMENT VII

DUKE POWER SELECTED LICENSEE COMMITMENTS 16.13-2 AND 16.13-3

16.13 CONDUCT OF OPERATIONS

16.13-2 TECHNICAL REVIEW AND CONTROL

COMMITMENT

A Technical Review and Control Program covering the preparation, review, and approval of documents important to station operation shall be established and maintained for the site.

APPLICABILITY:

This commitment is applicable at all times and applies to the review and control activities described in items a through k as listed below. Personnel performing the preparation, review, and approval activities covered by this commitment shall meet or exceed the qualifications of ANSI N18.1-1971 (the conformance status for this standard is as listed in Table 17-1 of the Duke Power Topical Report, Quality Assurance Program, Duke-1-A).

- a. The preparation, review, and approval of station procedures shall be done in accordance with station Technical Specifications.
- b. Proposed modifications shall be designed and reviewed in accordance with station Technical Specifications. Proposed modifications to nuclear safety related structures, systems and components shall be approved prior to implementation by the Station Manager or the Manager of Engineering; or for the Station Manager by the Mechanical Maintenance Superintendent, the Operations Superintendent, the I and E/ Maintenance Support Superintendent, or the Work Control Superintendent, as previously designated by the Station Manager.
- c. Individuals responsible for reviews performed in accordance with Items a and b shall be members of the supervisory staff assigned to the site, be previously designated by the Site Vice President as a Qualified Reviewer, and successfully complete the site Qualified Reviewer training program. Review of environmental radiological analysis procedures, shall be performed by the General Manager, Environmental Services or a designee. Each such review shall include a determination of whether or not additional, cross-disciplinary, review is necessary. If deemed necessary, such review shall be performed by the appropriately designated site review personnel.

- d. Proposed changes to the station Technical Specifications shall be prepared in accordance with station Technical Specifications. Each proposed Technical Specification change shall be reviewed by the Plant Operations Review Committee (PORC) and the Nuclear Safety Review Board (NSRB) prior to submittal to the Nuclear Regulatory Commission. Proposed changes to the Technical Specifications shall be approved by the Station Manager, or for the Station Manager by a designated manager or company officer. Technical Specifications submittal cover letters shall be signed by an officer of Duke Power Company.
- e. Proposed tests and experiments which affect station nuclear safety and are not addressed in the FSAR or Technical Specifications shall be reviewed by the Plant Operations Review Committee (PORC).
- f. Incidents reportable pursuant to station Technical Specifications and all violations of Technical Specifications shall be investigated and a report prepared which evaluates the occurrence and which provides recommendations to prevent recurrence. Such reports shall be approved by the Manager, Safety Assurance and provided to the Site Vice President and the Plant Operations Review Committee (PORC).
- g. The Manager, Safety Assurance shall assure the performance of special reviews and investigations, and the preparation and submittal of reports thereon, as requested by the Site Vice President. Such reports shall be provided to the Plant Operations Review Committee (PORC).
- h. The Manager, Safety Assurance shall assure the performance of a review by a knowledgeable individual/organization of every unplanned onsite release of radioactive material to the environs, including the preparation and forwarding of reports covering evaluation, recommendations, and disposition of the corrective action to prevent recurrence to the Site Vice President, and to the Plant Operations Review Committee (PORC).
- i. The Manager, Safety Assurance shall assure the performance of a review by a knowledgeable individual/organization of changes to the Process Control Program, Offsite Dose Calculation Manual (ODCM), and Radwaste Treatment Systems.

- j. The Manager, Safety Assurance shall ensure the performance of a review by a knowledgeable individual/organization of the Fire Protection program and implementing procedures and submittal of recommended changes to the Director, Organization Effectiveness Services.
- k. Reports documenting each of the activities performed under this commitment shall be maintained. Copies shall be provided to the NSRB.

REMEDIAL ACTION: Not Applicable

BASIS:

- a. The requirements contained in this selected licensee commitment were relocated from the McGuire Technical Specifications with the approval of the U. S. Nuclear Regulatory Commission. Changes to this SLC shall be considered a change in an NRC commitment and shall be made only in accordance with the approved Compliance Manual Procedure for the Control of Selected Licensee Commitments and by use of the 10CFR50.59 evaluation process.
- b. This SLC implements the review requirements of ANSI N18.7-1976/ANS-3.2 as referenced in the Duke Power Company Topical Report, Quality Assurance Program, Duke-1-A.

TESTING REQUIREMENTS: Not Applicable

REFERENCES:

- 1. ANSI N18.1-1971, Selection and Training of Nuclear Power Plant Personnel
- 1. ANSI N18.7-1976/ANS-3.2, Administrative Controls and Quality Assurance for the Operational Phase of Nuclear Power Plants
- 2. Compliance Manual Procedure for the Control of Selected Licensee Commitments
- 3. Nuclear System Directive 209, 10 CFR 50.59 Evaluations
- 4. 10 CFR 50.59
- 5. Nuclear System Directive 703, Administrative Instructions for Station Procedures

16.13 CONDUCT OF OPERATIONS

16.13-3 PLANT OPERATIONS REVIEW COMMITTEE

COMMITMENT

A Plant Operations Review Committee (PORC) shall be established and maintained for the site. The PORC shall be composed of the Manager of Safety Assurance, the Station Manager and his/her direct reports most responsible for station operation and maintenance, the Manager of Engineering and his/her direct reports most responsible for engineering support of station operation and maintenance, or designated alternates. The PORC Chairperson, members, and alternate members shall be qualified in accordance with ANSI N18.1-1971 and be appointed by the Site Vice President. The quorum necessary for conducting the PORC functions shall consist of the Chairperson, or his/her designated alternate, and at least three other PORC members including alternates.

Reports of reviews encompassed by this Selected Licensee Commitment shall be prepared and forwarded to the Site Vice President and the Nuclear Safety Review Board.

APPLICABILITY:

- a. The PORC shall be responsible for reviewing the following prior to final approval:
 - 1. All proposed tests and experiments which affect station nuclear safety and are not addressed in the FSAR or Technical Specifications;
 - 2. Operability evaluations resulting in a Justification for Continued Operation and a proposal for discretionary enforcement;
 - 3. Operability evaluations resulting in the decision that affected systems, structure or components are OPERABLE but degraded; and
 - 4. All proposed changes to the station Technical Specifications, Bases, or Facility Operating License.
- b. The PORC shall be responsible for reviewing the effectiveness of corrective actions for:
 - 1. Licensee Event Reports and Special Reports made to the NRC;
 - 2. Violations of Technical Specifications;