



Steven D. Capps
Vice President
McGuire Nuclear Station

Duke Energy
MG01VP | 12700 Hagers Ferry Road
Huntersville, NC 28078

o: 980.875.4805
f: 980.875.4809
Steven.Capps@duke-energy.com

10 CFR 50.4

August 26, 2015
Serial: MNS-15-060

ATTN: Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Duke Energy Carolina, LLC (Duke Energy)
McGuire Nuclear Station (MNS), Units 1 and 2
Docket Nos. 50-369 and 50-370
Renewed License Nos. NPF-9 and NPF-17

Subject: Fifth Six-Month Status Report in Response to March 12, 2012, Commission Order
Modifying Licenses With Regard to Requirements for Mitigation Strategies for
Beyond-Design-Basis External Events (Order Number EA-12-049)

References:

1. Nuclear Regulatory Commission (NRC) Order Number EA-12-049, Order Modifying Licensees With Regard to Requirements for Mitigation Strategies for Beyond-Design-Basis External Events, Revision 0, dated March 12, 2012 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML12054A735).
2. NRC Interim Staff Guidance JLD-ISG-2012-01, Compliance with Order EA-12-049, Order Modifying Licenses with Regard to Requirements for Mitigation strategies for Beyond-Design-Basis External Events, Revision 0, dated August 29, 2012 (ADAMS Accession No. ML12229A174).
3. NEI 12-06, Diverse and Flexible Coping Strategies (FLEX) Implementation Guide, Revision 0-A, dated August 2012 (ADAMS Accession No. ML12242A378).
4. Duke Energy's Initial Status Report in Response to March 12, 2012, Commission Order Modifying Licenses with Regard To Requirements for Mitigation Strategies for Beyond-Design-Basis External Events (Order EA-12-049), dated October 29, 2012 (ADAMS Accession No. ML12307A023).
5. McGuire Nuclear Station Overall Integrated Plan in Response to March 12, 2012, Commission Order to Modify Licenses With Regard To Requirements for Mitigation Strategies for Beyond Design Basis External Events (Order EA-12-049), dated February 28, 2013 (ADAMS Accession No. ML13063A185).

A151
NRR

6. McGuire Nuclear Station First Six-Month Status Report in Response to March 12, 2012, Commission Order Modifying Licenses with Regard to Requirements for Mitigation Strategies for Beyond-Design-basis External Events (Order Number EA-12-049) Dated August 28, 2013 (ADAMS Accession No. ML13254A204).
7. McGuire Nuclear Station Second Six-Month Status Report in Response to March 12, 2012, Commission Order Modifying Licenses with Regard to Requirements for Mitigation Strategies for Beyond-Design-basis External Events (Order Number EA-12-049) Dated February 27, 2014 (ADAMS Accession No. ML14073A462).
8. McGuire Nuclear Station Third Six-Month Status Report in Response to March 12, 2012, Commission Order Modifying Licenses with Regard to Requirements for Mitigation Strategies for Beyond-Design-basis External Events (Order Number EA-12-049) Dated August 27, 2014 (ADAMS Accession No. ML14253A188).
9. McGuire Nuclear Station Fourth Six-Month Status Report in Response to March 12, 2012, Commission Order Modifying Licenses with Regard to Requirements for Mitigation Strategies for Beyond-Design-basis External Events (Order Number EA-12-049) Dated February 26, 2015, (ADAMS Accession No. ML15075A016).

On March 12, 2012, the Nuclear Regulatory Commission (NRC) issued Order EA-12-049 (Reference 1) to Duke Energy. Reference 1 was immediately effective and directs Duke Energy to develop, implement, and maintain guidance and strategies to maintain or restore core cooling, containment, and spent fuel pool cooling capabilities in the event of a beyond-design-basis external event. Specific requirements are outlined in Attachment 2 of Reference 1.

Reference 1 required submission of an initial status report 60 days following issuance of the final interim staff guidance (Reference 2) and an Overall Integrated Plan (OIP) pursuant to Section IV, Condition C. Reference 2 endorses industry guidance document NEI 12-06, Revision 0 (Reference 3) with clarifications and exceptions identified in Reference 2. Reference 4 provided the initial status report regarding mitigation strategies at the Oconee, McGuire and Catawba Nuclear Stations. Reference 5 provided the OIP for MNS.

Reference 1 requires submission of a status report at six-month intervals following submittal of the OIP. Reference 3 provides direction regarding the content of the status reports.

The purpose of this letter is to provide the fifth six-month status report pursuant to Section IV, Condition C.2, of Reference 1. The attached report provides an update of milestone accomplishments since the last status report, including any changes to the compliance method, schedule, or need for relief and the basis, if any.

United States Nuclear Regulatory Commission
August 26, 2015
Page 3

This letter contains no new Regulatory Commitments and no revision to existing Regulatory Commitments.

Should you have any questions regarding this submittal, please contact George Murphy at (980) 875-5715.

I declare under penalty of perjury that the foregoing is true and correct. Executed on August 26, 2015.

Sincerely,

A handwritten signature in black ink, appearing to read "SD Capps", written in a cursive style.

Steven D. Capps

Enclosure:

MNS Fifth Six-Month Status Report in Response to Order EA-12-049

xc:

V.M. McCree, Region II Administrator
U.S. Nuclear Regulatory Commission
Marquis One Tower
245 Peachtree Center Avenue NE, Suite 1200
Atlanta, Georgia 30303-1257

John Boska, Project Manager (OMB)
U.S. Nuclear Regulatory Commission
One White Flint North, Mailstop O-8G9A
11555 Rockville Pike
Rockville, MD 20852-2738

G. E. Miller, Project Manager (CNS & MNS)
U.S. Nuclear Regulatory Commission
11555 Rockville Pike
Mail Stop 8 G9A
Rockville, MD 20852-2738

J. Zeiler
NRC Senior Resident Inspector
McGuire Nuclear Station

Justin Folkwein
American Nuclear Insurers
95 Glastonbury Blvd., Suite 300
Glastonbury, CT 06033-4453

ENCLOSURE

MNS FIFTH SIX MONTH STATUS REPORT IN RESPONSE TO ORDER EA-12-049

MNS Fifth FLEX Six Month Status Report in Response to Order EA 12-049

1 Introduction

McGuire developed an Overall Integrated Plan (OIP) (Reference 1 in Section 8), documenting the diverse and flexible strategies (FLEX), in response to NRC Order EA-12-049. The OIP was submitted to the NRC on February 28, 2013. The first six-month update was provided to the NRC on August 28, 2013 (Reference 3 in Section 8). The second six-month update was provided to the NRC on February 27, 2014 (Reference 5 in Section 8). The third six-month update was provided to the NRC on August 27, 2014 (Reference 6 in Section 8). The fourth six-month update was provided to the NRC on February 26, 2015 (Reference 7 in Section 8). This enclosure provides an update of milestone accomplishments including any changes to the compliance method, schedule, or need for relief/relaxation and the basis, if any, that occurred during the period from February 1, 2015 to July 31, 2015 (hereafter referred to as "the update period").

2 Milestone Accomplishments

Unit 1 is considered in compliance with NRC Order EA-12-049 as of November 15, 2014 (Reference 13 in Section 8).

The following Unit 2 milestones were completed during the update period:

- Complete Engineering Change Package and Plan Work Orders

3 Milestone Schedule Status

The following provides an update to Attachment 2 of the Overall Integrated Plan. It provides the activity status of each item, and whether the expected completion date has changed. The dates are planning dates subject to change as design and implementation details are developed.

Unit 1 Milestones	Target Completion Date	Activity Status	Revised Target Completion Date
Complete Engineering Change Package and Plan Work Orders	03/20/2014	Complete	NA
Receive all portable FLEX equipment	05/20/2014	Complete	NA
Transmit results of Phase 2 staffing study	05/20/2014	Complete	NA
Complete all FSGs, AOPs, EOPs, OPs	10/31/2014	Complete	NA
Complete training for all FSGs, AOPs, EOPs, OPs	09/20/2014	Complete	NA
Complete all Maintenance Procedures	09/20/2014	Complete	NA
Begin Outage (1EOC23)	Fall 2014	Complete	NA
Storage Plan - Reasonable Protection Facilities Complete	Fall 2014	Complete	NA
Regional Response Center in place	10/19/2014	Complete	NA

Unit 2 Milestones	Target Completion Date	Activity Status	Revised Target Completion Date
Complete Engineering Change Package and Plan Work Orders	03/20/2015	Complete	Date Not Revised
Receive all portable FLEX equipment	05/20/2015	Complete	NA
Transmit results of Phase 2 staffing study	05/20/2015	Complete	NA
Complete all FSGs, AOPs, EOPs, OPs	06/20/2015	Started	09/20/2015
Complete training for all FSGs, AOPs, EOPs, OPs	09/20/2015	Started	Date Not Revised
Complete all Maintenance Procedures	09/20/2015	Started	Date Not Revised
Begin Outage (2EOC23)	Fall 2015	Not Started	Date Not Revised

Unit 2 Milestones	Target Completion Date	Activity Status	Revised Target Completion Date
Storage Plan - Reasonable Protection Facilities Complete	Fall 2014	Complete*	Summer 2015
Regional Response Center in place	10/11/2015	Complete	NA

* Facility construction (three buildings) is complete. Unit 1 compliance was met with the completion and loading of the first two buildings in Fall 2014. Unit 2 compliance with Order EA-12-049 will be met with the loading of the remaining building.

4 Changes to Compliance Method

- 1) Change: The OIP for McGuire (Reference 1) identified that a modification would be implemented to provide assured air to the Turbine Driven Auxiliary Feed Water Pump (TDAFWP) flow control valves and the Steam Generator (SG) Pressure Operated Relief Valves (PORVs) during Extended Loss of AC Power (ELAP) Phase 1 response to allow these valves to be operated from the Control Room until a source of FLEX makeup air can be provided in Phase 2. In lieu of this, manual action is currently credited for the Unit 1 Phase 1 ELAP response, as identified in the McGuire Third Six-month update (Reference 6). The ELAP Phase 1 Assured Air modification will be completed during the Unit 2 RFO in fall 2015 for both Units, eliminating the associated temporary manual actions currently in place for the Unit 1, Phase 1 response.

Justification: Upon implementation of the Mitigating Strategies Order on Unit 2, the air supply to the TDAFWP flow control valves and SG PORVs will be assured via completion of the plant modification, allowing remote control of these components on both units from the Control Room per the McGuire OIP.

Documentation: Open Item #1

- 2) Change: The OIP for McGuire (Reference 1), and ISE Confirmatory Item 3.2.4.10.A, identified that the vital batteries would last for more than 24 hours after the onset of an ELAP event (assuming dc load stripping per ELAP response procedure). A revision to the original analysis that incorporated recent vendor extended discharge test data shows the vital batteries will last a minimum of 18 hours when starting the ELAP event at only 80% capacity (limited as required per existing guidance). Response procedures will be revised accordingly to reflect the need for recovery of the vital battery chargers within 18 hours in lieu of the currently stated 24 hours.

Justification: ELAP response procedural guidance (EP/1(2)/A/5000/ECA-0.0) prioritizes Phase 2 deployment of the FLEX electrical distribution system (FSG-20) and the subsequent recovery of the vital battery chargers (FSG-4). The time margin between the calculated station battery run-time for Phase 1 response and the expected deployment time for FLEX equipment to repower the battery chargers is approximately 3-4 hours.

The McGuire vital batteries are procedurally maintained above the required assumed capacity of 80%.

Documentation: Open Item #52

5 Need for Relief/Relaxation and Basis for the Relief/Relaxation

McGuire expects to comply with the Order implementation date and no relief/relaxation is required at this time contingent on the following:

- Resolution of issues regarding RCP #1 seal leak-off flow rates and leak-off piping integrity, as identified in NSAL 14-1, Revision 1.

6 Open Items

The following sections provide tables summarizing the status of the Open Items:

- Section 6.a provides the open items identified in the original OIP submitted on February 28, 2013.
- Section 6.b provides a list of open items that were added after February 28, 2013.
- Section 6.c provides a list of open items related to the Interim Staff Evaluation (ISE) received on January 16, 2014.
- Section 6.d provides a list of open items related to the NRC Audit Report received on October 9, 2014.

Unit 2 designators have been added to the Open Item descriptions where appropriate since Unit 1 is in compliance with the Order.

Section 6.a. Open Items Documented in the Overall Integrated Plan.

Note: items marked with an asterisk (*) are designed and awaiting implementation.

Note: items marked with a double asterisk (**) have been completed during this update period.

	Overall Integrated Plan Open Item	Status
1	Implement (Units 1 and 2) plant modification: Assured Air to the TDAFWP FCVs and SG PORVs (Current Unit 1 configuration is based on 3rd Six Month Update Change #2)	Started*
2	Implement (Unit 2) plant modification: Assured Water Supply to the TDAFWPs	Started*
3	Implement (Unit 2) plant modification: SFP Wide-Range Level Instrumentation	Started*
4	Implement plant modification: UHF Communication System Upgrades	Completed
5	Implement (Unit 2) plant modification: Process Connections	Started*
6	Implement (Unit 2) plant modification: Permanent Connections for Portable Electrical Equipment	Started*
7	Implement plant modification: FLEX Storage Facilities	Completed
8	Implement (Unit 2) plant modification: Submersible Ground Water Sump Pump	Completed
9	Implement (Unit 2) plant modification: FLEX Strategy Implementation	Started*
10	Implement (Unit 2) plant modification: "B" RN to CA Pump Suction Re-route	Started*
11	Implement plant modification: Install Emergency Hardhat Light Storage Boxes and Hardhat Hooks	Completed
12	A staffing Phase 2 study will be performed in accordance with NEI 12-01 to verify that all actions can be taken in accordance with the timeline. Time constraints shown in Attachment 1A will be validated to be reasonable as the strategy is finalized.	Completed

13	An analysis was performed in Duke Energy Calculation MCC-1223.31-00-0012 that indicates that flooding will not occur for at least 48 hours. This analysis will be revised to demonstrate that this time remains unaffected even if potential sources of water from Auxiliary Building or Turbine Building flooding are considered.	Completed
14	A calculation will be performed to demonstrate that sufficient negative reactivity can be added through use of a pump and a reactor coolant system vent path to achieve xenon free cool down in accordance with the PWROG FSG guidelines.	Completed
15	Complete vital battery area room hydrogen accumulation calculation (see open item #48 for heat up calculation) to determine if portable fans or HVAC units may be required, and timeframe for deployment.	Completed. Description of this Open Item revised per Open Item 48
16	Complete a calculation to determine when elevated interior doghouse temperatures adversely impact the FLEX strategy and to evaluate methods for mitigation.	Completed
17	An evaluation will be performed to determine how long raw water can be used to supply SGs without excessively affecting SG capability to remove heat and provide steam to the TDAFW pump. This will help determine when Phase 3 equipment may be needed to assist in providing cleaner water sources.	Completed
18	MNS will evaluate the need to provide freeze protection for critical instrumentation and exposed FLEX connections.	Completed
19	Methods will be initiated to circulate and cool air in lower containment sub compartments to prevent any adverse impact on critical instrumentation. The response time is based on engineering judgment and will be confirmed by analysis.	Completed

20	Deployment routes will be established and are expected to be utilized to transport FLEX equipment to the deployment areas. The identified paths and deployment areas will be accessible during all modes of operation. This deployment strategy will be included within an administrative program in order to keep pathways clear or actions to clear the pathways.	Completed
21	MNS will implement programmatic controls in accordance with NEI 12-06. Procedures and guidelines will be reviewed and revised and/or generated as required to address additional programmatic controls as a result of FLEX requirements.	Completed
22	Equipment associated with FLEX mitigation strategies will be procured as commercial equipment with design, storage, maintenance, testing, and configuration control in accordance with NEI 12-06 Section 11.1.	Completed
23	Installed structures, systems and components pursuant to 10 CFR 50.63(a) will continue to meet augmented guidelines of Regulatory Guide (RG) 1.155, Station Blackout.	Completed
24	The unavailability of equipment and applicable connections that directly perform a FLEX mitigation strategy will be managed using plant equipment control guidelines developed in accordance with NEI 12-06 Section 11.5.	Completed
25	Programs and processes will be established (Unit 2) to ensure that personnel proficiency in the mitigation of beyond-design-basis events is developed and maintained in accordance with NEI 12-06 Section 11.6.	Completed
26	The FLEX strategies and basis will be maintained in overall FLEX basis documents.	Completed

27	Existing plant configuration control documents will be modified to ensure that changes to the plant design, physical plant layout, roads, buildings, and miscellaneous structures will not adversely impact the approved FLEX strategies in accordance with NEI 12-06 Section 11.8.	Completed
28	Training will be initiated through the Systematic Approach to Training (SAT) process. Training will be developed and provided to all involved plant personnel based on any procedural changes or new procedures developed to address and identify FLEX activities. Applicable training will be completed prior to the implementation of FLEX.	Unit 1 Completed Unit 2 Started
29	The industry will establish two Regional Response Centers (RRCs) to support utilities during beyond-design-basis events.	Completed
30	The N+1 FLEX storage facilities will be designed in accordance with ASCE 7-10, Minimum Design Loads for Buildings and Other Structures. The FLEX storage facilities will be designed in accordance with ASCE 7-10, to withstand the maximum anticipated hurricane and tornado winds as outlined in NEI 12-06. The FLEX buildings will be located in accordance with NEI 12-06 Section 7.3.1 to prevent damage to more than one of the three facilities due to tornado missiles.	Completed
31	FLEX storage facilities will be located above any potential site flood level, and/or the effects of localized flooding will be evaluated in the FLEX facility design and equipment deployment.	Completed
32	Debris removal/remediation equipment and procedures will be provided to support FLEX equipment deployment.	Completed
33	Snow and ice removal/remediation equipment and procedures will be provided to support FLEX equipment deployment.	Completed

34	FLEX equipment will be capable of operation under extreme temperatures, and suitably maintained to ensure standby readiness. FLEX storage facilities will be designed to accommodate maximum snow and ice loading. FLEX storage facilities will be vented to maintain acceptable temperature.	Completed
35	An evaluation will be performed to ensure that the 300 gpm pump is capable of adequate flow and pressure to support feed and bleed core cooling in typical Mode 5 and Mode 6 configurations.	Completed
36	Since the overall FLEX strategy is aimed at preventing core damage, the engineering change process will drive out an evaluation to prioritize operator actions associated with containment isolation as time allows. For example, the containment isolations to the Containment Ventilation Unit Condensate Drain Tank (VUCDT) will be closed first since this path connects containment atmosphere directly to the Auxiliary Building.	Completed
37	An analysis will be performed to validate that containment spray for temperature/pressure control is not required over the long term. If the long term containment analysis determines that containment temperature and/or pressure will reach unacceptable levels over the long term, connections will be installed for containment spray mitigating strategies and will be used with the portable diesel driven pumps to supply water from the UHS to the connections located in the Auxiliary Building.	Completed
38	Fans in containment that circulate air will be restored as required to cool the cubicle areas and to prevent the increase in temperature from having an adverse impact on essential instrumentation. The engineering change process will drive out an evaluation to determine the appropriate timing of these actions.	Completed

39	Evaluate other long term strategies for cooling containment such as circulating the air volume in the annulus.	Completed
40	<p>In order to prevent or mitigate this inevitability [i.e., vital battery depletion]:</p> <ol style="list-style-type: none"> 1. Portable power distribution equipment will be used to repower the vital batteries, or 2. An alternate strategy to deploy portable generators and cables will be developed to directly reestablish power to the power supplies in the 7300 cabinets thereby re-powering the instrumentation loops, or 3. An alternate strategy to utilize handheld instruments will be developed to tap into the instrument loops locally to monitor essential parameters. 	Completed
41	An analysis was completed to demonstrate that adequate control room cooling would be available if action was taken to open various doors at around 2 hours after the event occurs. This action will be incorporated into procedures.	Completed
42	An evaluation will be performed to determine diesel fuel, gasoline and two-cycle oil requirements for Phase 1 and Phase 2 portable equipment.	Completed
43	Analysis will be performed to determine commodities requirements.	Completed
44	Evaluate SFP to ensure predicted makeup water dilution rates in the Spent Fuel Pools for the coping strategies described herein are bounded.	Completed
45	Analysis will be performed to determine radiation protection equipment requirements.	Completed
46	These instruments [i.e., Essential Instrumentation and Vital I&C] will be used in the FSGs in accordance with the PWROG guidance. Setpoint uncertainty analyses will be developed in accordance with PWROG guidance.	Completed

47	A portable battery pack with inverter is also being evaluated for use. The battery pack would be rapidly deployed for use to close the CLA motor-operated valves (MOVs) outside of containment penetration without reliance on the valve's limit or torque switches or the valve's relay circuitry.	Completed
----	---	-----------

Section 6.b. Open Items added after February 28, 2013

	First Six Month Update New Open Items	Status
48	Complete vital battery area room heatup calculation to determine if portable fans or HVAC units may be required, and the timeframe for deployment. (Reference open item 15)	Completed
49	To facilitate the Phase 2 FLEX mitigation response, the high and low pressure make-up pumps for the Reactor Coolant System for each Unit are currently planned to be pre-staged in the Auxiliary Building, near the FLEX connection points. As these areas are within a Category I seismic structure, no spares will be stored in the FLEX buildings for deployment, since "N" pieces of FLEX equipment are already deployed and protected. Per NEI 12-06 guidance, this equipment will be evaluated for seismic ruggedness.	Completed
50	Access to connection points for the Phase 2 FLEX mitigation response will be through the north end of the McGuire Auxiliary Building, an area which is not seismically designed. Section 5.3.2 part 2 of the NEI 12-06 Guidance Document states: "At least one connection point of FLEX equipment will only require access through seismically robust structures. This includes both the connection point and any areas that plant operators will have to access to deploy or control the capability." Evaluate Auxiliary Building access points for possible qualification of seismic ruggedness.	Completed

51	Reference OI #30. NEI 12-06 Guidance clarification documented in NEI FLEX Guidance Inquiry no. 2013-07 identifies that FLEX buildings designed for hurricane wind speeds and using diverse locations per Section 7.3.1 part (c) are protected from both tornado and hurricane generated missiles. Validate strategy with NEI.	Completed
	Second Six Month Update New Open Items	Status
52	McGuire to confirm that the FLEX strategy station battery run-time was calculated in accordance with the IEEE-485 methodology using manufacturer discharge test data applicable to the licensee's FLEX strategy as outlined in the NEI white paper on Extended Battery Duty Cycles. The detailed licensee calculations, supporting vendor discharge test data, FLEX strategy battery load profile, and other inputs/initial conditions required by IEEE-485 will be available on the licensee's web portal for documents and calculations. The time margin between the calculated station battery run-time for the FLEX strategy and the expected deployment time for FLEX equipment to supply the dc loads is 4 hours.	Completed
53	See ISE Open Item 3.2.1.8.A. The PWROG submitted to NRC a position paper, dated August 15, 2013, which provides test data regarding boric acid mixing under single-phase natural circulation conditions and outlined applicability conditions intended to ensure that boric acid addition and mixing would occur under conditions similar to those for which boric acid mixing data is available. McGuire to confirm its intent to abide by the generic approach discussed above, and address the clarifications in the NRC endorsement letter dated January 8, 2014.	Completed
54	NEI submitted the EPRI Report 3002000623 entitled "Nuclear Maintenance Applications Center: Preventative Maintenance Basis for FLEX Equipment" on October 23, 2013. McGuire to confirm its intent to abide by the generic approach discussed above in	Completed

	developing FLEX equipment maintenance and testing programs.	
55	See ISE Confirmatory Item 3.2.1.7.A. McGuire to confirm that the guidance in NEI position paper, "Position Paper: Shutdown/Refueling Modes," will be followed in the Mitigation Strategies for BDBEES occurring during Shutdown/Refueling Modes.	Completed
56	<p>McGuire to confirm viability of proposed long-term ELAP Phase 3 recovery strategy:</p> <p>Modes 1-4: rely on passive ice condenser response and repowered containment/annulus ventilation fan cooling to prevent Pressurizer/Steam Generator level indication reference legs from flashing during RCS cooldown/depressurization, and realign/restart RHR system (and support systems). Manage containment pressure as necessary.</p> <p>Modes 5-6: rely on passive ice condenser response and deploy portable FLEX pump within timeframe necessary to manage RCS boration/inventory. Manage containment pressure as necessary and realign/restart RHR system (and support systems).</p>	Completed
	Third Six Month Update New Open Items	Status
57	NSAL 14-1: Upon completion of the vendor evaluation of the RCP seal leak-off flow with the installed Number 1 Seal Leak-off orifice, identify the path forward for meeting the FLEX Mitigation Strategy Analysis assumptions based on the evaluated leak-off flow.	Completed
	Fourth Six Month Update New Open Items	Status
	None.	N/A
	New Open Items added this Update	Status
	None.	N/A

Section 6.c. Interim Staff Evaluation Open and Confirmatory Items

	Interim Staff Evaluation Open Item	Status
3.2.1.8.A	<p>The PWROG submitted to NRC a position paper, dated August 15, 2013, which provides test data regarding boric acid mixing under single-phase natural circulation conditions and outlined applicability conditions intended to ensure that boric acid addition and mixing would occur under conditions similar to those for which boric acid mixing data is available.</p> <p>During the audit process, the licensee informed the NRC staff of its intent to abide by the generic approach discussed above. The licensee should address the clarifications in the NRC endorsement letter dated January 8, 2014.</p>	Completed
	Interim Staff Evaluation Confirmatory Items	Status
3.1.1.2.A	Deployment of FLEX equipment - On page 57 of its Integrated Plan, in the chart identifying Pressurized Water Reactor (PWR) Portable Equipment Phase 2, the licensee lists (9) 9×12 trailers used to store and deploy power equipment, but does not list tow vehicles. Confirm abilities to move FLEX equipment and the level of protection afforded the means to move.	Completed
3.1.1.3.A	Procedural interfaces, seismic - Confirm evaluation that shows time is available to deploy ground water sump pumps as needed in critical locations in addition to the vicinity of the TDAFW pump.	Completed
3.1.1.4.A	Off Site Resources, seismic - Confirm development of the MNS playbook as well as identification of the local Assembly Area and routes to the plant.	Completed

3.1.3.1.A	Protection of FLEX equipment, high winds - Site specific data to justify the assumed tornado width of 1200 feet will be required to confirm the final locations of the FLEX storage facilities conform to NEI 12-06 guidance.	Completed
3.1.5.2.A	Deployment of FLEX equipment, high temperatures - Confirm that the storage facilities will be designed for extreme temperature ranges including concerns for expansion of sheet metal, swollen door seals, etc.	Completed
3.2.1.A	RCS Cooling and Heat Removal, and RCS Inventory Control Strategies - Justify the use of the analysis from Sections 5.2.1 and 5.2.2 of WCAP-17601-P by identifying and evaluating the important parameters and assumptions demonstrating that they are representative of MNS and appropriate for simulating the ELAP transient.	Completed
3.2.1.1.A	Computer Code Used for ELAP Analysis – Confirm that reliance on the NOTRUMP code for the ELAP analysis of Westinghouse plants is limited to the flow conditions prior to reflux condensation initiation. This includes specifying an acceptable definition for reflux condensation cooling.	Completed
3.2.1.2.A	RCP seals - Confirm that the RCP seal initial maximum leakage rate used in the analysis is greater than or equal to the upper bound expectation for the ELAP event (21 gpm/seal) discussed in the PWROG white paper addressing the RCP seal leakage for Westinghouse plants.	Completed
3.2.1.2.B	RCP seals - In some plant designs, such as those with 1200 to 1300 psia SG design pressures and no accumulator backing of the main steam system power-operated relief valve (PORV) actuators, the cold legs could experience temperatures as high as 580 degrees F before cooldown commences. This is beyond the qualification temperature (550 degrees F) of the O-rings used in the RCP seals. For those Westinghouse designs, a discussion of the information (including the applicable analysis and relevant seal leakage testing data) should be	Completed

	provided to justify that (1) the integrity of the associated O-rings will be maintained at the temperature conditions experienced during the ELAP event, and (2) the seal leakage rate of 21 gpm/seal used in the ELAP is adequate and acceptable.	
3.2.1.2.C	RCP seals - If the seals are changed to the newly designed Generation 3 SHIELD seals, or non-Westinghouse seals, the acceptability of the use of the newly designed Generation 3 SHIELD seals, or non-Westinghouse seals should be addressed, and the RCP seal leakages rates for use in the ELAP analysis should be provided with acceptable justification.	Completed
3.2.1.3.A	Decay Heat - Values of the following key parameters used to determine the decay heat should be specified and the adequacy of the values evaluated: (1) initial power level, (2) fuel enrichment, (3) fuel burnup, (4) effective full power operating days per fuel cycle, (5) number of fuel cycles, if hybrid fuels are used in the core, and (6) fuel characteristics are based on the beginning of the cycle, middle of the cycle, or end of the cycle.	Completed
3.2.1.4.A	Initial Values for Key Plant Parameters and Assumptions – Confirm results and appropriate actions subsequent to Westinghouse supplying McGuire with additional information regarding the key plant parameters and assumptions.	Completed
3.2.1.7.A	Confirm that MNS will abide by the generic resolution for shutdown and refueling concerns.	Completed
3.2.3.A	Containment Functions Strategies - Confirm completion of the long term containment analysis and appropriate actions.	Completed
3.2.4.4.A	Lighting and Communications - Confirmation will be required that upgrades to the site's communications systems have been completed.	Completed

3.2.4.6.A	Ventilation for Equipment Cooling and Personnel Habitability - Room heat up response for specific MNS areas are completed but need to be evaluated by NRC personnel. Confirm completion of evaluation and appropriate actions.	Completed
3.2.4.7.A	Water Sources - Confirm that plant procedures specify that a flow path is promptly established for makeup flow to the steam generators and identify backup water sources in order of intended use; and that plant procedures/guidance should specify clear criteria for transferring to the next preferred source of water.	Completed
3.2.4.8.A	Electrical Power Sources - Confirm completion of Flex DG sizing calculation and appropriate actions.	Completed
3.2.4.9.A	Portable Equipment Fuel - Confirm completion of evaluation and appropriate actions to assess long-term FLEX equipment fuel oil requirements.	Completed
3.2.4.10.A	The battery sizing calculation needs to be verified when revised to show that dc power for 2 of 4 channels can be maintained for 24 hours without a charger in place.	Completed
3.2.4.10.B	Load Reduction to Conserve DC Power - Confirm that ELAP procedures/guidance will direct operators to conserve dc power during the event by stripping nonessential loads as soon as practical.	Completed
3.4.A	Offsite Resources - Confirm NEI 12-06, Section 12.2, Guidelines 2 through 10 are addressed with the RRC.	Completed

Section 6.d. NRC Onsite Mitigating Strategies Audit Report

Note: status of items marked "Pending" are industry issues in the process of being addressed by NRC or via the PWROG. Site-specific evaluation of these items has been completed.

Open Item #	Audit Report Open Item Description	Status
Attachment 3, Item ISE CI 3.1.1.4.A	Off-Site Resources: The NRC staff requests that the licensee provide a copy of the playbook once it's finalized.	Completed
Attachment 3, Item ISE CI 3.2.4.4.A	Lighting and Communications: The NRC requested that the licensee provide confirmation of the modifications to the communications systems once completed.	Completed
Attachment 3, Licensee Identified OIP Open Item 5	Process Connections: The NRC staff requested that the licensee provide a summary of the plant modifications to implement the FLEX strategy for staff review.	Completed
Attachment 3, Item ISE CI 3.2.3.A	Containment Functions Strategies: The licensee has provided a response to the question on the e-Portal; however, the staff requests that the calculations be posted on the e-Portal.	Completed
Attachment 3, Item ISE CI 3.2.4.9.A	Portable Equipment Fuel: Provide information on the fuel quality from the trucks that will be onsite to initially refuel FLEX equipment.	Completed
Attachment 3, SE Review Item 5	NOTRUMP Code: The staff performed a confirmatory ELAP analysis using TRACE and is currently reviewing the results between TRACE and NOTRUMP. In addition, the staff has requested a copy of the PWROG application of NOTRUMP white paper.	Pending Final Integrated Plan submittal
Attachment 3, SE Review Item 7	RCP Leakage Rate: The NRC staff understands that Westinghouse has recently recalculated seal leak-off line pressures under loss of seal cooling events based on a revised seal leakage model and additional design-specific information for certain plants. Please clarify whether the piping in your seal leak-off line is capable of withstanding the pressure predicted during an ELAP event according to the revised seal leakage model. If not, please discuss	Pending Final Integrated Plan submittal

	any planned modifications to the seal leak-off piping design and the associated completion timeline. Alternately, please clarify that the seal leakage rate would remain in an acceptable range if the affected seal leak-off piping were to rupture.	
Attachment 3, SE Review Item 8	<p>RCP Leakage Rate: Please provide adequate justification for the seal leakage rates calculated according to the Westinghouse seal leakage model that was revised following the issuance of NSAL-14-1. The justification should include a discussion of the following factors:</p> <ol style="list-style-type: none"> 1. Benchmarking of the seal leakage model against relevant data from tests or operating events, 2. Discussion of the impact on the seal leakage rate due to fluid temperatures greater than 550°F resulting in increased deflection at the seal interface, 3. Clarification whether the second-stage reactor coolant pump seal would remain closed under ELAP conditions predicted by the revised seal leakage model and a technical basis to support the determination, and, 4. Justification that the interpolation scheme used to compute the integrated leakage from the reactor coolant pump seals from a limited number of computer simulations (e.g., three) is realistic or conservative. 	Pending Final Integrated Plan submittal
Attachment 4, Item ISE CI 3.2.4.10.A	Battery Sizing Calculations: The staff will complete a vendor audit of the batteries.	Pending Final Integrated Plan submittal
Attachment 4, Item ISE CI 3.4.A	Off-Site Resources: The NRC staff discussed with the licensee its plan to address minimum capabilities of off-site resources, outlined in the 10 guidelines in NEI 12-06. The licensee indicated that the National SAFER Response Center generated a generic response to address the guidelines, and coordination of McGuire strategies with the National SAFER Response Centers is ongoing. During the onsite audit, the licensee provided a copy of the generic response and the NRC staff is still in the process of reviewing the document.	Completed

Attachment 4, AQ 35	Loss of Heat Tracing Effects, NEI 12-06, Section 3.2.2, Guideline 12: The staff is currently reviewing the licensee's response on the e-Portal.	Completed
Attachment 4, Item ISE CI 3.2.1.7.A	Shutdown and Refueling Modes: During the onsite audit, the licensee provided a copy of the PWROG interim generic guidance that identified minimal coping strategies for PWRs when an ELAP event occurs in a shutdown mode, and the NRC staff is still in the process of reviewing the document.	Pending Final Integrated Plan submittal

7 Potential Interim Staff Evaluation Impacts

There are no potential impacts to the Interim Staff Evaluation identified at this time.

8 References

The following references support the updates to the Overall Integrated Plan described in this enclosure.

1. McGuire Nuclear Station Overall Integrated Plan in Response to March 12, 2012, Commission Order to Modify Licenses With Regard to Requirements for Mitigation Strategies for Beyond Design Basis External Events (Order EA-12-049), dated February 28, 2013, (ADAMS Accession No. ML13063A185).
2. NRC Order Number EA-12-049, "Order Modifying Licenses with Regard to Requirements for Mitigation Strategies for Beyond-Design-Basis External Events," dated March 12, 2012, (ADAMS Accession No. ML12054A735).
3. McGuire Nuclear Station, First Six-Month Status Report in Response to March 12, 2012, Commission Order Modifying Licenses With Regard to Requirements for Mitigation Strategies for Beyond-Design-Basis External Events (Order Number EA-12-049), dated August 28, 2013, (ADAMS Accession No. ML13254A204).
4. William B. McGuire Nuclear Station, Units 1 and 2—Interim Staff Evaluation Relating to Overall Integrated Plan in Response to Order EA-12-049 (Mitigation Strategies) dated January 16, 2014, (ADAMS Accession No. ML13338A421).
5. McGuire Nuclear Station, Second Six-Month Status Report in Response to March 12, 2012, Commission Order Modifying Licenses With Regard to Requirements for Mitigation Strategies for Beyond-Design-Basis External Events (Order Number EA-12-049), dated February 27, 2014, (ADAMS Accession No. ML14073A462).
6. McGuire Nuclear Station, Third Six-Month Status Report in Response to March 12, 2012, Commission Order Modifying Licenses With Regard to Requirements for Mitigation Strategies for Beyond-Design-Basis External Events (Order Number EA-12-049), dated August 27, 2014, (ADAMS Accession No. ML14253A188).

7. McGuire Nuclear Station, Fourth Six-Month Status Report in Response to March 12, 2012, Commission Order Modifying Licenses With Regard to Requirements for Mitigation Strategies for Beyond-Design-Basis External Events (Order Number EA-12-049), dated February 26, 2015, (ADAMS Accession No. ML15075A016).
8. NSAL-14-1, Revision 1 "Impact of Reactor Coolant Pump No. 1 Seal Leakoff Piping on Reactor Coolant Pump Seal Leakage During a Loss of All Seal Cooling," dated September 9, 2014.
9. WCAP-17601-P, Revision 1 "Reactor Coolant System Response to ELAP for Westinghouse, Combustion Engineering and B&W NSSS Designs", January 2013, (ADAMS Accession No. ML13042A010).
10. NEI letter "EA-12-049 Mitigating Strategies Resolution of Extended Battery Duty Cycles Generic Concern" dated August 27, 2013, (ML 13241A186), and NRC endorsement letter dated September 16, 2013, (ADAMS Accession No. ML13241A188).
11. EP/1/A/5000/ECA-0.0, Revision 35.
12. McGuire Nuclear Station, Units 1 and 2 - Report for the Audit Regarding Implementation of Mitigating Strategies and Reliable Spent Fuel Pool Instrumentation Related to Orders EA-12-049 and EA-12-051, dated October 9, 2014, (ADAMS Accession No. ML14241A454).
13. Notification of Full Compliance with Order EA-12-049, "Order Modifying Licenses with Regard to Requirements for Mitigation Strategies for Beyond Design Basis External Events" and with Order EA-12-051, "Order to Modify Licenses with Regard to Reliable Spent Fuel Pool Instrumentation" - McGuire Nuclear Station Unit 1, dated November 18, 2014, (ADAMS Accession No. ML14335A322).