

# REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 7912130411 DOC. DATE: 79/12/10 NOTARIZED: NO  
 FACIL: 50-269 Oconee Nuclear Station, Unit 1, Duke Power Co.  
 50-270 Oconee Nuclear Station, Unit 2, Duke Power Co.  
 50-287 Oconee Nuclear Station, Unit 3, Duke Power Co.  
 AUTH. NAME AUTHOR AFFILIATION  
 PARKER, W.O. Duke Power Co.  
 RECIP. NAME RECIPIENT AFFILIATION

DUCKET #  
 05000269  
 05000270  
 05000287

SUBJECT: R0: on 791126-30, only one standby bus had been energized by  
 LEE Station gas turbine, although Tech Specs require both  
 standby buses be energized during Keowee hydro outage.  
 Caused by inadequately revised procedures.

DISTRIBUTION CODE: A0155 COPIES RECEIVED: LTR L ENCL 0 SIZE: 2  
 TITLE: Unsite Emergency Power Systems

NOTES: M. CUNNINGHAM - ALL AMENDS TO FSAR & CHANGES TO TECH  
SPECS.

	RECIPIENT		COPIES			RECIPIENT		COPIES	
	ID CODE/NAME		LTTR	ENCL		ID CODE/NAME		LTTR	ENCL
ACTION:	05 BC <u>ORB #4</u>		7	1		LA <u>ORB #4</u>		1	0
INTERNAL:	01 <u>REG FILE</u>		1	1		02 NRC PDR		1	1
	12 TA/EDD		1	1		13 I&E		2	2
	15 OELD		1	1		16 MPA		1	1
	17 AUXIL SYS BR		1	1		18 I&C SYS BR		1	1
	20 ENGR BR		1	1		21 REAC SFTY BR		1	1
	22 PLANT SYS BR		1	1		23 ADV REAC BR		1	1
	24 PWR SYS BR		1	1		25 WAMBACH T		1	1
	26 TUNDI, D		1	1		27 MCDONALD, D		1	1
	29 BRINKMAN		1	1		30 EEB		1	1
	S HANAUER		1	1					
EXTERNAL:	03 LPDR		1	1		04 NSIC		1	1
	31 ACRS		16	16					

DEC 14 1979

TOTAL NUMBER OF COPIES REQUIRED: LTTR 46 ENCL 0

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# DUKE POWER COMPANY

POWER BUILDING

422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28242

WILLIAM O. PARKER, JR.  
VICE PRESIDENT  
STEAM PRODUCTION

December 10, 1979

TELEPHONE: AREA 704  
373-4083

Mr. Harold R. Denton, Director  
Office of Nuclear Reactor Regulation  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Attention: Mr. Robert W. Reid, Chief  
Operating Reactors Branch No. 4

Re: Oconee Nuclear Station  
Docket Nos. 50-269, -270, -287

Dear Sir:

This report is submitted pursuant to Oconee Nuclear Station Technical Specification 3.7.8 as a result of degradation in the emergency electrical power system beyond that permitted by the Technical Specifications. My letter of December 5, 1979 addressed the delays involved in the preparation of this report.

At 0609 on November 26, 1979 both Keowee Hydro units were removed from service for maintenance. Two Lee Station gas turbines had been started the previous day to energize the Oconee standby buses during the Keowee outage. Keowee Unit 1 was returned to service on November 29, 1979, but Unit 2 remained out of service for further maintenance, as described in my letter of November 7, 1979 to Mr. James P. O'Reilly. At 1551 on November 30 as a result of performance of a periodic test to verify proper alignment, it was discovered that only one of the two standby buses was energized by the Lee turbine, although Technical Specification 3.7.4 requires both standby buses to be energized when a Keowee unit is out of service. The open standby bus breaker was then closed. The operating procedure for the 100 KV power supply, by which the Oconee standby buses are energized from the Lee combustion units, had been revised recently to require the Lee units to carry a load from Oconee in order to improve the gas turbine reliability. However, the revision failed to retain instructions to have both standby buses energized. The procedure has been revised to assure that both buses are energized during Keowee outages.

During the period from November 26 to November 30 that only one standby bus was energized, Oconee 1 was in a refueling shutdown, and Oconee 2 and 3 were at approximately full power. In the unlikely event that a Lee combustion unit had been required to supply emergency power, the Engineered Safeguards switchgear for

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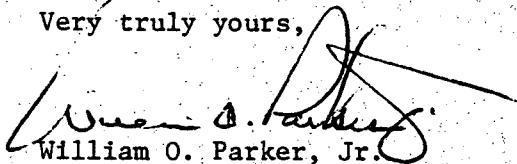
7912130 417

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Mr. Harold R. Denton, Director  
December 10, 1979  
Page Two

each Oconee unit would have been energized by Standby Bus No. 2. In addition, the breaker for Standby Bus No. 1 could have been closed manually from the Control Room if desired. Therefore, this incident is not considered to be significant with respect to safe operation, and the health and safety of the public were not affected.

Very truly yours,



William O. Parker, Jr.

SRL:scs

cc: Mr. James P. O'Reilly, Director  
U. S. Nuclear Regulatory Commission  
Region II  
101 Marietta Street, Suite 3100  
Atlanta, Georgia 30303

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DISTRIBUTION CODE: A0155 COPIES RECEIVED: LTR 1 ENCL 0 SIZE: 2  
 TITLE: Onsite Emergency Power Systems

NOTES: M. CUNNINGHAM - ALL AMENDS TO FSAR & CHANGES TO TECH SPECS.

	RECIPIENT ID CODE/NAME	COPIES LTR ENCL	RECIPIENT ID CODE/NAME	COPIES LTR ENCL
ACTION:	05 BC ORB #4	7 1	LA ORB #4	1 0
INTERNAL:	01 REG FILE	1 1	02 NRC PDR	1 1
	12 T&A/EDD	1 1	13 I&E	2 2
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	26 TUNDI, D	1 1	27 McDONALD, D	1 1
	29 BRINKMAN	1 1	30 EEB	1 1
	S HANAUER	1 1		
EXTERNAL:	03 LPDR	1 1	04 NSIC	1 1
	31 ACRS	16 16		

DEC 14 1979

TOTAL NUMBER OF COPIES REQUIRED: LTR 46 ENCL 0

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**LICENSEE EVENT REPORT**

EXHIBIT A

CONTROL BLOCK:		(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)																																																																																																																																																																																																																
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EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)																																																																																																																																																																																																																		
02		During a routine inspection of the Turbine Building basement while Oconee 3																																																																																																																																																																																																																
03		was at 74% full power, excessive packing leakage from LPSW pump 3B was dis-																																																																																																																																																																																																																
04		covered. LPSW pump 3A was started and pump 3B was secured and repaired. Since																																																																																																																																																																																																																
05		only one pump is required for normal or emergency conditions, one pump is																																																																																																																																																																																																																
06		permitted to be removed from service for up to 24 hours. Therefore, this																																																																																																																																																																																																																
07		incident was of no significance with respect to safe operation, and the																																																																																																																																																																																																																
08		health and safety of the public were not affected.																																																																																																																																																																																																																
7 8 9		80																																																																																																																																																																																																																
09		SYSTEM CODE										CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE					COMP. SUBCODE		VALVE SUBCODE																																																																																																																																																																																											
7 8		W   A   11										E   12		B   13		P   U   M   P   X   X   14					B   15		Z   16																																																																																																																																																																																											
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17		LER/RO REPORT NUMBER		EVENT YEAR		SEQUENTIA REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION NO.																																																																																																																																																																																																						
7 8		01		7 9		0 1 2		0 3		L		0																																																																																																																																																																																																						
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ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED		NPP-4 FORM SUB.		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER																																																																																																																																																																																																		
7 8		D   18		Z   19		Z   20		Z   21		0   0   0   0   22		Y   23		Y   24		L   25		I   0   7   5   26																																																																																																																																																																																																
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CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)																																																																																																																																																																																																																		
10		The pump's outboard shaft sleeve was discovered to have worked loose, damaging																																																																																																																																																																																																																
11		the pump's packing. Since no damage was observed which could have allowed																																																																																																																																																																																																																
12		the sleeve to work loose, the lock nut may not have been tightened properly.																																																																																																																																																																																																																
13		The pump was repaired and after a successful performance test, returned to																																																																																																																																																																																																																
14		service less than 12 hours after being declared inoperable.																																																																																																																																																																																																																
7 8 9		80																																																																																																																																																																																																																
15		FACILITY STATUS										% POWER		OTHER STATUS		METHOD OF DISCOVERY					DISCOVERY DESCRIPTION																																																																																																																																																																																													
7 8		E   28										0   7   4   29		NA		B   31					Routine Turbine Building Inspection																																																																																																																																																																																													
7 8		9										10		11		12					13					14					15					16					17					18					19					20					21					22					23					24					25					26					27					28					29					30					31					32					33					34					35					36					37					38					39					40					41					42					43					44					45					46					47					48					49					50				
16		ACTIVITY RELEASED										CONTENT		AMOUNT OF ACTIVITY		LOCATION OF RELEASE																																																																																																																																																																																																		
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17		PERSONNEL EXPOSURES										NUMBER		TYPE		DESCRIPTION																																																																																																																																																																																																		
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Corrective Action:

The immediate corrective action was to start LPSW pump 3A and secure pump 3B. The outboard shaft sleeve was repositioned correctly, and the packing was adjusted. The pump was declared operable after successful completion of the performance test. Since the procedure for tightening the shaft sleeve lock nut is adequate and since no similar problems have been encountered previously, this appears to be an isolated occurrence, and no further corrective actions are considered to be necessary