

# ACCELERATED DISTRIBUTION DEMONSTRATION SYSTEM

## REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 9101220358      DOC. DATE: 91/01/15      NOTARIZED: NO      DOCKET #  
 FACIL: 50-269 Oconee Nuclear Station, Unit 1, Duke Power Co.      05000269  
 50-270 Oconee Nuclear Station, Unit 2, Duke Power Co.      05000270  
 50-287 Oconee Nuclear Station, Unit 3, Duke Power Co.      05000287

AUTH. NAME      AUTHOR AFFILIATION  
 TUCKMAN, M.S.      Duke Power Co.  
 RECIP. NAME      RECIPIENT AFFILIATION  
                  Document Control Branch (Document Control Desk)

SUBJECT: Provides confirmation of status of investigation into NRC  
 Bulletin 89-002, "Stress Corrosion Cracking of High-Hardness  
 Type 410 Stainless Steel Internal Preloaded Bolting in  
 Anchor Darling Model S350W Swing Check Valves...."

DISTRIBUTION CODE: IE32D      COPIES RECEIVED: LTR 1 ENCL 0 SIZE: 2  
 TITLE: Bulletin Response 89-02 re Cracking of High-Hardness Type 410 Bolting

### NOTES:

	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL
	PD2-3 LA	1 0	PD2-3 PD	1 1
	WIENS, L	1 1		
INTERNAL:	AEOD/DOA	1 1	AEOD/DSP/TPAB	1 1
	NRR/DET/ECMB9H	2 2	NRR/DET/EMEB 7E	1 1
	NRR/DOEA/OEAB11	1 1	NRR/DOEA/OGCB11	1 1
	NRR/DREP/PEPB9D	1 1	NRR/DST 8E2	1 1
	NRR/EMCB/HERMAN	1 1	NRR/MCLELLAN, T	1 1
	NRR/PMAS/ILRB12	1 1	NRR/ROOD, H	1 1
	NRR/SELLERS, D.	1 1	REG FILE 02	1 1
	RES/DSIR/EIB	1 1	RGN2 FILE 01	1 1
EXTERNAL:	NRC PDR	1 1	NSIC	1 1

NOTE TO ALL "RIDS" RECIPIENTS:

PLEASE HELP US TO REDUCE WASTE! CONTACT THE DOCUMENT CONTROL DESK,  
 ROOM P1-37 (EXT. 20079) TO ELIMINATE YOUR NAME FROM DISTRIBUTION  
 LISTS FOR DOCUMENTS YOU DON'T NEED!

TOTAL NUMBER OF COPIES REQUIRED: LTTR 22 ENCL 21

*AD*  
*[Signature]*



**DUKE POWER**

January 15, 1991

U. S. Nuclear Regulatory Commission  
Attention: Document Control Desk  
Washington, DC 20555

Subject: Oconee Nuclear Station, Unit 2  
Docket Nos. 50-269, -270, and -287  
NRC Bulletin No. 89-02  
Stress Corrosion Cracking of High-Hardness Type  
410 Stainless Steel Internal Preloaded Bolting  
In Anchor Darling Model S350W Swing Check Valves  
or Valves of Similar Design  
Unit 2 Final Response

Gentlemen:

Mr. C. E. Rossi's (NRC/ONRR) July 19, 1989 letter (Bulletin 89-02) concerned stress corrosion cracking of high-hardness type 410 Stainless Steel internal preloaded bolting in Anchor Darling Model S350W swing check valves or valves of similar design. The purpose of this bulletin was to request identification, disassembly, and inspection of certain types of swing check valves which may contain type 410 Stainless Steel (SS) bolting material; and if the Type 410 SS bolting material is of sufficiently high hardness that it is susceptible to stress corrosion cracking (SCC), or has failed, to take appropriate actions.

Pursuant to Reporting Requirement No. 2 of the Bulletin, this letter provides confirmation of the status of our investigation into this bulletin at Oconee Nuclear Station Unit 2.

Although Oconee Nuclear Station Unit 2 does not have any Anchor Darling Model S350 swing check valves, a design review identified nine valves of a similar design. These valves were all manufactured by Velan and are located as follows:

<u>Application</u>	<u>Location and Size</u>
2 HP-105, 109, 113	High Pressure Injection Pump discharge, 3 inches
2 LP-55, 57	Decay heat Cooler discharge to High Pressure Injection pump suction, 3 inches
2 BS-7, 9	Decay heat Cooler discharge to Building Spray pump suction, 3 inches
2 HP-188, 194	High Pressure Injection Pump discharge, 4 inches

9101220358 910115  
PDR ADDOCK 05000269  
Q PDR

EB2  
110

U. S. Nuclear Regulatory Commission  
January 15, 1991  
Page 2

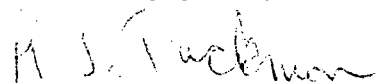
Seven of the nine applicable (i.e., subject to the Bulletin) check valves were disassembled and inspected during the past Unit 2 refueling outage (which ended on October 27, 1990). The internal 410 Stainless Steel fasteners were dye penetrant tested using ASME Section XI criteria. Although no cracks or unacceptable indications were found, the fasteners were replaced, with A 193 Grade B8 material with recommendation from the vendor.

The other two check valves, 2 HP-188 and 194, were not disassembled during the outage since the station has included these application in their valve replacement plan. No further action regarding this Bulletin is planned for Unit 2.

In conclusion, the results of our completed investigation revealed no broken or cracked 410 Stainless Steel Fasteners on Unit 2 check valve internals inspected. An appropriate report regarding Oconee Unit 3 will be submitted following inspections at the upcoming refueling outage (currently scheduled to begin February 13, 1991) in accordance with the Bulletin's requirements.

I declare under penalty of perjury that the statements set forth herein are true and correct to the best of my knowledge. Should there be any questions concerning this matter or if additional information is required, please advise.

Very truly yours,



M. S. Tuckman

SGB2/td

cc: Mr. S. D. Ebnetter, Regional Administrator  
U. S. Nuclear Regulatory Commission  
Region II  
101 Marietta St., NW, Suite 2900  
Atlanta, GA 30329

Mr. L. A. Wiens, Project Manager  
Office of Nuclear Reactor Regulation  
U. S. Nuclear Regulatory Commission  
Washington, DC 20555

Mr. P. H. Skinner  
NRC Resident Inspector  
Oconee Nuclear Station